

March 1, 1985

Docket No. 50-389

DISTRIBUTION:

Mr. J. W. Williams, Jr.
Vice President
Nuclear Energy Department
Florida Power & Light Company
P. O. Box 14000
Juno Beach, Florida 33408

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Dear Mr. Williams:

The Commission has issued the enclosed Amendment No. 9 to Facility Operating License No. NPF-16 for the St. Lucie Plant, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your application dated November 23, 1981.

The amendment changes License Condition 2.C.(1) and the Technical Specifications to authorize operation of St. Lucie Unit 2 at 2700 Megawatts thermal power. The previously authorized maximum power level was 2560 Megawatts thermal.

The Cycle 2 reload safety report for St. Lucie 2 presented analyses (at a core power level of 2700 Mwt) for the entire spectrum of anticipated operational occurrences and postulated accidents. The staff reviewed and evaluated these analyses from the standpoint of acceptable safety limits and design criteria.

These included:

- 1) specified acceptable fuel design limits (SAFDL)
- 2) loss of coolant accident (LOCA) criteria specified in 10 CFR 50.46
- 3) offsite dose limits
- 4) reactor coolant pressure limits, and
- 5) containment design criteria

The consequences of the above events were found to meet all safety limits and criteria. In addition, the analytical methods used were previously found acceptable by the NRC.

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A copy of the Safety Evaluation and of the notice of issuance are also enclosed.

Sincerely,

/S/

Donald E. Sells, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:

- 1. Amendment No. 9 to NPF-16
- 2. Safety Evaluation
- 3. Notice

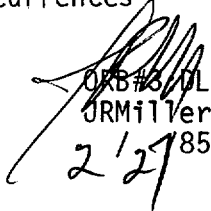
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**See previous white for concurrences

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PKreutzer
2/8/85

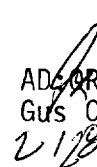
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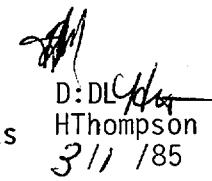


OELD **
WDPaton
2/11/85

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Gus C. Lainas
2/12/85



D:DL
HThompson
3/1/85



Docket No. 50-389

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Mr. J. W. Williams, Jr.
 Vice President
 Nuclear Energy Department
 Florida Power & Light Company
 P. O. Box 14000
 Juno Beach, Florida 33408

Dear Mr. Williams:

The Commission has issued the enclosed Amendment No. 9 to Facility Operating License No. NPF-16 for the St. Lucie Plant, Unit No. 2. This amendment consists of changes to the Technical Specifications in response to your application dated November 21, 1984.

The amendment revises the Technical Specifications to allow operation of St. Lucie 2 at a power level of 2700 Mwt.

A copy of the Safety Evaluation and of the notice of issuance are also enclosed.

Sincerely,

Donald E. Sells, Project Manager
 Operating Reactors Branch #3
 Division of Licensing

Enclosures:

1. Amendment No. 9 to NPF-16
2. Safety Evaluation
3. Notice

cc w/enclosures:
 See next page

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 DSells
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 JRMiller
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 W.D. Paton
 2/11/85

AD:OR:DL
 GCLatnas
 2/15/85

D:DL
 HThomson
 / /85

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c/o U.S. NRC
Senior Resident Inspector
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Regional Administrator
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Region IV Office
ATTN: Regional Radiation Representative
345 Courtland Street, NE
Atlanta, Georgia 30308



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY
ORLANDO UTILITIES COMMISSION OF
THE CITY OF ORLANDO, FLORIDA
AND
FLORIDA MUNICIPAL POWER AGENCY
DOCKET NO. 50-389
ST. LUCIE PLANT UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, et al., (the licensee) dated November 21, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraphs 2.C.1 and 2.C.2 to read as follows:

1. Maximum Power level

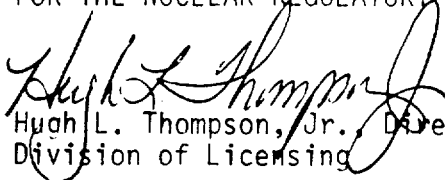
Florida Power and Light Company is authorized to operate the facility at reactor core power levels not in excess of 2700 megawatts thermal (100% power).

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 9, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Hugh L. Thompson, Jr., Director
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 1, 1985

TO FACILITY OPERATING LICENSE NO. NPF-16
PRESSURE BOUNDARY LEAKAGE

DOCKET NO. 50-389

1.22 PRESSURE BOUNDARY LEAKAGE shall be leakage (except steam generator tube leakage) through a non-isolable fault in a Reactor Coolant System component body, pipe wall, or vessel wall. Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

1.23 The PROCESS CONTROL PROGRAM (PCP) shall contain the provisions, based on full scale testing, to assure that dewatering of spent bead resins results in a waste form with the properties that meet the requirements of 10 CFR Part 61 (as implemented by 10 CFR Part 20) and of the low level radioactive waste disposal site at the time of disposal.

1-5

PURGE - PURGING

1.24 PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

RATED THERMAL POWER

1.25 RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 2700 MWt.

REACTOR TRIP SYSTEM RESPONSE TIME

1.26 The REACTOR TRIP SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its trip setpoint at the channel sensor until electrical power is interrupted to the CEA drive mechanism.

REPORTABLE OCCURRENCE

1.27 A REPORTABLE OCCURRENCE shall be any of those conditions specified in Specifications 6.9.1.8 and 6.9.1.9.

SHIELD BUILDING INTEGRITY

1.28 SHIELD BUILDING INTEGRITY shall exist when:

- a. Each door is closed except when the access opening is being used for normal transit entry and exit;
- b. The shield building ventilation system is in compliance with Specification 3.6.6.1, and
- c. The sealing mechanism associated with each penetration (e.g., welds, bellows or O-rings) is OPERABLE.

DEFINITIONS

PRESSURE BOUNDARY LEAKAGE

1.22 PRESSURE BOUNDARY LEAKAGE shall be leakage (except steam generator tube leakage) through a non-isolable fault in a Reactor Coolant System component body, pipe wall or vessel wall.

PROCESS CONTROL PROGRAM (PCP)

1.23 The PROCESS CONTROL PROGRAM shall contain the provisions, based on full scale testing, to assure that dewatering of spent bead resins results in a waste form with the properties that meet the requirements of 10 CFR Part 61 (as implemented by 10 CFR Part 20) and of the low level radioactive waste disposal site at the time of disposal.

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1.24 PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

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- a. Each door is closed except when the access opening is being used for normal transit entry and exit;
- b. The shield building ventilation system is in compliance with Specification 3.6.6.1, and
- c. The sealing mechanism associated with each penetration (e.g., welds, bellows or O-rings) is OPERABLE.

DEFINITIONS

SHUTDOWN MARGIN

1.29 SHUTDOWN MARGIN shall be the instantaneous amount of reactivity by which the reactor is subcritical or would be subcritical from its present condition assuming all full-length control element assemblies (shutdown and regulating) are fully inserted except for the single assembly of highest reactivity worth which is assumed to be fully withdrawn.

SITE BOUNDARY

1.30 The SITE BOUNDARY shall be that line beyond which the land is neither owned, leased, nor otherwise controlled by the licensee.

SOURCE CHECK

1.31 A SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

STAGGERED TEST BASIS

1.32 A STAGGERED TEST BASIS shall consist of:

- a. A test schedule for n systems, subsystems, trains or other designated components obtained by dividing the specified test interval into n equal subintervals, and
- b. The testing of one system, subsystem, train or other designated component at the beginning of each subinterval.

THERMAL POWER

1.33 THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.

UNIDENTIFIED LEAKAGE

1.34 UNIDENTIFIED LEAKAGE shall be all leakage which is not IDENTIFIED LEAKAGE or CONTROLLED LEAKAGE.

UNRESTRICTED AREA

1.35 An UNRESTRICTED AREA shall be any area at or beyond the SITE BOUNDARY access to which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation and radioactive materials, or any area within the SITE BOUNDARY used for residential quarters or for industrial, commercial, institutional, and/or recreational purposes.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 9

TO FACILITY OPERATING LICENSE NO. NPF-16

FLORIDA POWER & LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNIT NO. 2

DOCKET NO. 50-389

INTRODUCTION

By letter dated November 21, 1984, the Florida Power & Light Company (FP&L) submitted a request to amend Operating License No. NPF-16 for the St. Lucie Plant, Unit No. 2 (Ref. 1). The requested amendment would increase the presently licensed rated core power level of 2560 MWt to 2700 MWt. The NRC has previously approved the FP&L request to reload and operate Unit 2 of the St. Lucie Plant for Cycle 2 (Ref. 2). The staff approval was based on the reload safety analysis report (Ref. 3) and other supporting documents submitted by FP&L.

EVALUATION

The licensee's analysis incorporated and bounded operation for core power levels up to 2700 MWt, though operation at this power was not requested by the licensee at the time of the reload submittal. NRC approval was, therefore, given for operation of Cycle 2 at a continued power level of 2560 MWt, the same core power approved and licensed for the initial fuel cycle operation.

The previously approved reload safety report for Cycle 2 presented analyses (at a core power level of 2700 MWt) for the entire spectrum of anticipated operational occurrences and postulated accidents. The staff reviewed and evaluated these analyses from the standpoint of acceptable safety limits and design criteria. These included specified acceptable fuel design limits (SAFDL), loss of coolant accident (LOCA) criteria specified in 10 CFR 50.46, offsite dose limits, reactor coolant pressure limits, and containment design criteria. The consequences of these events were found to meet all safety limits and criteria. In addition, the analytical methods used were previously found acceptable by the NRC. The staff, therefore, concludes that St. Lucie Plant Unit No. 2 can be operated at a rated thermal power level of 2700 MWt.

ENVIRONMENTAL CONSIDERATION

In connection with the issuance of this amendment, an "Environmental Assessment and Finding of No Significant Impact" was prepared and published in the Federal Register on (50 FR 7422).

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 1, 1985

Principal Contributors:

L. Kopp
D. Sells

REFERENCES

1. Letter from J. W. Williams, Jr. to D. G. Eisenhut, L-84-354, St. Lucie Unit No. 2, Docket No. 50-389, Proposed License Amendment, Stretch Power, dated November 21, 1984.
2. St. Lucie Plant Unit 2 Amendment to Facility Operating License; Amendment No. 8; License No. NPF-16; November 9, 1984.
3. Letter from J. W. Williams, Jr. to D. G. Eisenhut, L-84-148, transmitting Reload Safety Report, St. Lucie 2 Cycle 2, Operation at 2560 MWt, June 4, 1984.

UNITED STATES NUCLEAR REGULATORY COMMISSION

FLORIDA POWER & LIGHT COMPANY, ET AL.

DOCKET NO. 50-389

ST. LUCIE PLANT UNIT NO. 2

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 9 to Facility Operating License No. NPF-16 issued to Florida Power & Light Company, the Orlando Utilities Commission of the City of Orlando, Florida, and the Florida Municipal Power Agency (the licensee), which revised the Technical Specifications for operation of the St. Lucie Plant, Unit No. 2 (the facility), located in St. Lucie County, Florida. The amendment was effective as of the date of its issuance.

The amendment revised the Technical Specifications to allow operation of St. Lucie 2 at a power level of 2700 MWt.

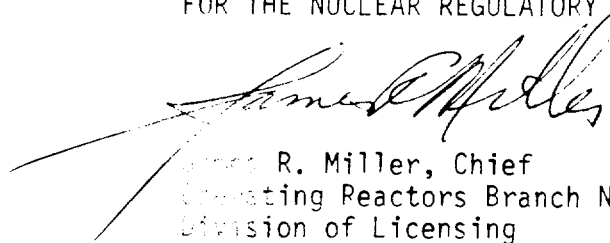
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Prior Hearing in connection with this action was published in the FEDERAL REGISTER on December 26, 1984 (49 FR 50131). No request for a hearing or petition for leave to intervene was filed following this notice.

For further details with respect to this action see (1) the application for amendment dated November 21, 1984, (2) Amendment No. 9 to Facility Operating License No. NPF-16, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Indian River Junior College Library, 3209 Virginia Avenue, Fort Pierce, Florida. A copy of items (2) and (3) will be provided upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 1st day of March, 1985.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch No. 3
Division of Licensing