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NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

JUN 17 1983

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Posted

Amdt. 3  
to NPF-16

Docket No.: 50-389

Dr. Robert E. Uhrig, Vice President  
Advanced Systems and Technology  
Florida Power & Light Company  
Post Office Box 14000  
Juno Beach, Florida 33408

Dear Dr. Uhrig:

Subject: Issuance of Amendment No. 3 to Facility Operating License NPF-16  
St. Lucie Plant, Unit 2

The U. S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 3 to Facility Operating License No. NPF-16 for the St. Lucie Plant, Unit 2 located in St. Lucie County, Florida.

This amendment is in response to your letters dated June 15, 16 and 17, 1983. The amendment authorizes use of an alternate means of monitoring the Pressure Operated Relief Valve leakage. The related safety evaluation supporting Amendment No. 3 to Facility Operating License NPF-16 will be forwarded under separate cover.

Sincerely,

George W. Knighton, Chief  
Licensing Branch No. 3  
Division of Licensing

Enclosure:  
Amendment No. 4

cc w/encls.: See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY  
ORLANDO UTILITIES COMMISSION OF THE CITY OF ORLANDO, FLORIDA  
AND  
FLORIDA MUNICIPAL POWER AGENCY  
DOCKET NO. 50-389  
ST. LUCIE PLANT, UNIT 2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 3  
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by the Florida Power and Light Company, on behalf of itself and Orlando Utilities Commission of the City of Orlando, Florida and Florida Municipal Power Agency (licensees) dated June 15, 1983 and supplemental letters dated June 16 and 17, 1983 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. NPF-16 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 3, are hereby incorporated in this license. Florida Power and Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Victor Henses*  
*for*

George W. Knighton, Chief  
Licensing Branch No. 3  
Division of Licensing

Date of Issuance: JUN 17 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 3

FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. Also to be replaced is the following overleaf page to the amended page.

Amendment Page

3/4 3-42

Overleaf Page

3/4 3-41

## INSTRUMENTATION

### ACCIDENT MONITORING INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

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3.3.3.6 The accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

- a. With the number of OPERABLE accident monitoring channels less than the Required Number of Channels shown in Table 3.3-10, either restore the inoperable channel to OPERABLE status within 7 days, or be in HOT SHUTDOWN within the next 12 hours.
- b. With the number of OPERABLE accident monitoring channels less than the Minimum Channels OPERABLE requirements of Table 3.3-10; either restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.
- c. The provisions of Specification 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.3.3.6 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

TABLE 3.3-10

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Containment Pressure	2	1
2. Reactor Coolant Outlet Temperature - T <sub>Hot</sub> (Narrow Range)	2	1
3. Reactor Coolant Inlet Temperature - T <sub>Cold</sub> (Wide Range)	2	1
4. Reactor Coolant Pressure - Wide Range	2	1
5. Pressurizer Water Level	2	1
6. Steam Generator Pressure	2/steam generator	1/steam generator
7. Steam Generator Water Level - Narrow Range	1/steam generator	1/steam generator
8. Steam Generator Water Level - Wide Range	1/steam generator*	1/steam generator*
9. Refueling Water Storage Tank Water Level	2	1
10. Auxiliary Feedwater Flow Rate (Each pump)	1/pump*	1/pump*
11. Reactor Cooling System Subcooling Margin Monitor	2	1
12. PORV Position/Flow Indicator	2/valve***	1/valve**
13. PORV Block Valve Position Indicator	1/valve**	1/valve**
14. Safety Valve Position/Flow Indicator	1/valve***	1/valve***
15. Containment Sump Water Level (Narrow Range)	1****	1****
16. Containment Water Level (Wide Range)	2	1
17. Incore Thermocouples	4/core quadrant	2/core quadrant

\*These corresponding instruments may be substituted for each other.

\*\*Not required if the PORV block valve is shut and power is removed from the operator.

\*\*\*If not available, monitor the quench tank pressure, level and temperature, and each safety valve/PORV discharge piping temperature at least once every 12 hours.

\*\*\*\*The non-safety grade containment sump water level instrument may be substituted.

CT INSTRUMENT UNIT 2

3/4 3-42

AMENDMENT NO. 3