

October 25, 1995

TRIBUTION

attached sheet

Mr. J. H. Goldberg
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: ST. LUCIE UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: RELOCATION
OF AUDIT MINIMUM FREQUENCY CRITERIA TO THE TOPICAL QUALITY
ASSURANCE REPORT (TQAR) (TAC NOS. M88978 AND M88979)

Dear Mr. Goldberg:

The Commission has issued the enclosed Amendment Nos. 140 and 80 to Facility
Operating License Nos. DPR-67 and NPF-16 for the St. Lucie Plant, Unit Nos. 1
and 2. These amendments consist of changes to the Technical Specifications in
response to your application dated February 28, 1994 and modified by letter
dated September 25, 1995.

These amendments delete the minimum frequency criteria prescribed for quality
assurance audits from Administrative Controls sections 6.5.2.8 and 6.8.4 of
the Technical Specifications (TS). Audit periodicity will thereby be
controlled by the program described in the Florida Power and Light Company
(FPL) TQAR.

A copy of the Safety Evaluation is enclosed. The Notice of Issuance will be
included in the Commission's biweekly Federal Register notice.

Sincerely,
Original signed by:
Jan A. Norris, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-335 and 50-389

Enclosures:

1. Amendment No. 140 to DPR-67
2. Amendment No. 80 to NPF-16
3. Safety Evaluation

NRC FILE CENTER COPY

cc w/enclosures:
See next page

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OFFICE	LA:PDII-2	PM:PDII-2	D:PDII	HQMB*	SPLB*
NAME	Dunnington ^{E10}	JNorris	DMatthews	RGramm	SWest
DATE	10/18/95	10/18/95	10/25/95	07/31/95	08/04/95
COPY	Yes/No	Yes/No	Yes/No	Yes/No	Yes/No
OFFICE	OGC	HQMB*			
NAME	OPW	SBlack			
DATE	10/23/95	07/31/95			
COPY	Yes/No	Yes/No			

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DATED: October 25, 1995

AMENDMENT NO. 140 TO FACILITY OPERATING LICENSE NO. DPR-67 - ST. LUCIE, UNIT 1
AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NO. NPF-16 - ST. LUCIE, UNIT 2

Docket File

PUBLIC

PDII-1 Reading

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Mr. J. H. Goldberg
Florida Power and Light Company

St. Lucie Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 140
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated February 28, 1994 and modified by letter dated September 25, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 140, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 25, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 140
TO FACILITY OPERATING LICENSE NO. DPR-67
DOCKET NO. 50-335

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages

6-11
6-14

Insert Pages

6-11
6-14

ADMINISTRATIVE CONTROLS

- g. All REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meeting minutes of the Facility Review Group.

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the CNRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training and qualifications of the entire unit staff.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50.
- e. Any other area of unit operation considered appropriate by the CNRB or the President - Nuclear Division.
- f. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel.
- g. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year.
- h. The radiological environmental monitoring program and the results thereof.
- i. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures.
- j. The PROCESS CONTROL PROGRAM and implementing procedures for dewatering of radioactive bead resin.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.2.9 The CNRB shall report to and advise the President - Nuclear Division on those areas of responsibility specified in Specifications 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of CNRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each CNRB meeting shall be prepared, approved and forwarded to the President - Nuclear Division within 14 days following each meeting.
- b. Reports of reviews encompassed by Specification 6.5.2.7 above, shall be prepared, approved and forwarded to the President - Nuclear Division within 14 days following completion of the review.
- c. Audit reports encompassed by Specification 6.5.2.8 above, shall be forwarded to the President - Nuclear Division and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the FRG, and the results of the review shall be submitted to the CNRB, and the President - Nuclear Division.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The President - Nuclear Division and the CNRB shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the FRG. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

6.0 ADMINISTRATIVE CONTROLS

- c. The change is documented, reviewed by the FRG and approved by the Plant General Manager within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, maintained, and shall be audited under the cognizance of the CNRB:

- a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the Shutdown Cooling System, High Pressure Safety Injection System, Containment Spray System, and RCS Sampling. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.

- b. In-Plant Radioiodine Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for monitoring, and
- (iii) Provisions for maintenance of sampling and analysis equipment.

- c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- (i) Identification of a sampling schedule for the critical variables and control points for these variables,
- (ii) Identification of the procedures used to measure the values of the critical variables,
- (iii) Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in-leakage,



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER & LIGHT COMPANY

ORLANDO UTILITIES COMMISSION OF

THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 80
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated February 28, 1994 and modified by letter dated September 25, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

2. Technical Specifications

- The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 80 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 25, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 80
TO FACILITY OPERATING LICENSE NO. NPF-16
DOCKET NO. 50-389

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages

6-11
6-12
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Insert Pages

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ADMINISTRATIVE CONTROLS

REVIEW

6.5.2.7 The CNRB shall review:

- a. The safety evaluations for (1) changes to procedures, equipment, or systems and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment, or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. ALL REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meetings minutes of the Facility Review Group.

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the CNRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training and qualifications of the entire unit staff.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety.

AUDITS (continued)

- d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50.
- e. Any other area of unit operation considered appropriate by the CNRB or the President - Nuclear Division.
- f. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel.
- g. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year.
- h. The radiological environmental monitoring program and the results thereof.
- i. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures.
- j. The PROCESS CONTROL PROGRAM and implementing procedures for dewatering of radioactive bead resin.

TECHNICAL REVIEW RESPONSIBILITIES

6.5.2.9 The technical review responsibilities under the cognizance of the CNRB shall encompass:

- a. Plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports and other sources that may indicate areas for improving plant safety:
- b. Plant operations, modifications, maintenance, and surveillance to verify independently that these activities are performed safely and correctly and that human errors are reduced as much as practical;
- c. Internal and external operational experience information that may indicate areas for improving plant safety; and
- d. Making detailed recommendations through the Chairman - CNRB for revising procedures, equipment modifications or other means of improving nuclear safety and plant reliability.

AUTHORITY

6.5.2.10 The CNRB shall report to and advise the President - Nuclear Division on those areas of responsibility specified in Specifications 6.5.2.7, 6.5.2.8, and 6.5.2.9.

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Control Program for effluent monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974.
- j. Quality Control Program for environmental monitoring using the guidance in Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each procedure of Specification 6.8.1a. through i. above, and changes thereto, shall be reviewed by the FRG and shall be approved by the Plant General Manager prior to implementation and shall be reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of Specification 6.8.1a. through i. above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the FRG and approved by the Plant General Manager within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, maintained, and shall be audited under the cognizance of the CNRB:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the Shutdown Cooling System, High Pressure Safety Injection System, Containment Spray System, and RCS Sampling. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radioiodine Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for monitoring, and
- (iii) Provisions for maintenance of sampling and analysis equipment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 140 AND 80

TO FACILITY OPERATING LICENSE NO. DPR-67 AND NO. NPF-16

FLORIDA POWER AND LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-335 AND 50-389

1.0 INTRODUCTION

By letter dated February 28, 1994, Florida Power & Light Company (FPL or the licensee) requested changes to the Administrative Controls Section of the Technical Specifications (TS) for St. Lucie Units 1 and 2. The substance of the TS changes is to delete the frequencies from the audits performed under the cognizance of the Company Nuclear Review Board (CNRB) from TS. The audits would be performed at the frequency specified in the FPL Topical Quality Assurance Report (TQAR). Some audits would be performed less frequently (biennial rather than annual). Subsequent to that, by letter dated September 25, 1995, FPL withdrew that portion of the request which relates to the Fire Protection Program audit frequencies.

2.0 BACKGROUND

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to state TS to be included as part of the license. The Commission's regulatory requirements related to the content of TS are set forth in 10 C.F.R. § 50.36. That regulation requires that the TS include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TS.

The Commission has provided guidance for the contents of TS in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" ("Final Policy Statement"), 58 Fed. Reg. 39132 (July 22, 1993), in which the Commission indicated that compliance with the Final Policy Statement satisfies Section 182a of the Act. In particular, the Commission indicated that certain items could be relocated from the TS to licensee-controlled documents, consistent with the standard enunciated in Portland General Electric Co. (Trojan Nuclear Plant), ALAB-531, 9 NRC 263, 273 (1979). In that

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case, the Atomic Safety and Licensing Appeal Board indicated that "technical specifications are to be reserved for those matters as to which the imposition of rigid conditions or limitations upon reactor operation is deemed necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety."

Consistent with this approach, the Final Policy Statement identified four criteria to be used in determining whether a particular matter is required to be included in the TS, as follows: (1) Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a Design Basis Accident or Transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a Design Basis Accident or Transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety.

The Commission's policy statement provides that many of the existing TS limiting conditions for operation which do not satisfy these four specified criteria may be relocated to the Updated Final Safety Analysis Report (UFSAR), such that future changes could be made to these provisions pursuant to 10 CFR 50.59. Other requirements may be relocated to more appropriate documents (e.g. Security Plan, Quality Assurance (QA) plan, and Emergency Plan) and controlled by the applicable regulatory requirement. While the content of the TS administrative controls is specified in 10 CFR 50.36(c)(5), particular details of the administrative controls may be relocated to licensee-controlled documents where §50.59 or comparable regulatory controls exist.

Administrative controls in existing TS related to the review and audit functions, including specified frequency provisions, should be relocated to a licensee-controlled document that provides adequate control over changes to these provisions and which provides an appropriate change control mechanism. As such, these review and audit provisions should be relocated to the Quality Assurance Program described or referenced in the facility's UFSAR and controlled pursuant to 10 CFR 50.54.

3.0 DESCRIPTION AND EVALUATION

3.1 GENERAL

The licensee proposed that the audit frequencies specified in existing TS (TS 6.5.2.8a., b., c., d., h., i., j., and TS 6.8.4) be relocated from the TS to the TQAR, such that future changes could be made pursuant to 10 CFR 50.54(a). In addition, the licensee requested to extend audit frequency (maximum time between audits) from 12 months to 24 months in four cases and from 6 months to

24 months in one case. These particular TS provisions are not necessary to assure safe operation of the facility, given that the requirements in the QA program implement the Commission's regulations pertaining to these review and audit functions as set forth below. The review and audit functions define an administrative framework to confirm that plant activities have been properly conducted in a safe manner. The reviews and audits serve also to provide a cohesive program that provides senior level utility management with assessments of facility operation and recommends actions to improve nuclear safety and reliability. As such, the review and audit program does not include any elements that are delineated in the Final Policy Statement criteria, as discussed above, for determining which limiting conditions are required to be included in the TS. As documented in the Final Policy Statement, the review and audit functions constitute requirements that can be relocated to the Quality Assurance plan and controlled by the applicable regulatory requirement. Such an approach would result in an equivalent level of regulatory authority while providing for a more appropriate change control process.

In addition, audit requirements including frequencies are specified in the QA program to satisfy 10 CFR Part 50, Appendix B, Criterion XVIII. The licensee relies upon the guidance in ANSI N18.7 and ANSI N45.2 to meet the requirements of Appendix B to 10 CFR Part 50. Audits are also governed by 10 CFR 50.54(t), 10 CFR 50.54(p), and 10 CFR Part 73. Therefore, duplication of these requirements does not enhance the level of plant safety. Control of changes to the QA program description are governed by the provisions of 10 CFR 50.54(a).

3.2 TS 6.5.2.8 a., b., c., d., h., i., j., AND TS 6.8.4

The licensee proposed deleting from TS the specific audit intervals associated with the audits of unit activities performed under the cognizance of the CNRB. This change relocates the frequency requirement for audit from TS to the licensee's TQAR. The licensee also proposed a change which would allow the specific audit frequency to be adjusted based upon programmatic performance up to a maximum of 2 years.

The licensee's TQAR establishes a minimum biennial frequency unless more frequent audits are otherwise mandated by regulatory requirements or license commitments. The administrative controls described in the TQAR are consistent with regulatory guidance and industry standards for QA Programs and provide for scheduling quality assurance audits based on the safety significance and extent of plant activities. Follow-up action, including re-audit of deficient areas, where necessary, is also required. The TQAR was found acceptable by the NRC staff for satisfying the Quality Assurance Criteria of 10 CFR 50, Appendix B, and changes to this document are controlled pursuant to 10 CFR 50.54(a). 10 CFR 50.54(a)(3) states that the licensee may make changes to the TQAR provided the change does not reduce the commitments in the program description previously accepted by the NRC.

The QA program's performance will continue to be assessed through self-assessments, management reviews, performance evaluations and audits in related areas, and other trend indicators. Reducing the audit frequency requirements to biennial (2 years) will not impact compliance with provisions of the TS, but will provide for more meaningful audits of those activities conducted less frequently than annually (i.e., refueling activities), and allow for additional effort on operational activities.

4.0 TECHNICAL FINDING

The staff concludes that removing the audit frequencies from the TS and relocating them to the TQAR is acceptable since they do not need to be controlled by TS and sufficient regulatory controls exist for any future changes to the TQAR. Also the staff concludes that reducing the audit frequency is acceptable since sufficient guidance will exist in the TQAR to ensure audits are conducted on a performance basis and audits would be performed more frequently than the minimum specified for areas demonstrating poor or questionable performance. The staff finds, therefore, that these changes are acceptable.

5.0 STATE CONSULTATION

Based upon the written notice of the proposed amendments, the Florida State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Norris

Date: October 25, 1995