

April 29, 1992

Docket No. 50-389

Mr. J. H. Goldberg
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

DISTRIBUTION

Docket File DMiller
NRC & Local PDRs ACRS (10)
PDII-2 RF OPA
SVarga OC/LFMB
GLainas MSinkule, RII
JNorris
OGC
DHagan

Dear Mr. Goldberg:

SUBJECT: ST. LUCIE UNIT 2 - REQUEST FOR AMENDMENT RE: REDUCTION
IN SAFETY INJECTION TANK PRESSURE (TAC NO. 83245)

The Commission has requested the Office of the Federal Register to publish the enclosed "Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for Hearing." This notice relates to your application for amendment dated April 21, 1992, which would reduce the safety injection tank minimum pressure from 570 psi to 500 psi.

Sincerely,

ORIGINAL SIGNED BY HERBERT N. BERKOW
FOR

Jan A. Norris, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
Notice

cc w/enclosure:
See next page

OFC	:LA:PDII-2	:PM:PDII-2	:D:PDII-2	:	:	:
NAME	:DMiller	:JNorris	:kj:HBerkow	:	:	:
DATE	: 4/29/92	: 4/29/92	: 4/29/92	:	:	:

CP-1

Document Name - INDSL2.LTR

NRC FILE CENTER COPY

9205130131 920429
PDR ADOCK 05000389
P PDR

DF01/1

Mr. J. H. Goldberg
Florida Power and Light Company

St. Lucie Plant

cc:
Jack Shreve, Public Counsel
Office of the Public Counsel
c/o The Florida Legislature
111 West Madison Avenue, Room 812
Tallahassee, Florida 32399-1400

Mr. Jacob Daniel Nash
Office of Radiation Control
Department of Health and
Rehabilitative Services
1317 Winewood Blvd.
Tallahassee, Florida 32399-0700

Senior Resident Inspector
St. Lucie Plant
U.S. Nuclear Regulatory Commission
7585 S. Hwy A1A
Jensen Beach, Florida 33457

Regional Administrator, RII
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, Georgia 30323

Mr. Robert G. Nave, Director
Emergency Management
Department of Community Affairs
2740 Centerview Drive
Tallahassee, Florida 32399-2100

Mr. R. E. Grazio
Director, Nuclear Licensing
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

Harold F. Reis, Esq.
Newman & Holtzinger
1615 L Street, N.W.
Washington, DC 20036

John T. Butler, Esq.
Steel, Hector and Davis
4000 Southeast Financial Center
Miami, Florida 33131-2398

Administrator
Department of Environmental Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 34982

Mr. James V. Chisolm, County
Administrator
St. Lucie County
2300 Virginia Avenue
Fort Pierce, Florida 34982

Mr. Charles B. Brinkman, Manager
Washington Nuclear Operations
ABB Combustion Engineering, Nuclear Power
12300 Twinbrook Parkway, Suite 330
Rockville, Maryland 20852

UNITED STATES NUCLEAR REGULATORY COMMISSIONFLORIDA POWER AND LIGHT COMPANYDOCKET NO. 50-389NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-16 issued to Florida Power and Light Company for operation of St. Lucie Unit 2 located in St. Lucie County, Florida.

The proposed amendment would reduce the safety injection tank (SIT) minimum pressure from 570 psig to 500 psig. The proposed change provides the benefit of a greater differential pressure margin between the SIT operating pressure and the SIT relief valve pressure setpoint (669 psig). Reducing the SIT limiting condition for operation (LCO) to 500 psig means that the minimum SIT pressure would be approximately 75 percent of the relief valve pressure setpoint, which represents a 10 percent increase in margin when compared to current conditions. This additional margin lessens the potential for SIT relief valve leakage that has impacted plant availability in the past.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in

9205130138 920429
PDR ADOCK 05000389
P PDR

accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

- (1) Operation of the facility in accordance with the proposed amendment would not involve a significant increase in the probability or consequences of an accident previously evaluated.

Reducing the safety injection tank (SIT) minimum pressure does not involve a significant increase in the probability of a loss-of-coolant accident (LOCA), since the SITs are passive systems and have no effect on the reactor coolant system (RCS) until after the depressurization of the RCS due to a LOCA.

Reducing the SIT minimum pressure to 500 psig for St. Lucie Unit 2 increases the small break loss-of-coolant [accident] (SBLOCA) limiting break size from 0.0375 ft² to 0.0450 ft² and increases peak cladding temperature from 1771°F to 1905°F for the new limiting break size. The calculated peak cladding temperature (1905°F) for the SBLOCA analysis with reduced SIT minimum pressure remains less than the current large break loss-of-coolant [accident] (LBLOCA) analysis peak cladding temperature of 2107°F. This LBLOCA analysis was performed assuming [an] SIT pressure of 200 psig, a condition which conservatively covers the proposed minimum SIT pressure of 500 psig. Therefore, although the consequences of a SBLOCA are increased slightly, the LBLOCA calculation remains the limiting analysis of record for emergency core cooling system (ECCS) performance evaluation.

The LBLOCA analysis of record is for cycle 3 which has been shown to bound later cycles. This LBLOCA analysis also shows acceptable conformance to 10 CFR 50.46, [a]cceptance [c]riteria for ECCS performance for light water nuclear power reactors, in support of

the proposed minimum SIT pressure of 500 psig. This ECCS performance evaluation for St. Lucie Unit 2 was performed consistent with NRC approved methodology and 10 CFR 50 Appendix K criteria. The LBLOCA analysis was performed assuming [an] SIT pressure of 200 psig, therefore, there is no increase in the consequences of [an] LBLOCA due to reducing the minimum SIT pressure to 500 psig.

Non-LOCA design basis events have been reviewed to evaluate the impact of decreasing the minimum SIT pressure to 500 psig. For St. Lucie Unit 2, none of the non-LOCA events calculate or credit SIT injection into the RCS, since none of the non-LOCA accidents result in RCS depressurization below the SIT maximum pressure setpoint. Therefore, the reduction of the SIT minimum pressure setpoint has no increase in the consequences on non-LOCA design bases events due to reducing the minimum SIT pressure to 500 psig.

The station blackout event (SBO), which is presented in Section 15.10.5 of the [Updated Final Safety Analysis Report (UFSAR)], has been reviewed to evaluate the impact of decreasing the SIT minimum pressure to 500 psig. With the SIT pressure reduced to 500 psig, SIT injection is initiated prior to the occurrence of voiding in the RCS loops, thus preventing a loss of natural circulation.

Therefore, reducing the St. Lucie Unit 2 Technical Specification limit for SIT minimum nitrogen cover pressure from 570 psig to 500 psig does not involve a significant increase in the probability or consequences of an accident previously evaluated.

- (2) Use of the modified specification would not create the possibility of a new or different kind of accident from any previously evaluated.

There are no additional failure modes for the SITs due to reducing the nitrogen cover pressure from 570 psig to 500 psig. The SITs are passive systems and have no effect on the RCS until after the depressurization of the RCS due to a LOCA.

Therefore, reducing the St. Lucie Unit 2 Technical Specification limit for SIT minimum nitrogen cover pressure from 570 psig to 500 psig does not create the possibility of a new or different kind of accident from any previously evaluated.

- (3) Use of the modified specification would not involve a significant reduction in a margin of safety.

The SBLOCA analysis with the SIT pressure of 500 psig satisfies the criteria of 10 CFR 50.46 and remains bounded by the LBLOCA analysis of record. The LBLOCA analysis was performed assuming [an] SIT pressure of 200 psig. Review of the SBO analysis with the SIT pressure of 500 psig demonstrates that the original conclusions, presented in UFSAR Section 15.10.5, have not been adversely affected.

Therefore, reducing the St. Lucie Unit 2 Technical Specification limit for SIT minimum nitrogen cover pressure from [570] psig to 500 psig does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within thirty (30) days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Written comments may be submitted by mail to the Rules and Directives Review Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this FEDERAL

REGISTER notice. Written comments may also be delivered to Room P-223, Phillips Building, 7920 Norfolk Avenue, Bethesda, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555. The filing of requests for hearing and petitions for leave to intervene is discussed below.

By June 4, 1992, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the local public document room located at the Indian River Junior College Library, 3209 Virginia Avenue, Fort Pierce, Florida 34954-9003. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and

how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish

those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission

may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555, by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 325-6000 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to Herbert N. Berkow: petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to Harold F. Reis, Esquire, Newman and Holtzinger, 1615 L Street NW., Washington, DC 20036, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be

entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated April 21, 1992, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC 20555 and at the local public document room located at Indian River Junior College Library, 3209 Virginia Avenue, Fort Pierce, Florida 34954-9003.

Dated at Rockville, Maryland, this 29th day of April 1992.

FOR THE NUCLEAR REGULATORY COMMISSION



L. Raghavan, Acting Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation