

September 30, 1986

PUR 016

Docket No. 50-389

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Mr. C. O. Woody
 Group Vice President
 Nuclear Energy
 Florida Power & Light Company
 P. O. Box 14000
 Juno Beach, Florida 33408

Dear Mr. Woody:

The Commission has issued the enclosed Amendment No.15 to Facility Operating License No. NPF-16 for the St. Lucie Plant, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your application dated July 22, 1986.

This amendment deletes the references to the maximum enrichment of reload fuel in Technical Specification 5.3.1.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

/S/

E. G. Tourigny, Project Manager
 PWR Project Directorate #8
 Division of PWR Licensing-B

Enclosures:

1. Amendment No.15 to NPF-16
2. Safety Evaluation

cc w/enclosures:

See next page

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No. legal advisor
[Signature]

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Mr. C. O. Woody
Florida Power & Light Company

St. Lucie Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

ORLANDO UTILITIES COMMISSION OF

THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated July 22, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 15, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ashok C. Thadani, Director
PWR Project Directorate #8
Division of PWR Licensing-B

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 30, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 15
TO FACILITY OPERATING LICENSE NO. NPF-16
DOCKET NO. 50-389

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove Page

5-3

Insert Page

5-3

DESIGN FEATURES

5.3 REACTOR CORE

FUEL ASSEMBLIES

5.3.1 The reactor core shall contain 217 fuel assemblies with each fuel assembly containing 236 fuel and poison rod locations. All fuel and poison rods are clad with Zircaloy-4. Each fuel rod shall have a nominal active fuel length of 136.7 inches and contain approximately 1700 grams uranium. The initial core loading shall have a maximum enrichment of 2.73 weight percent U-235. Reload fuel shall be similar in physical design to the initial core loading.

CONTROL ELEMENT ASSEMBLIES

5.3.2 The reactor core shall contain 91 full-length control element assemblies and no part-length control element assemblies.

5.4 REACTOR COOLANT SYSTEM

DESIGN PRESSURE AND TEMPERATURE

5.4.1 The Reactor Coolant System is designed and shall be maintained:

- a. In accordance with the code requirements specified in Section 5.2 of the FSAR with allowance for normal degradation pursuant to the applicable Surveillance Requirements,
- b. For a pressure of 2485 psig, and
- c. For a temperature of 650°F, except for the pressurizer which is 700°F.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 15

TO FACILITY OPERATING LICENSE NO. NPF-16

FLORIDA POWER & LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNIT NO. 2

DOCKET NO. 50-389

1.0 INTRODUCTION

By letter (Ref. 1) dated July 22, 1986 the Florida Power and Light Company (the licensee) made application to amend the Technical Specifications of the St. Lucie Unit 2 facility. The proposed amendment would delete the reference to the maximum enrichment of reload fuel in Technical Specification 5.3.1. The staff has reviewed the proposed amendment and prepared the following evaluation.

2.0 EVALUATION

Technical Specification 5.3.1 appears in the Design Features section of the Technical Specifications and concerns fuel assemblies. The purpose of this Technical Specification is to provide the nominal design characteristics of fuel assemblies. This inclusion of the maximum enrichment in weight percent of the uranium-235 isotope in this Technical Specification is unnecessary for the following reasons:

- (1) The fuel enrichment appears implicitly in the nuclear design analysis of the core and it affects, directly or indirectly, all safety related parameters (e.g., reactor criticality, power distributions, reactivity coefficients, control rod worths, etc.). This nuclear design analysis includes all normal operation, transients, and accident analyses. The results of this nuclear design analysis establish or confirm existing Limiting Conditions for Operation Technical Specifications.
- (2) The fuel enrichment is limited by Technical Specification 5.6.1 to 4.5 weight percent enrichment for both the spent fuel storage racks and the dry storage rack.

On the basis of the staff's review, which is described above, the staff concludes that the proposed change to Technical Specification 5.3.1 is acceptable.

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3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

5.0 REFERENCES

1. Letter from C. O. Woody (Florida Power & Light Company) to Ashok C. Thadani (NRC), L-86-302, July 22, 1986.

Date: September 30, 1986

Principal Contributor:
D. Fieno