



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

July 14, 1998

Mr. J. A. Scalice
Chief Nuclear Officer
and Executive Vice President
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

*SEE AMENDS TO DL
7 binders*

SUBJECT: AMENDMENT NOS. 234, 253 AND 212 TO FACILITY OPERATING LICENSE NOS. DPR-33, DPR-52, AND DPR-68: REGARDING CONVERSION TO IMPROVED STANDARD TECHNICAL SPECIFICATIONS FOR THE BROWNS FERRY NUCLEAR PLANT, UNITS 1, 2, AND 3 (TAC NOS. M96431, M96432, AND M96433)

Dear Mr. Scalice:

The Commission has issued the enclosed Amendment Nos. 234, 253 and 212 to Facility Operating License Nos. DPR-33, DPR-52, and DPR-68 for the Browns Ferry Nuclear Plant (BFN), Units 1, 2, and 3, respectively. These amendments are in response to your application dated September 6, 1996, and supplements which proposed technical specification (TS) changes for conversion to Improved Technical Specifications (ITS) based on NUREG-1433, "Standard Technical Specifications, General Electric Plants BWR/4," Revision 1, dated April 1995, and on guidance provided in the Commission's Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors, published on July 22, 1993 (58 FR 39132).

In addition, the U.S. Nuclear Regulatory Commission (NRC) staff has reviewed and approved several TS changes which, although included in your improved TS conversion submittal, were identified as being beyond the scope of the conversion. They are related to:

1. Plant specific application of the NRC approved generic methodology (BWR Owners Group topical report) supporting instrument surveillance intervals and allowed outage times.
2. Elimination of the zonal concept (three reactor zones and a refueling zone) within the secondary containment zone and instead establish a single secondary containment zone for secondary containment Isolation.
3. Number of residual heat removal service water system pumps required to be operable under certain conditions.
4. Number of emergency core cooling system subsystems required to be operable during shutdown to provide sufficient makeup capability.
5. For BFN Unit 2 only, installation of the Power Range Neutron Monitoring System, Average Power Range Monitor and Rod Block Monitor TS improvements, and the Maximum Extended Load Line Limit analysis.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

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Mr. J. A. Scalice
Chief Nuclear Officer
and Executive Vice President
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Dear Mr. Scalice:

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2. Elimination of the zonal concept (three reactor zones and a refueling zone) within the secondary containment zone and instead establish a single secondary containment zone for secondary containment Isolation.
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5. For BFN Unit 2 only, installation of the Power Range Neutron Monitoring System, Average Power Range Monitor and Rod Block Monitor TS improvements, and the Maximum Extended Load Line Limit analysis.

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6. Revising reactor vessel water level to specify that the level be maintained greater than the top of the active irradiated fuel instead of specifying actual water level.
7. Deleting the average U-235 enrichment of [4.5] weight percent requirement from the spent and the new fuel storage racks design.
9. TS changes to allow spiral off-load procedures and adopt a revision to surveillance requirement for count rate verification during spiral loading.
10. Extension of the surveillance frequency for reactivity anomalies.
11. Changing the Surveillance Requirements (SRs) for BFN Unit 3 only, to allow position verification of either the motor-operated or the manual shutoff valve for ensuring that the Low Pressure Coolant Injection System cross-tie flow path is isolated.
12. Changing the calibration frequency of the low power range monitors to every 1000MWD/T instead of every 1000 effective-full-power-hours average core exposure to match NUREG-1433.

Under the subject amendments, additional conditions are added to Appendix B of the BFN Units 1, 2 and 3 facility licenses. These additional conditions are related to relocation of certain requirements from the TS to licensee-controlled documents, performance of surveillances in accordance with the ITS, and validation of ITS for BFN Unit 1.

The relocated requirements are not required to be in the TS under 10 CFR 50.36 and do not meet any of the four criteria in the "NRC Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors," published on July 22, 1993 (58 FR 39132) and 10 CFR 50.36, as amended July 19, 1995 (60 FR 36953). The NRC staff has concluded that appropriate controls have been established for all of the current specifications, information, and requirements that are being moved to licensee-controlled documents. The license condition is adopted herewith, to require incorporation of these matters in the appropriate licensee-controlled document(s). Following implementation, the NRC will audit the removed provisions to ensure that an appropriate level of control has been achieved. Accordingly, these TS, information, and requirements, as described in detail in the enclosed Safety Evaluation, may be relocated from current TS and placed in the Updated Final Safety Analysis Report or other licensee-controlled documents as specified in letter dated September 6, 1996 and supplements.

For SRs that are new in Amendments 234, 253 and 212 to Final Operating Licenses DPR-33, DPR-52, and DPR-68, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment. For SRs that existed prior to Amendments 234, 253, and 212, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendments.

With respect to BFN Unit 1 ITS, an additional condition is imposed which requires Tennessee Valley Authority (TVA) to validate the Unit 1 ITS, and submit the validation, and any changes to the ITS, to the NRC for its review and approval before entering mode 5.

Additionally, our safety evaluation (SE) associated with the subject amendments, reflects expiration of previous one time license Amendment Nos. 250 and 209 for BFN Units 2 and 3, respectively, relating to allowable outage time for each of the emergency diesel generators (EDGs) to accommodate a vendor recommended 12-year EDG maintenance. By letter dated

May 27, 1998, TVA indicated that the EDG maintenance for all eight EDGs has been successfully completed and, therefore, the one-time license amendments are no longer necessary. Also, the license conditions, 2(C)15 and 2D(5), for operating Licenses DPR- 52 and DPR-68 for BFN Units 2 and 3, respectively, associated with these one-time license amendments are deleted.

A copy of the related SE is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice (enclosed).

Sincerely,

Original signed by:

L. Raghavan, Senior Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-259, 50-260 and 50-296
Serial No. BFN-98-007

- Enclosures: 1. Amendment No. 234 to License No. DPR-33
- 2. Amendment No. 253 to License No. DPR-52
- 3. Amendment No. 212 to License No. DPR-68
- 4. Safety Evaluation
- 5. Notice

cc w/enclosures: See next page

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J. A. Scalice

- 3 -

May 27, 1998, TVA indicated that the EDG maintenance for all eight EDGs has been successfully completed and, therefore, the one-time license amendments are no longer necessary. Also, the license conditions, 2(C)15 and 2D(5), for operating Licenses DPR- 52 and DPR-68 for BFN Units 2 and 3, respectively, associated with these one-time license amendments are deleted.

A copy of the related SE is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice (enclosed).

Sincerely,



L. Raghavan, Senior Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-259, 50-260 and 50-296
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4. Safety Evaluation
5. Notice

cc w/enclosures: See next page

Mr. J. A. Scalice
Tennessee Valley Authority

BROWNS FERRY NUCLEAR PLANT

cc:

Senior Vice President
Nuclear Operations
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Mr. Mark J. Burzynski, Manager
Nuclear Licensing
Tennessee Valley Authority
4X Blue Ridge
1101 Market Street
Chattanooga, TN 37402-2801

Mr. Jack A. Bailey, Vice President
Engineering & Technical Services
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Mr. Timothy E. Abney, Manager
Licensing and Industry Affairs
Browns Ferry Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Decatur, AL 35609

Mr. C. M. Crane, Site Vice President
Browns Ferry Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Decatur, AL 35609

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
61 Forsyth Street, SW., Suite 23T85
Atlanta, GA 30303-3415

General Counsel
Tennessee Valley Authority
ET 10H
400 West Summit Hill Drive
Knoxville, TN 37902

Mr. Leonard D. Wert
Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Browns Ferry Nuclear Plant
10833 Shaw Road
Athens, AL 35611

Mr. Raul R. Baron, General Manager
Nuclear Assurance
Tennessee Valley Authority
5M Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

State Health Officer
Alabama Dept. of Public Health
434 Monroe Street
Montgomery, AL 36130-1701

Mr. Karl W. Singer, Plant Manager
Browns Ferry Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Decatur, AL 35609

Chairman
Limestone County Commission
310 West Washington Street
Athens, AL 35611



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-259

BROWNS FERRY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 234
License No. DPR-33

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee), dated September 6, 1996, 1996, as supplemented December 11, 1996, April 11, May 1, August 14, October 15, November 5 and 14, December 3, 4, 11, 22, 23, 29, and 30, 1997, January 23, March 12 and 13, April 16, 20, and 28, May 7, 14, 19, and 27, and June 2, 5, 10 and 19, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-33 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 234, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

For Surveillance Requirements (SRs) that are new in Amendment 234 to Final Operating License DPR-33, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment. For SRs that existed prior to Amendment 234, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendment.

3. Additional Conditions

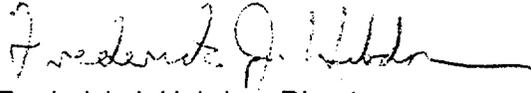
The Additional Conditions contained in Appendix B, as revised through Amendment No. 234, are hereby incorporated into this license. Tennessee Valley Authority shall operate the facility in accordance with the Additional Conditions.

<u>Amend. Number</u>	<u>Additional Conditions</u>	<u>Implementation Date</u>
234	The licensee is authorized to relocate certain requirements included in Appendix A and the former Appendix B to licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the appropriate documents, as described in the licensee's application dated September 6, 1996, as supplemented December 11, 1996, April 11, May 1, August 14, October 15, November 5 and 14, December 3, 4, 11, 22, 23, 29, and 30, 1997, January 23, March 12 and 13, April 16, 20, and 28, May 7, 14, 19, and 27, and June 2, 5, 10 and 19, 1998, evaluated in the NRC staff's Safety Evaluation enclosed with this amendment.	This amendment is effective immediately and shall be implemented within 90 days of the date of this amendment.
234	The licensee shall review the Technical Specification (TS) changes made by License Amendment No.234 and any subsequent TS changes, verify that the required analyses and modifications needed to support the changes are complete, and submit them for NRC review and approval prior to entering the mode for which the TS applies.	This amendment is effective immediately and shall be implemented prior to entering the mode for which the TS applies..

Appendix B to Facility Operating License No. DPR-33, is replaced by this amendment with Attachment 2.

4. This license amendment is effective as of its date of issuance and shall be implemented within 90 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment 1: Page 3 of License DPR-33
Attachment 2: Appendix B
Attachment 3: Changes to the Technical
Specifications

Date of Issuance: July 14, 1998

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or equipment and instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 3293 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 234 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

For Surveillance Requirements (SRs) that are new in Amendment 234 to Final Operating License DPR-33, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment. For SRs that existed prior to Amendment 234, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendment.

(3) Deleted.

(4) Deleted.

(5) The facility may be modified by plugging the bypass flow holes in the lower core support plate as described in Browns Ferry Nuclear Plant Units 1 and 2 Safety Analysis Report for Plant Modifications to Eliminate Significant In-Core Vibrations (NEDC-21091), October 1975. The reactor shall not be operated with the plugs installed in the lower core support plate bypass flow holes without further authorization by the NRC.

APPENDIX B
ADDITIONAL CONDITIONS

<u>Amend. Number</u>	<u>Additional Conditions</u>	<u>Implementation Date</u>
234	<p>The licensee is authorized to relocate certain requirements included in Appendix A and the former Appendix B to licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the appropriate documents, as described in the licensee's application dated September 6, 1996, as supplemented December 11, 1996, April 11, May 1, August 14, October 15, November 5 and 14, December 3, 4, 11, 22, 23, 29, and 30, 1997, January 23, March 12 and 13, April 16, 20, and 28, May 7, 14, 19, and 27, and June 2, 5, 10 and 19, 1998, evaluated in the NRC staff's Safety Evaluation enclosed with this amendment.</p>	<p>This amendment is effective immediately and shall be implemented within 90 days of the date of this amendment.</p>
234	<p>The licensee shall review the Technical Specification (TS) changes made by License Amendment No.234 and any subsequent TS changes, verify that the required analyses and modifications needed to support the changes are complete, and submit them for NRC review and approval prior to entering the mode for which the TS applies.</p>	<p>This amendment is effective immediately and shall be implemented prior to entering the mode for which the TS applies.</p>



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-260

BROWNS FERRY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 253
License No. DPR-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee), dated September 6, 1996, 1996, as supplemented December 11, 1996, April 11, May 1, August 14, October 15, November 5 and 14, December 3, 4, 11, 22, 23, 29, and 30, 1997, January 23, March 12 and 13, April 16, 20, and 28, May 7, 14, 19, and 27, and June 2, 5, 10 and 19, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 253, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

For Surveillance Requirements (SRs) that are new in Amendment 253 to Final Operating License DPR-52, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment. For SRs that existed prior to Amendment 253, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendment.

3. Additional Conditions

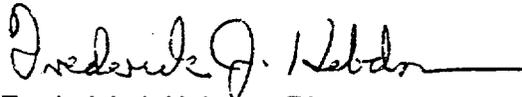
The Additional Conditions contained in Appendix B, as revised through Amendment No. 253, are hereby incorporated into this license. Tennessee Valley Authority shall operate the facility in accordance with the Additional Conditions.

<u>Amend. Number</u>	<u>Additional Conditions</u>	<u>Implementation Date</u>
253	The licensee is authorized to relocate certain requirements included in Appendix A and the former Appendix B to licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the appropriate documents, as described in the licensee's application dated September 6, 1996, as supplemented December 11, 1996, April 11, May 1, August 14, October 15, November 5 and 14, December 3, 4, 11, 22, 23, 29, and 30, 1997, January 23, March 12 and 13, April 16, 20, and 28, May 7, 14, 19, and 27, and June 2, 5, 10 and 19, 1998, evaluated in the NRC staff's Safety Evaluation enclosed with this amendment.	This amendment is effective immediately and shall be implemented within 90 days of the date of this amendment.

Appendix B to Facility Operating License No. DPR-52, is replaced by this amendment with Attachment 2.

4. Also, the license is amended to delete license condition 2(c)15.
5. This license amendment is effective as of its date of issuance and shall be implemented within 90 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment 1: Pages 3, 6 and 7 of License DPR-52
Attachment 2: Appendix B
Attachment 3: Changes to the Technical
Specifications

Date of Issuance: June 14, 1998

- (2) Pursuant to the Act and 10 CFR Parts 40 and 70, to receive, possess, and use at any time source and special nuclear material as reactor fuel in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report as supplemented and amended;
 - (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or equipment and instrument calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 3293 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 253 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

For Surveillance Requirements (SRs) that are new in Amendment 253 to Final Operating License DPR-52, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment. For SRs that existed prior to Amendment 253, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendment.

- (12) The licensee is authorized to temporarily store low-level radioactive waste in an existing covered pavilion that is situated outside the security fence, as presently located, but inside the site exclusion area. The total amount of low-level waste to be stored shall not exceed 1320 curies of total activity. This authorization expires two years from the effective date of this amendment and is subject to all the conditions and restrictions in TVA's application dated January 21, 1980.
- (13) Commission Order dated March 25, 1983 is modified as follows: in Attachment 1, for item II.F.1.1 and II.F.1.2 change "12/31/84" to "Prior to startup in Cycle 6."
- (14) Browns Ferry Nuclear Plant shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for BFN as approved in the SEs dated December 8, 1988/ March 6, 1991, March 31, 1993, November 2, 1995 and Supplement dated November 3, 1989 subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- ~~(15) a. When emergency diesel generators are removed from service for up to 14 days for preventive maintenance under the provisions of Amendment No. 250, the licensee shall:~~
- ~~1. Require another offsite power source be available in addition to the requirements of Technical Specification 3.9.A.1, that two offsite sources be available.~~
 - ~~2. Restrict work activities affecting the ability to cross-tie the associated Unit 3 emergency diesel generator to the 4-kV shutdown board for the emergency diesel generator that is out of service.~~
 - ~~3. Restrict work activities on the 500-kV switchyard cross-tie breakers supporting the affected unit.~~
 - ~~4. No high risk switchyard maintenance will be scheduled while the emergency diesel generator is out of service. If emergent conditions require performance of such high risk activities, such activities shall be approved by the Plant Manager and the Operations Manager. "High risk switchyard maintenance" is defined as an activity that if a single error or problem occurs, a full reactor scram, transient requiring a reduction in reactor power, and/or an unplanned engineered safety features actuation requiring a report to the NRC within 4 hours, could occur.~~

~~b. These provisions apply during the time period from January 1, 1998 to February 1, 1999, or completion of preventive maintenance under the provisions of Amendment No. 20, whichever occurs first.~~

D. This amended license is effective as of the date of issuance and shall expire midnight on June 28, 2014.

FOR THE ATOMIC ENERGY COMMISSION

S/ A. Giambusso
A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Attachment:
Appendices A & B - Technical
Specifications

Date of Issuance: JUN 28, 1974

APPENDIX B
ADDITIONAL CONDITIONS

<u>Amend. Number</u>	<u>Additional Conditions</u>	<u>Implementation Date</u>
253	<p>The licensee is authorized to relocate certain requirements included in Appendix A and the former Appendix B to licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the appropriate documents, as described in the licensee's application dated September 6, 1996, as supplemented May 1, August 14, November 5 and 14, December 3, 4, 11, 22, 23, 29 and 30, 1997, January 23, March 12, April 16, 20 and 28, and May 7, 14, 19 and 27, and June 2, 5, 10 and 19, 1998, evaluated in the NRC staff's Safety Evaluation enclosed with this amendment.</p>	<p>This amendment is effective immediately and shall be implemented within 90 days of the date of this amendment.</p>



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-296

BROWNS FERRY NUCLEAR PLANT, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 212
License No. DPR-68

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated September 6, 1996, 1996, as supplemented December 11, 1996, April 11, May 1, August 14, October 15, November 5 and 14, December 3, 4, 11, 22, 23, 29, and 30, 1997, January 23, March 12 and 13, April 16, 20, and 28, May 7, 14, 19, and 27, and June 2, 5, 10 and 19, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 212, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

For Surveillance Requirements (SRs) that are new in Amendment 212 to Final Operating License DPR-68, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment. For SRs that existed prior to Amendment 212, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendment.

3. Additional Conditions

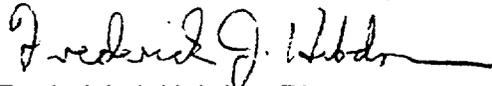
The Additional Conditions contained in Appendix B, as revised through Amendment No. 212, are hereby incorporated into this license. Tennessee Valley Authority shall operate the facility in accordance with the Additional Conditions.

<u>Amend. Number</u>	<u>Additional Conditions</u>	<u>Implementation Date</u>
212	The licensee is authorized to relocate certain requirements included in Appendix A and the former Appendix B to licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the appropriate documents, as described in the licensee's application dated September 6, 1996, as supplemented December 11, 1996, April 11, May 1, August 14, October 15, November 5 and 14, December 3, 4, 11, 22, 23, 29, and 30, 1997, January 23, March 12 and 13, April 16, 20, and 28, May 7, 14, 19, and 27, and June 2, 5, 10 and 19, 1998, evaluated in the NRC staff's Safety Evaluation enclosed with this amendment.	This amendment is effective immediately and shall be implemented within 90 days of the date of this amendment.

Appendix B to Facility Operating License No. DPR-68, is replaced by this amendment with Attachment 2.

4. Also, the license is amended to delete license condition 2(D)5.
5. This license amendment is effective as of its date of issuance and shall be implemented within 90 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment 1: Pages 3 and 6 of License DPR-68
Attachment 2: Appendix B
Attachment 3: Changes to the Technical
Specifications

Date of Issuance: June 14, 1998

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form for sample analysis or instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 3293 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 212 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

For Surveillance Requirements (SRs) that are new in Amendment 212 to Final Operating License DPR-68, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment. For SRs that existed prior to Amendment 212, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendment.

(3) Deleted.

- (2) The licensee is required, upon completion of the Mark I Owners Group containment long-term program related to relief valve operation, to make such modifications on a timely basis as may be necessary to restore the original design safety margins approved for the construction permit and used for the design of the torus structures when subjected to relief valve operation.
- (3) The facility may be modified as described in "Browns Ferry Nuclear Plant Unit 3 Emergency Core Cooling Systems Low Pressure Coolant Injection Modifications for Performance Improvement (October 1977)" and as described in TVA's letter of December 28, 1977 transmitting the aforementioned report and in TVA's supplemental letter of December 13, 1978.
- (4) Commission Order dated March 25, 1983 is modified as follows:
In Attachment 1, for item II.F.1.1 and II.F.1.2 change "12/31/84" to "Prior to Unit 2 startup in Cycle 6."
- (5) a. When emergency diesel generators are removed from service for up to 14 days for preventive maintenance under the provisions of Amendment No. 209, the licensee shall:
1. Require another offsite power source be available in addition to the requirements of Technical Specification 3.9.A.1.c that two offsite sources be available.
 2. Restrict work activities affecting the ability to cross-tie the associated Unit 1/2 emergency diesel generator to the 4-kV shutdown board for the emergency diesel generator that is out of service.
 3. Restrict work activities on the 500-kV switchyard cross-tie breakers supporting the affected unit.
 4. No high risk switchyard maintenance will be scheduled while the emergency diesel generator is out of service. If emergent conditions require performance of such high risk activities, such activities shall be approved by the Plant Manager and the Operations Manager. "High risk switchyard maintenance" is defined as an activity that if a single error or problem occurs, a full reactor scram, transient requiring a reduction in reactor power, and/or an unplanned engineered safety features actuation requiring a report to the NRC within 4 hours, could occur.
- b. These provisions apply during the time period from January 1, 1998 to February 1, 1999, or completion of preventive maintenance under the provisions of Amendment No. 209, whichever occurs first.

APPENDIX B
ADDITIONAL CONDITIONS

<u>Amend. Number</u>	<u>Additional Conditions</u>	<u>Implementation Date</u>
212	<p>The licensee is authorized to relocate certain requirements included in Appendix A and the former Appendix B to licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the appropriate documents, as described in the licensee's application dated September 6, 1996, as supplemented May 1, August 14, November 5 and 14, December 3, 4, 11, 22, 23, 29 and 30, 1997, January 23, March 12, April 16, 20 and 28, and May 7, 14, 19 and 27, and June 2, 5, 10 and 19, 1998, evaluated in the NRC staff's Safety Evaluation enclosed with this amendment.</p>	<p>This amendment is effective immediately and shall be implemented within 90 days of the date of this amendment.</p>