

September 12, 1994

Mr. Oliver D. Kingsley, Jr.
President, TVA Nuclear and
Chief Nuclear Officer
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

SUBJECT: CORRECTION TO AMENDMENT NOS. 158, 154, AND 129
BROWNS FERRY NUCLEAR PLANT UNITS 1, 2, AND 3
(TAC NOS. R00068, R00069, AND R00070)

Dear Mr. Kingsley:

This letter corrects a recently discovered administrative error in Amendment No. 158 to Facility Operating License No. DPR-33, Amendment No. 154 to Facility Operating License No. DPR-52, and Amendment No. 129 to Facility Operating License No. DPR-68 for the Browns Ferry Nuclear Plant (BFN) Units 1, 2, and 3, respectively. These amendments were issued by the Commission by letter dated November 18, 1988.

The enclosed corrected pages provide the proper wording for technical specification pages 3.9/4.9-9 and 3.9/4.9-10, which had been revised by Amendments 153, 149, and 124 to the respective BFN Unit 1, 2, and 3 licenses. These amendments were issued in a letter dated August 19, 1988.

We regret any inconvenience this error may have caused. Please call me at (301)504-1470 if you have any questions regarding this topic.

Sincerely,
Original signed by
Joseph F. Williams, Project Manager
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-259, 50-260, and 50-296

Enclosure: Corrected TS Pages 3.9/4.9-9 & 3.9/4.9-10
BFN Units 1, 2 & 3

cc w/enclosures: See next page

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BROWNS FERRY NUCLEAR PLANT

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**CORRECTION TO AMENDMENT NO. 158, 154, AND 129 - BROWNS FERRY NUCLEAR PLANT,
UNIT 1, 2, AND 3
DATED:**

DISTRIBUTION

Docket File

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BFN Rdg. File

S. Varga

0-14-E-4

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0-15-B-18

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T-5-C-3

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0-11-E-22

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0-2-G-5

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M. Lesser

RII

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B. Boger

RII

CORRECTION TO LICENSE AMENDMENT NO. 158

FACILITY OPERATING LICENSE NO. DPR-33

DOCKET NO. 50-259

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The corrections to the pages are identified by the marginal lines.

Remove

3.9/4.9-9
3.9/4.9-10

Insert

3.9/4.9-9
3.9/4.9-10

ENCLOSURE

3.9/4.9 AUXILIARY ELECTRICAL SYSTEM

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.9.B. Operation With Inoperable Equipment

3. When one of the units 1 and 2 diesel generator is INOPERABLE, continued REACTOR POWER OPERATION permissible during the succeeding 7 days, provided that 2 offsite power sources are available as specified in 3.9.A.1.c and all of the CS, RHR (LPCI and containment cooling) systems, and the remaining three units 1 and 2 diesel generators are OPERABLE. If this requirement cannot be met, an orderly shutdown shall be initiated and the reactor shall be in the COLD SHUTDOWN CONDITION within 24 hours.
4. When one units 1 and 2 4-kV shutdown board is INOPERABLE, continued REACTOR POWER OPERATION is permissible for a period of 5 days provided that 2 offsite power sources are available as specified in 3.9.A.1.c and the remaining 4-kV shutdown boards and associated diesel generators, CS, RHR (LPCI and containment cooling) systems, and all 480-V emergency power boards are OPERABLE. If this requirement cannot be met, an orderly shutdown shall be initiated and the reactor shall be in the COLD SHUTDOWN CONDITION within 24 hours.

4.9.B. Operation With Inoperable Equipment

3. When one of the units 1 and 2 diesel generators is found to be INOPERABLE, all of the remaining diesel generators shall be demonstrated to be OPERABLE within 24 hours, and power availability for the associated boards shall be verified within 1 hour and at least once per 8 hours thereafter.
4. When one 4-kV shutdown board is found to be INOPERABLE, all diesel generators associated with the remaining 4-kV shutdown boards shall be demonstrated to be OPERABLE within 24 hours, and power availability for the remaining 4-kV shutdown boards shall be verified within 1 hour and at least once per 8 hours thereafter.

3.9/4.9 AUXILIARY ELECTRICAL SYSTEM

LIMITING CONDITIONS FOR OPERATION

3.9.B. Operation With Inoperable Equipment

5. When one of the shutdown buses is INOPERABLE, REACTOR POWER OPERATION is permissible for a period of 7 days.
6. When one of the 480-V diesel auxiliary boards becomes INOPERABLE, REACTOR POWER OPERATION is permissible for a period of 5 days.
7. From and after the date that one of the three 250-V unit batteries and/or its associated battery board is found to be INOPERABLE for any reason, continued REACTOR POWER OPERATION is permissible during the succeeding 7 days. Except for routine surveillance testing, NRC shall be notified within 24 hours of the situation, the precautions to be taken during this period, and the plans to return the failed component to an OPERABLE state.

SURVEILLANCE REQUIREMENTS

4.9.B. Operation With Inoperable Equipment

5. When a shutdown bus is found to be INOPERABLE, all 1 and 2 diesel generators shall be proven OPERABLE within 24 hours.
6. When one units 1 and 2 diesel auxiliary board is found to be INOPERABLE, each unit 1 and 2 diesel generator shall be proven OPERABLE within 24 hours, and power availability for the remaining diesel auxiliary board shall be verified within 1 hour and at least once per 8 hours thereafter.

CORRECTION TO LICENSE AMENDMENT NO. 154

FACILITY OPERATING LICENSE NO. DPR-52

DOCKET NO. 50-260

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The corrections to the pages are identified by the marginal lines.

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Insert

3.9/4.9-9
3.9/4.9-10

3.9/4.9-9
3.9/4.9-10

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ENCLOSURE

3.9/4.9 AUXILIARY ELECTRICAL SYSTEM

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.9.B. Operation With Inoperable Equipment

4.9.B. Operation With Inoperable Equipment

3. When one of the units 1 and 2 diesel generator is INOPERABLE, continued REACTOR POWER OPERATION is permissible during the succeeding 7 days, provided that 2 offsite power sources are available as specified in 3.9.A.1.c and all of the CS, RHR (LPCI and containment cooling) systems, and the remaining three units 1 and 2 diesel generators are OPERABLE. If this requirement cannot be met, an orderly shutdown shall be initiated and the reactor shall be in the COLD SHUTDOWN CONDITION within 24 hours.
4. When one units 1 and 2 4-kV shutdown board is INOPERABLE, continued REACTOR POWER OPERATION is permissible for a period of 5 days provided that 2 offsite power sources are available as specified in 3.9.A.1.c and the remaining 4-kV shutdown boards and associated diesel generators, CS, RHR (LPCI and containment cooling) systems, and all 480-V emergency power boards are OPERABLE. If this requirement cannot be met, an orderly shutdown shall be initiated and the reactor shall be in the COLD SHUTDOWN CONDITION within 24 hours.

3. When one of the units 1 and 2 diesel generators is found to be INOPERABLE, all of the remaining diesel generators shall be demonstrated to be OPERABLE within 24 hours, and power availability for the associated boards shall be verified within 1 hour and at least once per 8 hours thereafter.
4. When one 4-kV shutdown board is found to be INOPERABLE, all diesel generators associated with the remaining 4-kV shutdown boards shall be demonstrated to be OPERABLE within 24 hours, and power availability for the remaining 4-kV shutdown boards shall be verified within 1 hours and at least once per 8 hours thereafter.

3.9/4.9 AUXILIARY ELECTRICAL SYSTEM

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.9.B. Operation With Inoperable Equipment

5. When one of the shutdown buses is INOPERABLE, REACTOR POWER OPERATION is permissible for a period of 7 days.
6. When one of the 480-V diesel auxiliary boards becomes INOPERABLE, REACTOR POWER OPERATION is permissible for a period of 5 days.
7. From and after the date that one of the three 250-V unit batteries and/or its associated battery board is found to be INOPERABLE for any reason, continued REACTOR POWER OPERATION is permissible during the succeeding 7 days. Except for routine surveillance testing, NRC shall be notified within 24 hours of the situation, the precautions to be taken during this period, and the plans to return the failed component to an OPERABLE state.

4.9.B. Operation With Inoperable Equipment

5. When a shutdown bus is found to be INOPERABLE, all 1 and 2 diesel generators shall be proven OPERABLE within 24 hours.
6. When one units 1 and 2 diesel auxiliary board is found to be INOPERABLE, each unit 1 and 2 diesel generator shall be proven OPERABLE within 24 hours, and power availability for the remaining diesel auxiliary board shall be verified within 1 hour and at least once per 8 hours thereafter.

CORRECTION TO LICENSE AMENDMENT NO. 129

FACILITY OPERATING LICENSE NO. DPR-68

DOCKET NO. 50-296

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The corrections to the pages are identified by the marginal lines.

Remove

Insert

3.9/4.9-9
3.9/4.9-10

3.9/4.9-9
3.9/4.9-10

ENCLOSURE

3.9/4.9 AUXILIARY ELECTRICAL SYSTEM

LIMITING CONDITIONS FOR OPERATION

3.9.B. Operation With Inoperable Equipment

3. From and after the date that the 4-kV bus tie board becomes inoperable, REACTOR POWER OPERATION is permissible indefinitely provided one of the required offsite power sources is not supplied from the 161-kV system through the bus tie board.

4. When one unit 3 4-kV shutdown board is INOPERABLE, continued REACTOR POWER OPERATION is permissible for a period of 5 days provided that 2 offsite power sources are available as specified in 3.9.A.1.c and the remaining unit 3 4-kV shutdown boards and associated diesel generators, CS, RHR (LPCI and containment cooling) systems, and all unit 3 480-V emergency power boards are OPERABLE. If this requirement cannot be met, an orderly shutdown shall be initiated and the reactor shall be shut down and in the COLD SHUTDOWN CONDITION within 24 hours.

SURVEILLANCE REQUIREMENTS

4.9.B. Operation With Inoperable Equipment

3. When a required offsite power source is unavailable because the 4-kV bus tie board or a start bus is INOPERABLE, all unit 3 diesel generators shall be demonstrated OPERABLE within 24 hours, and power availability for the associated boards shall be verified within 1 hour and at least once per 8 hours thereafter. The remaining offsite source and associated buses shall be checked to be energized daily.

4. When one unit 3 4-kV shutdown board is found to be INOPERABLE, all diesel generators associated with the remaining 4-kV shutdown boards shall be demonstrated to be OPERABLE within 24 hours, and power availability for the remaining 4-kV shutdown boards shall be verified within 1 hour and at least once per 8 hours thereafter.

LIMITING CONDITIONS FOR OPERATION

3.9.B. Operation With Inoperable Equipment

5. From and after the date that one of the 480-V, diesel auxiliary boards becomes INOPERABLE, REACTOR POWER OPERATION is permissible for a period of 5 days.

6. From and after the date that the 250-V shutdown board 3EB battery or one of the three 250-V unit batteries and/or its associated battery board is found to be INOPERABLE for any reason, continued REACTOR POWER OPERATION is permissible during the succeeding seven days. Except for routine surveillance testing, the NRC shall be notified within 24 hours of the situation, the precautions to be taken during this period, and the plans to return the failed component to an OPERABLE state.

7. When one division of the logic system is INOPERABLE, continued REACTOR POWER OPERATION is permissible under this condition for seven days, provided the CSCS requirements listed in Specification 3.9.B.2 are satisfied. The NRC shall be notified within 24 hours of the situation, the precautions to be taken during this period, and the plans to return the failed component to an OPERABLE state.

SURVEILLANCE REQUIREMENTS

4.9.B. Operation With Inoperable Equipment

5. When one 480-V diesel auxiliary board is found INOPERABLE, each unit 3 diesel shall be verified OPERABLE within 24 hours, and power availability for the remaining diesel auxiliary board shall be verified within 1 hour and at least once per 8 hours thereafter.