



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23T85
ATLANTA, GEORGIA 30303-8931**

December 27, 2001

Southern Nuclear Operating Company, Inc.
ATTN: Mr. D. N. Morey
Vice President
P. O. Box 1295
Birmingham, AL 35201

**SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT - NRC INSPECTION REPORT NO.
50-348/02-03, 50-364/02-03**

Dear Mr. Morey:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a safety system design and performance capability inspection at your Farley Nuclear Plant during March 2002. A team of four inspectors will perform the inspection. The inspection team will be led by Mr. F. Jape, a senior reactor inspector from the NRC Region II Office. The inspection will be conducted in accordance with baseline Inspection Procedure 71111.21, Safety System Design and Performance Capability.

The inspection objective will be to evaluate performance of the auxiliary feedwater system including the interface with the assured water supply provided by the service water system. The team will also review scenarios leading to partial or total loss of the auxiliary feedwater system.

During a telephone conversation on December 18, 2001, Mr. F. Jape of my staff, and Mr. Ajluni of your staff, confirmed arrangements for an information gathering site visit and the two-week onsite inspection. The schedule is as follows:

- Information gathering visit: February 4-8, 2002
- Onsite inspection: March 4-8, 2002, and March 18-22, 2002

The purpose of the information gathering visit is to obtain information and documentation outlined in the enclosure needed to support the inspection. Mr. Walter Rogers, a Region II Senior Reactor Analyst, may accompany Mr. F. Jape during a portion of the information gathering visit to review PRA data and identify risk significant components which will be examined during the inspection. Please contact Mr. F. Jape prior to preparing copies of the materials listed in the enclosure. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space; arrangements for reactor site access; and the availability of

knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact me at (404) 562-4605, or Mr. F. Jape at (404) 562-4541.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's Document system (ADAMS). ADAMS is accessible from the NRC web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Charles R. Ogle, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos.: 50-348, 50-364
License Nos.: NPF-2, NPF-8

Enclosure: Information Request for the Safety System Design
and Performance Capability Inspection

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(cc w/encl cont'd - See page 3)

SNC

3

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PUBLIC DOCUMENT (circle one): YES NO

OFFICE	RII:DRS	RIIDRP:					
SIGNATURE	JAPE	RAPP FOR:					
NAME	JAPE	CAHILL					
DATE	12/27/2001	12/27/2001	January 7, 2002				
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY

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**INFORMATION REQUEST FOR THE SAFETY SYSTEM DESIGN AND
PERFORMANCE CAPABILITY INSPECTION**

AUXILIARY FEEDWATER SYSTEM (AUX FW SYSTEM)

Note: Electronic media is preferred if readily available (i.e., on computer disc).

1. Site specific administrative procedures related to standard operation, abnormal operation, and emergency operation of the AUX FW System.
2. Design criteria (i.e., design basis documents) for the AUX FW System.
3. Technical Specification requirements and a list of associated surveillance tests and calibration procedures for the AUX FW System.
4. P&IDs that include suction sources for AUX FW System, and a listing of instrumentation and mechanical drawings. Electrical drawings consisting of key one line diagrams of power supply for the motor driven AUX Feedwater pumps, and one line diagrams of power from the switchyard to the feeder buses.
5. A list of engineering calculations (electrical, instrumentation and controls, mechanical and nuclear) applicable to the AUX FW System.
6. Strategy for handling loss of AUX FW events.
7. A list of plant modifications to the AUX FW System implemented since 1998.
8. List of current open temporary modifications and operator work arounds involving operation of the AUX FW System.
9. List of corrective actions initiated since 1998 affecting the AUX FW System.
10. Summary of corrective maintenance activities, including the maintenance rule event log, performed on the AUX FW System in the past 12 months.
11. Self-assessments performed on AUX FW System in the last 24 months.
12. System description and operator training modules for the AUX FW System.
13. List of operating experience program evaluations of industry, vendor, or NRC generic issues related to the AUX FW System for the past 2 years.
14. List of instrument setpoint changes affecting AUX FW System initiated since 1998. Include the number and title, date, brief description, and corresponding calculation number.