November 14, 1985

Docket No. 50-389

Mr. J. W. Williams, Jr. Vice President Nuclear Energy Department Florida Power & Light Company P. O. Box 14000 Juno Beach, Florida 33408 DISTRIBUTION: Docket File NRC & LPDRs EReeves ORB#3 Rdg HThompson PMKreutzer-3 DSells AThadani LJHarmon SECY BGrimes RDiggs LTremper OPA, CMiles ACRS-10 MVirgilio WJones TBarnhart-4 EJordan OELD Grav File +4

Dear Mr. Williams:

The Commission has issued the enclosed Amendment No. 12to Facility Operating License No. NPF-16 for the St. Lucie Plant, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your application dated August 31, 1984 as modified April 12, 1985.

This amendment would amend the Technical Specifications to limit the use of the 8-inch containment purge system to required safety-related purposes.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next bi-weekly <u>Federal Register</u> notice.

Sincerely,

/S/

Donald E. Sells, Project Manager Operating Reactors Branch #3 Division of Licensing

Enclosures: 1. Amendment No.12 to NPF-16

2. Safety Evaluation

cc w/enclosures: See next page

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Mr. J. W. Williams, Jr. Florida Power & Light Company

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON. D. C. 20555

FLORIDA POWER & LIGHT COMPANY

ORLANDO UTILITIES COMMISSION OF

THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 12 License No. NPF-16

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, et al., (the licensee) dated August 31, 1984, as modified April 12, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

 Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 12, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Edward J. Butcher, Acting Chief Operating Reactors Branch #3 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: November 12, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 12

TO FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove Pages	<u>Insert Pages</u>	
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3/4 6-14

CONTAINMENT SYSTEMS

CONTAINMENT VESSEL STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.6 The structural integrity of the containment vessel shall be maintained at a level consistent with the acceptance criteria in Surveillance Requirement 4.6.1.6.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the structural integrity of the containment vessel not conforming to the above requirements, restore the structural integrity to within the limits prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.6 The structural integrity of the containment vessel shall be determined during the shutdown for each Type A containment leakage rate test (reference Surveillance Requirement 4.6.1.2) by a visual inspection of the exposed accessible interior and exterior surfaces of the vessel and verifying no apparent changes in appearance of the surfaces or other abnormal degradation. Any abnormal degradation of the containment vessel detected during the above required inspections shall be reported to the Commission pursuant to Specification 6.9.1.

3/4 6-13

CONTAINMENT SYSTEMS

CONTAINMENT VENTILATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.6.1.7 Each containment purge supply and exhaust isolation valve shall be OPERABLE and:

- a. Each 48-inch containment purge supply and exhaust isolation valve shall be sealed closed.
- b. The 8-inch containment purge supply and exhaust isolation valves may be open for purging and/or venting as required for safety related purposes such as:
 - 1. Maintaining containment pressure within the limits of Specification 3.6.1.4.
 - 2. Reducing containment atmosphere airborne radioactivity and/or improving air quality to an acceptable level for containment access.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With a 48-inch containment purge supply and/or exhaust isolation valve(s) open or not sealed closed, close and/or seal close the open valve(s) or isolate the penetration(s) within 4 hours, otherwise be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With an 8-inch containment purge supply and/or exhaust isolation valve(s) open for reasons other than those stated in Specification 3.6.1.7.b, close | the open 8-inch valve(s) or isolate the penetration(s) within 4 hours, otherwise be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With a containment purge supply and/or exhaust isolation valve(s) having a measured leakage rate exceeding the limits of Surveillance Requirements 4.6.1.7.3 and/or 4.6.1.7.4, restore the inoperable valve(s) to OPERABLE status within 24 hours, otherwise be in at least HOT STANDY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.7.1 Each 48-inch containment purge supply and exhaust isolation valve shall be verified to be sealed-closed at least once per 31 days.

4.6.1.7.2 Documentation shall be reviewed every 18 months to confirm that purging and venting were performed in accordance with Specification 3.6.1.7.b.

4.6.1.7.3 At least once per 6 months on a STAGGERED TEST BASIS each sealed closed 48-inch containment purge supply and exhaust isolation valve with resilient material seals shall be demonstrated OPERABLE by verifying that the measured leakage rate is less than or equal to 0.05 L_a when pressurized to P_a .

4.6.1.7.4 Each 8-inch containment purge supply and exhaust isolation valve with resilient material seals shall be demonstrated OPERABLE by verifying that the measured leakage rate is less than or equal to 0.05 L_a when pressurized to P_a prior to entering MODE 4 from COLD SHUTDOWN if not tested within the previous 31 days.

ST. LUCIE - UNIT 2



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 12

TO FACILITY OPERATING LICENSE NO. NPF-16

FLORIDA POWER & LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNIT NO. 2

DOCKET NO. 50-389

Introduction

Evaluation

Based on a review of the FP&L proposal, the staff concluded that simply deleting the 1000 hour limit was not acceptable. Unrestricted use of the system is not permissable; the 8-inch containment purge supply and exhaust isolation valves. should only be opened for safety-related purposes. After several discussions with the licensee, FP&L amended their proposal by letter dated April 12, 1985. The proposed TS were revised as shown in the enclosure to that letter to limit use of the 8-inch containment purge system for required safety-related purposes, such as, (1) maintaining containment pressure within the TS limit and (2) reducing containment atmosphere activity and/or improving air quality to an acceptable level for containment entry to conduct safety-related tasks. The stated action to be taken if the 8-inch purge valves are open for reasons other than as specified above were also included in the TS change. Furthermore, the licensee is required to conduct documentation reviews once per 18 months to ascertain that operation of the 8-inch containment purge system is in accordance with the TS. The licensee has demonstrated in their submittal that the St. Lucie 2 containment purge system design meets the requirements of Branch Technical Position CSB 6-4, "Containment Purging During Normal Plant Operations," Revision 2. The licensee also analyzed the radiological consequences of a loss-of-coolant accident occurring during purging, assuming a 5 second isolation valve closure time; the results remain within 10 CFR Part 100 guidelines. The purge system will not be used for temperature and humidity control within the containment since a separate internal ventilation system is provided for that purpose.

The staff therefore, concludes that it is acceptable to revise the TS in a way that emphasizes using the 8-inch containment purge system for safety-related purposes and requires recordkeeping for periodic accountability, rather than merely prescribing an annual time limit for system use.

Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and a change in surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR S51.22(c)(9). Pursuant to 10 CFR S51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: November 14, 1985

Principal Contributors: C. Li D. Sells