

November 1, 1994

Mr. J. H. Goldberg
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

Distribution
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SUBJECT: ST. LUCIE UNIT 2 - ISSUANCE OF AMENDMENT RE: DELETION OF ASME
CODE REFERENCE (TAC NO. M89642)

Dear Mr. Goldberg:

The Commission has issued the enclosed Amendment No. 68 to Facility Operating License No. NPF-16 for the St. Lucie Plant, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your application dated May 23, 1994.

This amendment revises Technical Specifications Section 3/4.7.1.1, Turbine Cycle, Safety Valves, to delete a specific reference to the 1974 edition of the ASME Code and refer to testing in accordance with Technical Specification 4.0.5, the In-Service Inspection and In-Service Testing Specification.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed By)

Jan A. Norris, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-389

Enclosures:

1. Amendment No. 68 to NPF-16
2. Safety Evaluation

cc w/enclosures:
See next page

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DATED: November 1, 1994

AMENDMENT NO. 68 TO FACILITY OPERATING LICENSE NO. NPF-16 - ST. LUCIE, UNIT 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER & LIGHT COMPANY

ORLANDO UTILITIES COMMISSION OF

THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 68
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated May 23, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

2. Technical Specifications

- The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 68, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Mohan Thadani, Acting Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 1, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 68

TO FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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3/4.7.1 TURBINE CYCLE

SAFETY VALVES

LIMITING CONDITION FOR OPERATION

3.7.1.1 All main steam line code safety valves shall be OPERABLE with lift settings and orifice sizes as shown in Table 3.7-2.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

- a. With both reactor coolant loops and associated steam generators in operation and with one or more main steam line code safety valves inoperable, operation in MODES 1, 2 and 3 may proceed provided that, within 4 hours, either the inoperable valve is restored to OPERABLE status or the Power Level-High trip setpoint is reduced per Table 3.7-1; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.1.1 No additional Surveillance Requirements other than those required by Specification 4.0.5.

ST. LUCIE-UNIT 2

3/4 7-2

Amendment No. 8

TABLE 3.7-1

MAXIMUM ALLOWABLE POWER LEVEL-HIGH TRIP SETPOINT WITH INOPERABLE
STEAM LINE SAFETY VALVES DURING OPERATION WITH BOTH STEAM GENERATORS

<u>Maximum Number of Inoperable Safety Valves on Any Operating Steam Generator</u>	<u>Maximum Allowable Power Level-High Trip Setpoint (Percent of RATED THERMAL POWER)</u>
1	92.8
2	79.6
3	66.3

ST. LUCIE-UNIT 2

3/4 7-3

Amendment No. 8, 68

TABLE 3.7-2
STEAM LINE SAFETY VALVES PER LOOP

	<u>VALVE NUMBER</u>		<u>LIFT SETTING ($\pm 1\%$)</u>	<u>ORIFICE SIZE</u>
	<u>Header A</u>	<u>Header B</u>		
a.	8201	8205	1000 psia	16 in. ²
b.	8202	8206	1000 psia	16 in. ²
c.	8203	8207	1000 psia	16 in. ²
d.	8204	8208	1000 psia	16 in. ²
e.	8209	8213	1040 psia	16 in. ²
f.	8210	8214	1040 psia	16 in. ²
g.	8211	8215	1040 psia	16 in. ²
h.	8212	8216	1040 psia	16 in. ²

PLANT SYSTEMS

AUXILIARY FEEDWATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.2 At least three independent steam generator auxiliary feedwater pumps and associated flow paths shall be OPERABLE with:

- a. Two feedwater pumps, each capable of being powered from separate OPERABLE emergency busses, and
- b. One feedwater pump capable of being powered from an OPERABLE steam supply system.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With one auxiliary feedwater pump inoperable, restore the required auxiliary feedwater pumps to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With two auxiliary feedwater pumps inoperable be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.
- c. With three auxiliary feedwater pumps inoperable, immediately initiate corrective action to restore at least one auxiliary feedwater pump to OPERABLE status.

SURVEILLANCE REQUIREMENTS

4.7.1.2 Each auxiliary feedwater pump shall be demonstrated OPERABLE:

- a. At least once per 31 days by:
 1. Verifying that each motor-driven pump develops a discharge pressure of greater than or equal to 1270 psig on recirculation flow.
 2. Verifying that the turbine-driven pump develops a discharge pressure of greater than or equal to 1260 psig on recirculation flow when the secondary steam supply pressure is greater than 50 psig. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3.
 3. Verifying that each valve (manual, power-operated, or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 68

TO FACILITY OPERATING LICENSE NO. NPF-16

FLORIDA POWER AND LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNIT NO. 2

DOCKET NO. 50-389

1.0 INTRODUCTION

The Code of Federal Regulations, 10 CFR 50.55a, requires that inservice testing (IST) of certain American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (the Code) Class 1, 2, and 3 pumps and valves be performed in accordance with Section XI of the Code and applicable addenda, except where alternatives have been authorized or relief has been requested by the licensee and granted by the Commission based on impracticalities and limitations of design, materials, or construction. The inservice testing requirements were added to 10 CFR 50.55a by rulemaking published in the Federal Register February 1976. At the time the inservice testing requirements were originally included, the latest edition of the ASME Code referenced in the regulation was the 1971 Edition, with addenda through the Summer 1973 Addenda. Subsequently, the Winter Addenda 1973 was added to the regulations in March 1976. The 1974 Edition with addenda through the Summer 1975 Addenda was added to the regulations in January 1977. Later editions and addenda have been incorporated by reference into 10 CFR 50.55a, with the most recent reference being the 1989 Edition which was incorporated by rulemaking effective September 8, 1992.

2.0 DISCUSSION

Florida Power & Light Company's letter dated May 23, 1994, submitted changes to the St. Lucie Plant, Unit 2, technical specifications. The proposed changes would modify the surveillance requirements specified for the main steam safety valves to be consistent with 10 CFR 50.55a requirements for inservice testing. Specifically, the changes propose the deletion of the reference to demonstration of operability of the valves in accordance with the 1974 Edition of the ASME Code. The changes are administrative.

3.0 EVALUATION

The St. Lucie Plant, Unit 2, received a construction permit May 2, 1977. The requirements in 10 CFR 50.55a were that during the initial 20-month interval (now 120-month interval), the inservice testing of pumps and valves for assessing operational readiness comply with those requirements in editions of the code and addenda in effect no more than six months prior to the start of

facility commercial operation. Commercial operation for St. Lucie Plant, Unit 2, commenced August 8, 1983. The regulations were later changed such that the IST for the initial 120-month interval was to comply with the Code Edition incorporated by reference in 10 CFR 50.55a(b) 12 months prior to the date of issuance of the operating license. The operating license for the St. Lucie Plant, Unit 2, was issued June 10, 1983. In subsequent intervals, the regulations require that the inservice testing program be updated to the latest code and addenda incorporated by reference in Paragraph (b) of 10 CFR 50.55a 12 months prior to the start of the interval. The second 120-month interval for the St. Lucie Plant, Unit 2, began August 8, 1993. The requirements of the 1986 Edition of Section XI of the ASME Code are implemented by the current inservice testing program at the plant.

The technical specifications for the St. Lucie Plant, Unit 2, were issued with reference to the 1974 Edition of the ASME Code in the surveillance requirements of Specification 4.7.1.1 for the main steam safety valves. Specification 4.0.5 provides the requirements for implementation of the inservice inspection and testing requirements for the St. Lucie Plant, Unit 2, which indicate that the inspection and testing are to be performed in accordance with Section XI of the ASME Code as required by 10 CFR 50.55a. As noted above, the current edition of Section XI of the ASME Code for implementation of the inservice testing requirements for the St. Lucie Plant, Unit 2, is the 1986 Edition. By deleting the reference to the 1974 Edition of the ASME Code, the technical specifications will no longer be inconsistent internally or with the regulations.

No safety concerns are created because the main steam safety valves will continue to be testing in accordance with the requirements of the ASME Code pursuant to Specification 4.0.5. No changes to the lift settings are proposed as part of this change. Therefore, it is acceptable to delete the following sentence in Specification 4.7.1.1:

Each main steam line code safety valve shall be demonstrated operable, with lift settings and orifice sizes as shown in Table 4.7-0, in accordance with Section XI of the ASME Boiler and Pressure Vessel Code, 1974 Edition.

Table 4.7-0, which lists the safety valve settings, will be renumbered to "Table 3.7-2," which changes the character of the table from a "surveillance" requirement to a "limiting condition for operation." Such a change in the character of the table is conservative, as it establishes a condition that more specifically applies at any time the plant is in modes 1, 2, and 3, and not just during testing. Specification 3.7.1.1 will be revised to include a reference to Table 3.7-2, so that it will now state: "All main steam line code safety valves shall be operable *with lift settings and orifice sizes as shown in Table 3.7-2*. The sentence that will replace the deletion in Specification 4.7.1.1 will state:

No additional Surveillance Requirements other than those required by Specification 4.0.5.

The only other change is the table number in the "List of Tables" in the technical specifications index.

4.0 TECHNICAL FINDING

The proposed changes are acceptable for achieving consistency in the surveillance requirements specified for the main steam safety valves and ensuring that the testing of the lift settings are subject to the updated inservice testing requirements for the current and later inservice test intervals.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (59 FR 34664). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Campbell

Date: November 1, 1994