



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

May 21, 1993

DESIGNATED ORIGINAL

Certified By

King W. Row
PDR

Docket Nos. 50-259, 50-260
and 50-296

Dr. Mark O. Medford, Vice President
Technical Support
Tennessee Valley Authority
3B Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Dr. Medford:

SUBJECT: ISSUANCE OF TECHNICAL SPECIFICATION AMENDMENTS REGARDING SECTION 5,
"DESIGN FEATURES" - BROWNS FERRY NUCLEAR PLANT (TS-309)
(TAC NOS. M86369, M86370, AND M86371)

The Commission has issued the enclosed Amendment Nos. 198, 215, and 171 to Facility Operating Licenses Nos. DPR-33, DPR-52, and DPR-68 for the Browns Ferry Nuclear Plant (BFN), Units 1, 2, and 3, respectively. These amendments are in response to an application by the Tennessee Valley Authority (TVA) dated May 17, 1993, that requested emergency approval of proposed changes to the BFN Technical Specifications (TS) pursuant to the provisions 10 CFR 50.91(a)(5). The enclosed amendments revise Section 5.2 of the BFN TS according to the guidelines of Generic Letter 90-02, "Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications."

A copy of the NRC's Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Gus C. Lainas, Assistant Director
for Region II Reactors
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.198 to
License No. DPR-33
2. Amendment No.215 to
License No. DPR-52
3. Amendment No.171 to
License No. DPR-68
4. Safety Evaluation

cc w/enclosures:
See next page

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and 50-296

May 21, 1993

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Technical Support
Tennessee Valley Authority
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SUBJECT: ISSUANCE OF TECHNICAL SPECIFICATION AMENDMENTS REGARDING SECTION 5,
"DESIGN FEATURES" - BROWNS FERRY NUCLEAR PLANT (TS-309)
(TAC NOS. M86369, M86370, AND M86371)

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Sincerely,
Original signed by
Gus C. Lainas, Assistant Director
for Region II Reactors
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.198 to License No. DPR-33
2. Amendment No.215 to License No. DPR-52
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4. Safety Evaluation

cc w/enclosures:
See next page

PDII-4/LA	PDII-4/PM	PDII-4/PM	OGC	SRXB	PDII-4/D	ADRII
MSanders	TRoss:as	JWilliams	CPhillips	RJones	FHebbon	GLainas
5/21/93	5/21/93	5/21/93	5/21/93	5/21/93	5/21/93	5/21/93

DOCUMENT NAME: TS309S1.AMD

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-259

BROWNS FERRY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.198
License No. DPR-33

The Nuclear Regulatory Commission (the Commission) has found that:

- A. The application for amendment by Tennessee Valley Authority (the licensee) dated May 17, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
- C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
- D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-33 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.198 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Gus C. Lainas, Assistant Director
for Region II Reactors
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 21 , 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 198

FACILITY OPERATING LICENSE NO. DPR-33

DOCKET NO. 50-259

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. A spillover page* is provided to maintain document completeness.

REMOVE

5.0-1
5.0-2

INSERT

5.0-1
5.0-2*

5.0 MAJOR DESIGN FEATURES

5.1 SITE FEATURES

Browns Ferry unit 1 is located at Browns Ferry Nuclear Plant site on property owned by the United States and in custody of the TVA. The site shall consist of approximately 840 acres on the north shore of Wheeler Lake at Tennessee River Mile 294 in Limestone County, Alabama. The minimum distance from the outside of the secondary containment building to the boundary of the exclusion area as defined in 10 CFR 100.3 shall be 4,000 feet.

5.2 REACTOR

- A. The reactor shall contain 764 fuel assemblies. Each assembly shall consist of a matrix of zirconium alloy fuel rods with an initial composition of natural or slightly enriched uranium dioxide (UO_2) as fuel material and water rods. Limited substitutions of zirconium alloy or stainless steel filler rods for fuel rods, in accordance with approved applications of fuel rod configurations, may be used. Fuel assemblies shall be limited to those fuel designs that have been analyzed with NRC staff approved codes and methods and have been shown by tests or analyses to comply with all safety design bases. A limited number of lead test assemblies that have not completed representative testing may be placed in nonlimiting core regions.
- B. The reactor core shall contain 185 cruciform shaped control rod assemblies. The control material shall be boron carbide and/or hafnium metal as approved by the NRC.

5.3 REACTOR VESSEL

The reactor vessel shall be as described in Table 4.2-2 of the FSAR. The applicable design codes shall be as described in Table 4.2-1 of the FSAR.

5.4 CONTAINMENT

- A. The principal design parameters for the primary containment shall be as given in Table 5.2-1 of the FSAR. The applicable design codes shall be as described in Section 5.2 of the FSAR.
- B. The secondary containment shall be as described in Section 5.3 of the FSAR.

5.0 MAJOR DESIGN FEATURES (Continued)

5.4 CONTAINMENT (Continued)

- C. Penetrations to the primary containment and piping passing through such penetrations shall be designed in accordance with the standards set forth in Section 5.2.3.4 of the FSAR.

5.5 FUEL STORAGE

- A. The arrangement of fuel in the new-fuel storage facility shall be such that k_{eff} , for dry conditions, is less than 0.90 and flooded is less than 0.95 (Section 10.2 of FSAR).
- B. The k_{eff} of the spent fuel storage pool shall be less than or equal to 0.95.
- C. Loads greater than 1000 pounds shall not be carried over spent fuel assemblies stored in the spent fuel pool.

5.6 SEISMIC DESIGN

The station class I structures and systems have been designed to withstand a design basis earthquake with ground acceleration of 0.2g. The operational basis earthquake used in the plant design assumed a ground acceleration of 0.1g (see Section 2.5 of the FSAR).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-260

BROWNS FERRY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.215
License No. DPR-52

The Nuclear Regulatory Commission (the Commission) has found that:

- A. The application for amendment by Tennessee Valley Authority (the licensee) dated May 17, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
- C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
- D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.215 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Gus C. Lainas, Assistant Director
for Region II Reactors
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 21, 1993

ATTACHMENT TO LICENSE AMENDMENT NO.215

FACILITY OPERATING LICENSE NO. DPR-52

DOCKET NO. 50-260

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. A spillover* page is provided to maintain document completeness.

REMOVE

5.0-1
5.0-2

INSERT

5.0-1
5.0-2*

5.0 MAJOR DESIGN FEATURES

5.1 SITE FEATURES

Browns Ferry unit 2 is located at Browns Ferry Nuclear Plant site on property owned by the United States and in custody of the TVA. The site shall consist of approximately 840 acres on the north shore of Wheeler Lake at Tennessee River Mile 294 in Limestone County, Alabama. The minimum distance from the outside of the secondary containment building to the boundary of the exclusion area as defined in 10 CFR 100.3 shall be 4,000 feet.

5.2 REACTOR

- A. The reactor shall contain 764 fuel assemblies. Each assembly shall consist of a matrix of zirconium alloy fuel rods with an initial composition of natural or slightly enriched uranium dioxide (UO₂) as fuel material and water rods. Limited substitutions of zirconium alloy or stainless steel filler rods for fuel rods, in accordance with approved applications of fuel rod configurations, may be used. Fuel assemblies shall be limited to those fuel designs that have been analyzed with NRC staff approved codes and methods and have been shown by tests or analyses to comply with all safety design bases. A limited number of lead test assemblies that have not completed representative testing may be placed in nonlimiting core regions.
- B. The reactor core shall contain 185 cruciform shaped control rod assemblies. The control material shall be boron carbide and/or hafnium metal as approved by the NRC.

5.3 REACTOR VESSEL

The reactor vessel shall be as described in Table 4.2-2 of the FSAR. The applicable design codes shall be as described in Table 4.2-1 of the FSAR.

5.4 CONTAINMENT

- A. The principal design parameters for the primary containment shall be as given in Table 5.2-1 of the FSAR. The applicable design codes shall be as described in Section 5.2 of the FSAR.
- B. The secondary containment shall be as described in Section 5.3 of the FSAR.

5.0 MAJOR DESIGN FEATURES (Continued)

5.4 CONTAINMENT (Continued)

- C. Penetrations to the primary containment and piping passing through such penetrations shall be designed in accordance with the standards set forth in Section 5.2.3.4 of the FSAR.

5.5 FUEL STORAGE

- A. The arrangement of fuel in the new-fuel storage facility shall be such that k_{eff} , for dry conditions, is less than 0.90 and flooded is less than 0.95 (Section 10.2 of FSAR).
- B. The k_{eff} of the spent fuel storage pool shall be less than or equal to 0.95.
- C. Loads greater than 1000 pounds shall not be carried over spent fuel assemblies stored in the spent fuel pool.

5.6 SEISMIC DESIGN

The station class I structures and systems have been designed to withstand a design basis earthquake with ground acceleration of 0.2g. The operational basis earthquake used in the plant design assumed a ground acceleration of 0.1g (see Section 2.5 of the FSAR).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-296

BROWNS FERRY NUCLEAR PLANT, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 171
License No. DPR-68

The Nuclear Regulatory Commission (the Commission) has found that:

- A. The application for amendment by Tennessee Valley Authority (the licensee) dated May 17, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
- C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
- D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

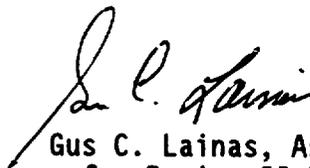
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-68 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 171, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Gus C. Lainas, Assistant Director
for Region II Reactors
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 21, 1993

ATTACHMENT TO LICENSE AMENDMENT NO.171

FACILITY OPERATING LICENSE NO. DPR-68

DOCKET NO. 50-296

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. A spillover* page is provided to maintain document completeness.

REMOVE

5.0-1
5.0-2

INSERT

5.0-1
5.0-2*

5.0 MAJOR DESIGN FEATURES

5.1 SITE FEATURES

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5.2 REACTOR

- A. The reactor shall contain 764 fuel assemblies. Each assembly shall consist of a matrix of zirconium alloy fuel rods with an initial composition of natural or slightly enriched uranium dioxide (UO₂) as fuel material and water rods. Limited substitutions of zirconium alloy or stainless steel filler rods for fuel rods, in accordance with approved applications of fuel rod configurations, may be used. Fuel assemblies shall be limited to those fuel designs that have been analyzed with NRC staff approved codes and methods and have been shown by tests or analyses to comply with all safety design bases. A limited number of lead test assemblies that have not completed representative testing may be placed in nonlimiting core regions.
- B. The reactor core shall contain 185 cruciform shaped control rod assemblies. The control material shall be boron carbide and/or hafnium metal as approved by the NRC.

5.3 REACTOR VESSEL

The reactor vessel shall be as described in Table 4.2-2 of the FSAR. The applicable design codes shall be as described in Table 4.2-1 of the FSAR.

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- A. The principal design parameters for the primary containment shall be as given in Table 5.2-1 of the FSAR. The applicable design codes shall be as described in Section 5.2 of the FSAR.
- B. The secondary containment shall be as described in Section 5.3 of the FSAR.

5.0 MAJOR DESIGN FEATURES (Continued)

5.4 CONTAINMENT (Continued)

- C. Penetrations to the primary containment and piping passing through such penetrations shall be designed in accordance with the standards set forth in Section 5.2.3.4 of the FSAR.

5.5 FUEL STORAGE

- A. The arrangement of the fuel in the new-fuel storage facility shall be such that k_{eff} , for dry conditions, is less than 0.90 and flooded is less than 0.95 (Section 10.2 of FSAR).
- B. The k_{eff} of the spent fuel storage pool shall be less than or equal to 0.95.
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The station class I structures and systems have been designed to withstand a design basis earthquake with ground acceleration of 0.2g. The operational basis earthquake used in the plant design assumed a ground acceleration of 0.1g (see Section 2.5 of the FSAR).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

ENCLOSURE 4

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 198 TO FACILITY OPERATING LICENSE NO. DPR-33

AMENDMENT NO. 215 TO FACILITY OPERATING LICENSE NO. DPR-52

AMENDMENT NO. 171 TO FACILITY OPERATING LICENSE NO. DPR-68

TENNESSEE VALLEY AUTHORITY

BROWNS FERRY NUCLEAR PLANT, UNITS 1, 2, AND 3

DOCKET NOS. 50-259, 50-260 AND 50-296

1.0 INTRODUCTION

By letter dated August 20, 1992, the Tennessee Valley Authority (TVA, the licensee) submitted a request to revise the Technical Specifications (TS) of the Browns Ferry Nuclear Plant (BFN), Units 1, 2, and 3, in accordance with the guidelines of Generic Letter (GL) 88-16, "Removal of Cycle-Specific Parameter Limits from Technical Specifications." Included in this letter, were proposed changes to Section 5, "Major Design Features," of the BFN TS. These changes were subsequently determined to be outside the scope of GL 88-16. Consequently, TVA supplanted the proposed TS changes specifically related to Section 5 by a letter dated May 17, 1993. This letter utilized the guidance provided by GL 90-02, "Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications," to justify proposed TS changes to Section 5.

TVA's letter of May 17, 1993, requested emergency NRC approval of the proposed Section 5 TS changes pursuant to the provisions of 10 CFR 50.91(a)(5). BFN, Unit 2 is scheduled to restart from the current Cycle 6 refueling outage on May 21, 1993. TVA will be unable to restart Unit 2 without NRC approval of the proposed Section 5 TS changes.

2.0 EVALUATION

Section 5.2, "Reactor," of the existing BFN TS provide an explicit description of the number and type of fuel assemblies and control rods utilized in the reactor core (for the current fuel cycle). In the past, each subsequent fuel cycle for each BFN unit has necessitated a TS amendment to update Section 5.2 regarding the specific description of fuel assembly designs used in the refueled reactor core. By letter dated August 20, 1992, TVA submitted a TS amendment application in accordance with GL 88-16. This amendment proposed to relocate all cycle-specific parameter limits from the TS to a Core Operating Limits Report (COLR). The purpose of this amendment was to allow TVA the flexibility of refueling any BFN unit without requiring the submittal of a cycle-specific TS amendment application for each new core reload. By letter dated May 20, 1993, the staff transmitted license amendments numbered 197, 214, and 170 for BFN, Units 1, 2, and 3, respectively, that approved TVA's proposed TS changes except for Section 5.2.

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The staff considered TVA's proposed changes to the "Major Design Features" section of the TS were inappropriately included with the TS changes associated with GL 88-16.

In response to the staff's determination that the TS changes to Section 5 could not be approved as part of TVA's original application per GL 88-16, TVA decided to reapply using the guidance of GL 90-02. In order to support Cycle 7 restart of BFN, Unit 2, and allow for future fuel cycles without the necessity of submitting a license amendment each time, TVA proposed TS changes to Sections 5.2.A and 5.2.B consistent with the language used by the new Standard Technical Specifications (STS) for General Electric Plants, BWR/4, issued by the NRC as NUREG-1433 dated September 1992.

Supplement 1 of GL 90-02, provided guidance for a line-item change of TS Section 5 for "Fuel Assemblies." This guidance recommended that licensees incorporate TS modeled after the STS to permit the substitution of fuel rods as long as the substitution was justified by cycle-specific reload analyses using NRC-approved methodology. The staff considers an NRC-approved methodology to be any methodology that the NRC staff has explicitly approved in a written safety evaluation. The NRC-approved methodology must be used only for the purpose and the scope of application specified in the reviewed document as approved or modified in the NRC approval documentation.

The staff confirmed that the TS changes proposed by TVA for Sections 5.2.A (fuel assemblies) and 5.2.B (control rods) are consistent with the STS. Furthermore, these changes are in accordance with the guidelines of GL 90-02, Supplement 1. The proposed TS will ensure that future fuel assemblies and control rod assemblies utilized at BFN are designed and analyzed in accordance with NRC-approved methodologies. Consequently, the NRC staff concludes the TS changes proposed by TVA in their application of May 17, 1993 are acceptable.

3.0 EMERGENCY CIRCUMSTANCES

TVA requested, in its application of May 17, 1993, that proposed changes to Section 5 of the BFN TS be approved on an emergency basis. TVA claimed in this letter that an emergency situation existed because current TS prevent the resumption of Unit 2 power operation. Furthermore, TVA explained that the cause of this emergency situation was the NRC's determination that the changes to Section 5 of the BFN TS proposed by the August 20, 1992 letter, could not be approved within the scope of GL 88-16. As such, TVA was impelled to supplement its original TS amendment application and to request emergency NRC approval due to the limited time available prior to Unit 2 restart.

TVA's original TS amendment application of August 20, 1992, was submitted in a timely manner. However, the staff's determination that the justification for proposed TS changes to Section 5 were not suitable, was not identified until just recently. Upon notification that its TS amendment application was deficient, TVA revised the proposed changes to Section 5 of the BFN TS and provided an appropriate justification as quickly as was practicable.

After reviewing this situation pursuant to the provisions of 10 CFR 50.91(a)(5), the staff concluded an emergency situation does exist that would prevent startup of BFN, Unit 2 and that TVA did respond in a timely manner.

4.0 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The Commission's regulations in 10 CFR 50.92 state that the Commission may make a final determination that the proposed license amendment involves no significant hazards consideration if operation of the facility, in accordance with the amendment, would not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

Core reload design analyses which address the applicable safety criteria are performed prior to each cycle of operation. These safety criteria require that the radiological consequences of any design basis accident not exceed the guidelines set forth in 10 CFR 100. The analytical methodologies used to satisfy these guidelines are approved by NRC. The proposed amendments require that these analytical methodologies continue to be approved by NRC. The anticipated operational transients and design basis accidents described in the FSAR are initiated by operator errors or equipment malfunctions other than by the fuel or control rod assemblies. Thus, the adoption of this amendment will not increase the probability or consequences of any accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any accident previously evaluated.

Operation of the facility in accordance with the proposed amendment would not create the possibility of a new or different kind of accident from any accident previously evaluated because the proposed amendments will not result in changes to any safety-related equipment or safety functions. The proposed amendments will not change any equipment, systems, or setpoints designed to prevent or mitigate accidents.

3. Involve a significant reduction in a margin of safety.

Operation of the facility, in accordance with the proposed amendment, would not involve a significant reduction in the margin of safety because an adequate margin of safety is ensured by performing analyses using NRC-approved methodologies to verify compliance with the conditions and acceptance criteria assumed in the FSAR. These analyses are performed to establish specific numerical values for core operating limits/restrictions to insure that adequate margin to safety is maintained should an event occur. These limits/restrictions will be contained in the TS and the Core Operating Limits Reports. The TS will continue to require compliance and operation within the bounds of these limits/restrictions. No changes will be made to actions required by the TS in the event of noncompliance. Development of limits/restrictions and core reload design for future cycles will conform to NRC-approved methods.

Accordingly, based on the evaluation above pursuant to 10 CFR 50.92(c), the NRC staff concludes that the proposed amendment does not involve significant hazards considerations.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Alabama State official was notified of the proposed issuance of the amendments. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has made a final no significant hazards consideration determination with respect to the amendments. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Ross

Date: May 21, 1993