February 8, 1994

Docket No. 50-389

DISTRIBUTION
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Mr. J. H. Goldberg
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

Dear Mr. Goldberg:

SUBJECT: ST. LUCIE UNIT 2 - ISSUANCE OF AMENDMENT RE: DELETION OF MOVABLE INCORE DETECTORS OPTION FROM TECHNICAL SPECIFICATIONS (TAC NO. M87723)

The Commission has issued the enclosed Amendment No. 64 to Facility Operating License No. NPF-16 for the St. Lucie Plant, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your application dated August 23, 1993.

This amendment will delete the option of using a movable incore detector to determine Incore Instrumentation System operability from the provisions of Technical Specification 3.3.3.2.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed By)

Jan A. Norris, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 64 to NPF-16
2. Safety Evaluation

cc w/enclosures:
See next page

OFC	:LA:PDII-2	:PM:PDII-2	:D:PDII-2	:OGC	:	:
NAME	:ETana	:JNorris	:HBerkow	:RBachman	:	:
DATE	: 1/24/94	: 1/25/94	: 2/8/94	: 1/29/94	:	:

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Florida Power and Light Company

St. Lucie Plant

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DATED: February 8, 1994

AMENDMENT NO. 64 TO FACILITY OPERATING LICENSE NO. NPF-16 - ST. LUCIE, UNIT 2

Docket File
NRC & Local PDRs
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER & LIGHT COMPANY

ORLANDO UTILITIES COMMISSION OF

THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 64
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated August 23, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

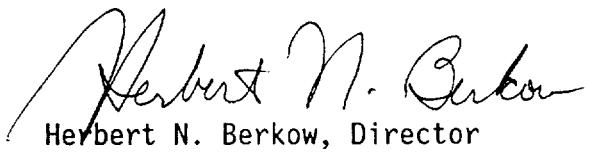
2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

   2. Technical Specifications

      The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 64 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 8, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 64

TO FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove Page	Insert Page
3/4 3-30	3/4 3-30

TABLE 4.3-3 (Continued)

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST	MODES FOR WHICH SURVEILLANCE IS REQUIRED
PROCESS MONITORS (Continued)				
c. Noble Gas Effluent Monitors				
i. Reactor Auxiliary Building Exhaust System (Plant Vent Low Range Monitor)	S	R	M	1, 2, 3 & 4
ii. Reactor Auxiliary Building Exhaust System (Plant Vent High Range Monitor)	S	R	M	1, 2, 3, & 4
iii. Steam Generator Blowdown Treatment Building Exhaust System	S	R	M	1, 2, 3 & 4
iv. Steam Safety Valve Discharge#	S	R	M	1, 2, 3 & 4
v. Atmospheric Steam Dump Valve Discharge#	S	R	M	1, 2, 3 & 4
vi. ECCS Exhaust	S	R	M	1, 2, 3 & 4

The steam safety valve discharge monitor and the atmospheric steam dump valve discharge monitor are the same monitor.

INCORE DETECTORS

LIMITING CONDITION FOR OPERATION

3.3.3.2 The incore detection system shall be OPERABLE with:

a. At least 75% of all incore detector locations, and

b. A minimum of two quadrant symmetric incore detector locations per core quadrant.

An OPERABLE incore detector location shall consist of a fuel assembly containing a fixed detector string with a minimum of three OPERABLE rhodium detectors.

APPLICABILITY: When the incore detection system is used for:

a. Recalibration of the excore axial flux offset detection system,

b. Monitoring the AZIMUTHAL POWER TILT,

c. Calibration of the power level neutron flux channels, or

d. Monitoring the linear heat rate.

ACTION:

a. With the incore detection system inoperable, do not use the system for the above applicable monitoring or calibration functions.

b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.2 The incore detection system shall be demonstrated OPERABLE:

a. By performance of a CHANNEL CHECK within 24 hours prior to its use and at least once per 7 days thereafter when required for:

1. Recalibration of the excore axial flux offset detection system,

2. Monitoring the linear heat rate pursuant to Specification 4.2.1.3,

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 64

TO FACILITY OPERATING LICENSE NO. NPF-16

FLORIDA POWER & LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNIT NO. 2

DOCKET NO. 50-389

1.0 INTRODUCTION

By letter dated August 23, 1993 Florida Power and Light Company (FPL) requested a change to the St. Lucie Unit 2 Technical Specifications for Incore Detector Instrumentation. The revision will delete the use of movable incore detectors as an alternative means of determining operability of an incore detector location. This will allow FPL to delete the Movable Incore Detection System from the plant.

The Incore Instrumentation System at St. Lucie Unit 2 consists of the Fixed Incore Detection System (FICDS) and the Movable Incore Detection System (MICDS). The functions of the incore system are:

1) To provide data sufficient to determine the gross power distribution in the core during different operating conditions from 20 percent to 100 percent power.
2) To provide data to estimate fuel burnup in each fuel assembly.
3) To provide data for the evaluation of thermal margins in the core.
4) To assist in the calibration of the excore neutron detectors by providing azimuthal and axial power distribution information.

The Incore Instrumentation System is not used for automatic protective or control functions. These functions are based on neutron flux provided by the excore neutron detector system.

The MICDS is not a safety-related system and was included in the St. Lucie plant design as a backup to the FICDS. Functions for which the MICDS could be used to back up the FICDS include:

1) detection/verification of fixed detector failures
2) confirmation of power distribution anomalies measured by the fixed detectors

3) substitution for the fixed detectors and
4) calibration of the fixed detectors.

2.0 EVALUATION

Experience with the rhodium detectors of the Fixed Incore Detection System has shown that they completely satisfy the functions of the Incore Instrumentation System. On-line calibration of these detectors is accomplished by means of a well-established rhodium detector sensitivity depletion correlation implemented via the Digital Data Processing System.

The MICDS has never been used at St. Lucie to assist in meeting the operability requirements for incore detector locations. No correlation between the FICDS and the MICDS has been made and thus it would not be possible to use a MICDS detector to replace a FICDS detector. Because of this the MICDS has become obsolete and is no longer a useful backup system for the FICDS.

Since the Movable Incore Detection System is not used for any of the Incore Instrumentation System functions and can not even serve as a backup to the Fixed Incore Detector System, it is acceptable that reference to the MICDS be removed from the Technical Specifications.

3.0 TECHNICAL SPECIFICATION CHANGES

Technical Specification 3.3.3.2 - The statement "or an OPERABLE movable incore detector capable of mapping the location" will be removed from the statement defining an OPERABLE incore detector location.

4.0 TECHNICAL FINDING

Based on the staff evaluation in Section 2.0 above, the staff finds that the proposed Technical Specification change is acceptable. This change is consistent with the Standard Technical Specifications.

5.0 STATE CONSULTATION

Based upon the written notice of the proposed amendment, the Florida State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (58 FR 52985). Accordingly, this amendment

meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Chatterton

Date: February 8, 1994