**PSEG Nuclear LLC** P.O. Box 236, Hancocks Bridge, New Jersey 08038-0236

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United States Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Gentlemen:

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REQUEST FOR CHANGE TO TECHNICAL SPECIFICATIONS REQUIREMENTS SUPPLEMENTAL REQUEST EMERGENCY CORE COOLING SYSTEMS SURVEILLANCE HOPE CREEK GENERATING STATION FACILITY OPERATING LICENSE NPF-57 DOCKET NO. 50-354

On January 8, 2001, PSEG Nuclear LLC submitted a request for a revision to the Technical Specifications (TS) to reduce the surveillance test values for the core spray flow in Section 4.5.1.b.1 for the Hope Creek Generating Station (HC). The information contained in this letter documents our response to a request by the Nuclear Regulatory Commission's Hope Creek Project Manager, Mr. Richard Ennis, during a telephone conference on November 19, 2001. In accordance with 10CFR50.91(b)(1), a copy of this submittal has been sent to the State of New Jersey.

Should you have any questions regarding this request, please contact Mr. John Nagle at 856-339-3171.

Sincerely Garthow

David F. Garchow Vice President - Operations

Attachment



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I declare under penalty of perjury that the foregoing is true and correct.

Executed on \_\_\_\_\_

Garchow Vige President - Operations

C: Mr. H. Miller, Administrator – Region I U. S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406

> Mr. R. Ennis, Project Manager-Hope Creek U. S. Nuclear Regulatory Commission One White Flint North Mail Stop 08B1 11555 Rockville Pike Rockville, MD 20852

USNRC Senior Resident Inspector (X24)

Mr. K. Tosch, Manager IV Bureau of Nuclear Engineering PO Box 415 Trenton, New Jersey 08625 Document Control Desk LR-N01-0398 Attachment

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## HOPE CREEK GENERATING STATION FACILITY OPERATING LICENSE NPF-57 DOCKET NO. 50-354 REVISIONS TO THE TECHNICAL SPECIFICATIONS (TS)

The following two question were discussed in the Nov. 19, 2001 telephone conference:

1. What is the impact of reduced flow on long term cooling?

For the reduced flow case the required flow rate to support core cooling is adequately achieved at 0 psid (i.e. the vessel condition for long term cooling). This information is clearly shown in Figure 1 of the analysis attached to the original submittal. This figure plots flow versus dp. Long term cooling is represented by the core spray flow rate corresponding to zero dp (0 psid) on this graph.

2. What changes will be made to the IST test for the core spray pumps?

Based on discussions with engineering and the IST group, there will be no change to the testing.