



**Duke Energy**

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Vice President

November 29, 2001

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Subject: Oconee Nuclear Site Docket No. 50-287  
Core Operating Limits Report (COLR)

Gentlemen:

Attached, pursuant to Oconee Technical Specifications 5.6.5, is an information copy of a revision to the Core Operating Limits Report for Oconee Unit 3, Cycle 20, Rev. 15.

Very truly yours,

W. R. McCollum, Site Vice President  
Oconee Nuclear Site

Attachment

ADD1

NRC Document Control Desk  
November 29, 2001  
Page 2

xc w/att: Mr. L. A. Reyes, Regional Administrator  
U. S. Nuclear Regulatory Commission, Region II

Mr. L. N. Olshan, Project Manager  
Office of Nuclear Reactor Regulation

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Senior Resident Inspector  
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 CORE OPERATING LIMITS REPORT

Page 1 of 1

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Oconee 3 Cycle 20

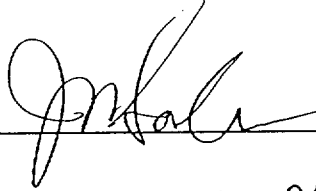
Core Operating Limits Report

QA Condition 1


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REVIEWED AND APPROVED BY CFAM 3.13


~~Not Reviewed or Approved by CFAM 3.13~~

Prepared By : J. Mark Sanders 

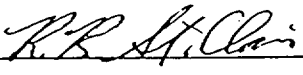
Date : 20 Nov 2001

Checked By : G. M. Presnell 

Date : Nov. 20, 2001

CDR By : M. W. Scott 

Date : Nov. 20, 2001

Approved By : R. R. StClair 

Date : Nov. 20, 2001

Oconee 3 Cycle 20  
Core Operating Limits Report

Insertion Sheet for Revision 15

This revision is not valid until the end of operation for Oconee 3 Cycle 19.

Remove these Revision 14 pages

1 - 3

Insert these Revision 15 pages

1 - 3

Revision Log

Revision	Effective Date	Pages Revised	Pages Added	Pages Deleted	Total Effective Pages
<b>Oconee 3 Cycle 20 revisions below</b>					
15	Nov-01	1 - 3	-	-	31
14	Nov-01	1 - 31	-	-	31
<b>Oconee 3 Cycle 19 revisions below</b>					
13	Apr-00	1 - 31	-	-	31
<b>Oconee 3 Cycle 18 revisions below</b>					
12	Feb-00	1 - 4	-	-	31
11	Jun-99	1-3, 31	-	-	31
10	Mar-99	1 - 31	-	32 - 38	31
9	Oct-98	1 - 38	-	-	38
<b>Oconee 3 Cycle 17 revisions below</b>					
8	Mar-98	1, 2, 3, 5, 13 16, 17, 32, 36			38
7	Dec-96	1 - 38	-	-	38
<b>Oconee 3 Cycle 16 revisions below</b>					
6	Sep-95	1, 2, 3, 9, 28, 29, 30 31	-	-	38
5	Jun-95	1, 2, 3, 7	-	-	38
4	May-95	1 - 33	34 - 38	-	38

## Oconee 3 Cycle 20

### 1.0 Error Adjusted Core Operating Limits

The Core Operating Limits Report for O3C20 has been prepared in accordance with the requirements of ITS 5.6.5. The core operating limits within this report have been developed using NRC approved methodology identified in references 1 through 10. The RPS protective limits and maximum allowable setpoints are documented in references 11 through 13. These limits are validated for use in O3C20 by references 14 through 16. The O3C20 analyses assume a design flow of 107.5% of 88,000 gpm per RCS pump, radial local peaking ( $F_{\Delta h}$ ) of 1.714, an axial peaking factor ( $F_z$ ) of 1.5, and an EOC ( $\leq 100$  ppmB) Tav<sub>g</sub> reduction of up to 10 °F provided 4 RCPs are in operation and Tav<sub>g</sub> does not decrease below 569 °F.

The error adjusted core operating limits included in section 1 of the report incorporate all necessary uncertainties and margins required for operation of the O3C20 reload core.

### 1.1 References

1. Nuclear Design Methodology Using CASMO-3 / SIMULATE-3P, DPC-NE-1004P-A, SER dated November 23, 1992.
2. Oconee Nuclear Station Reload Design Methodology II, DPC-NE-1002A, Revision 1, SER dated October 1, 1985.
3. Oconee Nuclear Station Reload Design Methodology, NFS-1001A, Revision 4, SER dated July 29, 1981.
4. Oconee Nuclear Station Core Thermal Hydraulic Methodology Using VIPRE-01, DPC-NE-2003P-A, SER dated July 19, 1989.
5. Thermal Hydraulic Statistical Core Design Methodology, DPC-NE-2005P-A, Revision 2, SER dated June 8, 1999.
6. Fuel Mechanical Reload Analysis Methodology Using TACO3, DPC-NE-2008P-A, SER dated April 3, 1995.
7. UFSAR Chapter 15 Transient Analysis Methodology, DPC-NE-3005-PA, Revision 1, SER dated May 25, 1999.
8. DPC-NE-3000P-A, Thermal Hydraulic Transient Analysis Methodology, Rev. 2, SER dated October 14, 1998.
9. BAW-10192-PA, BWNT LOCA - BWNT Loss of Coolant Accident Evaluation Model for Once-Through Steam Generator Plants, SER dated February 18, 1997.
10. BAW-10227-PA, Evaluation of Advanced Cladding and Structural Material (M5) in PWR Reactor Fuel, SER dated December 14, 1999.
11. Variable Low Pressure Safety Limit, OSC-4048, Revision 3, July 1998.
12. Power Imbalance Safety Limits and Tech Spec Setpoints Using Error Adjusted Flux-Flow Ratio of 1.094, OSC-5604, Revision 1, November 1998.
13.  $\Delta T_c$  and EOC Reduced Tav<sub>g</sub> Operation, OSC-7265, Rev. 0, Duke Power Co., April 2001.
14. O3C20 Maneuvering Analysis, OSC-7727, Revision 3, November 2001.
15. O3C20 Specific DNB Analysis, OSC-7845, Revision 0, June 2001.
16. O3C20 Reload Safety Evaluation & 10CFR50.59, OSC-7959, Revision 1, November 2001.