

July 27, 1984

Docket Nos. 50-259/260

Mr. Hugh G. Parris  
Manager of Power  
Tennessee Valley Authority  
500A Chestnut Street, Tower II  
Chattanooga, Tennessee 37401

Dear Mr. Parris:

The Commission has issued the enclosed Amendment Nos. 104 and 98 to Facility Operating License Nos. DPR-33, and DPR-52 for the Browns Ferry Nuclear Plant, Units 1 and 2. These amendments are in response to your application dated August 24, 1983 (TVA BFNP TS 176 Supp 10).

The amendments change the Technical Specifications to delete surveillance requirements for dampers FCO-150A and FCO-150C which have been removed.

A copy of the Safety Evaluation is also enclosed.

Sincerely,

**Original signed by  
Richard J. Clark**

Richard J. Clark, Project Manager  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. 104 to License No. DPR-33
2. Amendment No. 98 to License No. DPR-52
3. Safety Evaluation

cc w/enclosures:  
See next page

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Mr. Hugh G. Parris  
Tennessee Valley Authority  
Browns Ferry Nuclear Plant, Units 1, 2 and 3

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-259

BROWNS FERRY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 104  
License No. DPR-33

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated August 24, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-33 is hereby amended to read as follows:

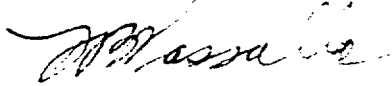
(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 104, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 27, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 104

FACILITY-OPERATING LICENSE NO. DPR-33

DOCKET NO. 50-259

Revise Appendix A as follows:

Remove page 245 and replace with revised page 245

Remove

245

Insert

245

The marginal lines on this page denote the area being changed.

3.7 CONTAINMENT SYSTEMS

- c. System flow rate shall be shown to be within  $\pm 10\%$  design flow when tested in accordance with ANSI N510-1975.
3. From and after the date that one of the control room emergency pressurization systems is made or found to be inoperable for any reason, reactor operation or refueling operations is permissible only during the succeeding 7 days unless such circuit is sooner made operable.
4. If these conditions cannot be met, reactor shutdown shall be initiated and all reactors shall be in cold shutdown within 24 hours for reactor operations and refueling operations shall be terminated within 2 hours.

4.7 CONTAINMENT SYSTEMS

- c. Halogenated hydrocarbon testing shall be performed after each complete or partial replacement of the charcoal adsorber bank or after any structural maintenance on the system housing.
- d. Each circuit shall be operated at least 10 hours every month.
3. At least once per operating cycle not to exceed 18 months, automatic initiation of the control room emergency pressurization system shall be demonstrated.
4. During the simulated automatic actuation test of this system (see Table 4.2.G), it shall be verified that the following dampers operate as indicated:

Close: FCO-150 B,  
D, E, and F  
Open: FCO-151  
FCO-152



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-260

BROWNS FERRY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 98  
License No. DPR-52

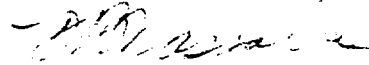
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated August 24, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 98, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: July 27, 1984



ATTACHMENT TO LICENSE AMENDMENT NO. 98

FACILITY OPERATING LICENSE NO. DPR-52

DOCKET NO. 50-260

Revise Appendix A as follows:

Remove page 245 and replace with revised page.

Remove

Insert

245

245

The marginal lines on this page denote the area being changed.

4.7 CONTAINMENT SYSTEMS

- c. System flow rate shall be shown to be within  $\pm 10\%$  design flow when tested in accordance with ANSI NS10-1975.
3. From and after the date that one of the control room emergency pressurization systems is made or found to be inoperable for any reason, reactor operation or refueling operations is permissible only during the succeeding 7 days unless such circuit is sooner made operable.
4. If these conditions cannot be met, reactor shutdown shall be initiated and all reactors shall be in cold shutdown within 24 hours for reactor operations and refueling operations shall be terminated within 2 hours.

4.7 CONTAINMENT SYSTEMS

- c. Halogenated hydrocarbon testing shall be performed after each complete or partial replacement of the charcoal adsorber bank or after any structural maintenance on the system housing.
- d. Each circuit shall be operated at least 10 hours every month.
3. At least once per operating cycle not to exceed 18 months, automatic initiation of the control room emergency pressurization system shall be demonstrated.
4. During the simulated automatic actuation test of this system (see Table 4.2.G), it shall be verified that the following dampers operate as indicated:
- Close: FCO-150 B,  
D, E, and F  
Open: FCO-151  
FCO-152



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 104 TO FACILITY OPERATING LICENSE NO. DPR-33  
AMENDMENT NO. 98 TO FACILITY OPERATING LICENSE NO. DPR-52

TENNESSEE VALLEY AUTHORITY

BROWNS FERRY NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-259 AND 50-260

1.0 Introduction

By letter dated August 24, 1983 (TS 176 Supplement 10) the Tennessee Valley Authority (the licensee or TVA) requested amendments to Facility Operating License Nos. DPR-33 and DPR-52 for the Browns Ferry Nuclear Plant, Units 1 and 2. The proposed amendments would change the Appendix A Technical Specifications to delete surveillance requirements for Control Room Emergency Pressurization System dampers FCO-150A and FCO-150C. These isolation dampers were removed, as part of a design change, when the Units 1, 2 and 3 Control Room Emergency Pressurization Systems were combined.

2.0 Evaluation

The duct openings associated with dampers FCO-150A and FCO-150C have been permanently closed so that the isolation function previously provided by these dampers is no longer required. The requested change is therefore, acceptable.

3.0 Environmental Considerations

The amendments involve a change in the installation or use of a facility component located within the restricted areas defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupation radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

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#### 4.0 Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. Long

Dated: July 27, 1984