

June 22, 1995

Mr. J. H. Goldberg
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: ST. LUCIE UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: CONTAINMENT
AIR LOCK AUTOMATIC TESTER (TAC NOS. M91697 AND M91698)

Dear Mr. Goldberg:

The Commission has issued the enclosed Amendment Nos. 137 and 77 to Facility Operating License Nos. DPR-67 and NPF-16 for the St. Lucie Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated February 22, 1995.

These amendments will eliminate reference to an automatic containment air lock tester from Technical Specification 4.6.1.3. The automatic air lock tester is no longer being used.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Jan A. Norris, Senior Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-335 and 50-389

Enclosures:

1. Amendment No. 137 to DPR-67
2. Amendment No. 77 to NPF-16
3. Safety Evaluation

cc w/enclosures: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, appearing to read "Jan A. Norris".

Jan A. Norris, Senior Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-335 and 50-389

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3. Safety Evaluation

cc w/enclosures: See next page

Mr. J. H. Goldberg
Florida Power and Light Company

St. Lucie Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 137
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated February 22, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.


2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 137, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

for 
David B. Matthews, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 22, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 137
TO FACILITY OPERATING LICENSE NO. DPR-67
DOCKET NO. 50-335

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Page

3/4 6-11

Insert Page

3/4 6-11

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

SURVEILLANCE REQUIREMENTS (continued)

1. For the personnel air lock, greater than or equal to P_a , 39.6 psig for at least 15 minutes.
 2. For the emergency air lock, greater than or equal to 10.0 psig for at least 15 minutes.
- b. By conducting overall air lock leakage tests at not less than P_a , 39.6 psig, and verifying the overall air lock leakage rate is within its limit:
1. At least once per 6 months,[#] and
 2. Prior to establishing CONTAINMENT INTEGRITY when maintenance has been performed on the air lock that could affect the air lock sealing capability.*
- c. At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

[#] The provisions of Specification 4.0.2 are not applicable.

^{*} This constitutes an exemption to Appendix J of 10 CFR 50.

CONTAINMENT SYSTEMS

INTERNAL PRESSURE

LIMITING CONDITION FOR OPERATION

3.6.1.4 Primary containment internal pressure shall be maintained between -0.7 and 2.4 PSIG.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the containment internal pressure outside of the limits above, restore the internal pressure to within the limits within 1 hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.4 The primary containment internal pressure shall be determined to be within the limits at least once per 12 hours.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER & LIGHT COMPANY
ORLANDO UTILITIES COMMISSION OF
THE CITY OF ORLANDO, FLORIDA
AND
FLORIDA MUNICIPAL POWER AGENCY
DOCKET NO. 50-389
ST. LUCIE PLANT UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated February 22, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

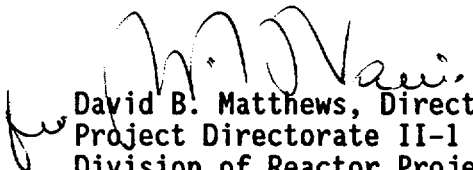
2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

2. Technical Specifications

- The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 77, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION


David B. Matthews, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 22, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 77

TO FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Page

3/4 6-10

Insert Page

3/4 6-10

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
- b. An overall air lock leakage rate of less than or equal to $0.05 L_a$ at P_a , 47.8 psig.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

- a. With one containment air lock door inoperable*:
 1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.
 2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be locked closed at least once per 31 days.
 3. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
 4. The provisions of Specification 3.0.4 are not applicable.
- b. With the containment air lock inoperable, except as the result of an inoperable air lock door, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

* If the inner air lock door is inoperable, passage through the OPERABLE outer air lock door is permitted to effect repairs to the inoperable inner air lock door. No more than one airlock door shall be open at any time.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS

- 4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:
- a. Within 72 hours following each closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying the seal leakage is $< 0.01 L_a$ as determined by precision flow measurement when the volume between the door seals is pressurized to greater than or equal to:
 1. For the personnel air lock, greater than or equal to P_a , 41.8 psig for at least 15 minutes.
 2. For the emergency air lock, greater than or equal to 41.8 psig for at least 15 minutes.
 - b. By conducting overall air lock leakage tests at not less than P_a , 41.8 psig, and verifying the overall air lock leakage rate is within its limit:
 1. At least once per 6 months,* and
 2. Prior to establishing CONTAINMENT INTEGRITY when maintenance has been performed on the air lock that could affect the air lock sealing capability.*
 - c. At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

The provisions of Specification 4.0.2 are not applicable.

* This constitutes an exemption to Appendix J of 10 CFR 50.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 137 AND 77

TO FACILITY OPERATING LICENSE NO. DPR-67 AND NO. NPF-16

FLORIDA POWER AND LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-335 AND 50-389

1.0 INTRODUCTION

By letter dated February 22, 1995, Florida Power and Light Company (FPL) requested that Appendix A of Facility Operating License DPR-67 for St. Lucie Unit 1 (PSL1) and NPF-16 for St. Lucie Unit 2 (PSL2) be revised to delete references to the automatic tester for the containment personnel air lock. FPL has discontinued using the automatic leakage measurement systems and has no plans to use them in the future. The systems are not safety related and are scheduled for removal in accordance with approved plant procedures.

2.0 DESCRIPTION OF REQUESTED CHANGES

TS 3/4.6.1.3 provides operability and surveillance requirements (SR) for the containment air locks and, in part, requires that seal leakage be determined by precision flow measurement when the volume between the door seals is pressurized to the specified pressure. Specifically, SR 4.6.1.3.a.1 states (PSL2 value in parenthesis):

"For the personnel air lock, greater than or equal to P_a , 39.6 (41.8) psig for at least 15 minutes if not tested with the automatic tester."

Requested change for SR 4.6.1.3.a.1: Delete the words, "if not tested with the automatic tester"

3.0 BACKGROUND

The licensee stated that approved plant procedures are used to perform air lock testing and include specific instructions for the use of a portable local leak rate (LLRT) cart, or the automatic leak rate tester, to check the integrity of the seals for personnel air lock inner and outer doors. Either test method is capable of demonstrating operability pursuant to SR 4.6.1.3.a.1.

Each PSL unit has an automatic tester that is designed as a dedicated system to automatically initiate a test sequence upon door closure, or by manual switch actuation. The system function is to pressurize the intra-seal volume to a preset pressure and monitor the air flow necessary to maintain that pressure. All timing functions and calibration values for measuring leakage

are user programmable. Leak test "trouble" alarms are actuated in the control room to annunciate a test failure and/or failure of the system to operate properly.

The same model automatic tester is installed at each PSL unit. The tester is not safety related, is not used to maintain air lock integrity, and the tubing installed to pressurize the intra-seal volume does not penetrate the containment atmosphere boundary.

The licensee further stated that persistent difficulties have been experienced with operation of the automatic testing systems for the containment personnel air locks for both St. Lucie Units 1 and 2. These operational difficulties include initiation of distracting "nuisance alarms" in the control rooms and, as a result, the systems are no longer used. FPL has no plans to use the automatic testers in the future, and has scheduled them for removal using approved plant configuration control procedures.

4.0 EVALUATION

TS 3/4.6.1.3 specifies Limiting Conditions for Operation (LCO) for the Containment Air Locks. Surveillance testing of air lock seals provides assurance that the overall air lock leakage will not become excessive due to seal damage during the intervals between leakage tests. The required test intervals, test pressure, and leakage acceptance criteria are not being changed by the proposed license amendment. The containment personnel air lock automatic leak rate tester is only one method of verifying seal leakage. Automatic testing is not required to demonstrate operability of the air locks, or otherwise required to comply with the specifications of the LCO.

5.0 TECHNICAL FINDING

The proposed TS revision does not alter the operability requirements for the containment air locks; rather, it eliminates the wording associated with optional use of the personnel airlock automatic leakage tester. The requirement for testing the personnel airlock at a pressure greater than or equal to P_a for at least 15 minutes remains unchanged. The acceptance criteria of personnel airlock seal leakage less than $0.01 L_a$ is also unchanged. Based on the above, the staff finds the proposed changes acceptable.

6.0 STATE CONSULTATION

Based upon the written notice of the proposed amendments, the Florida State official had no comments.

7.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (60 FR 16186). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

8.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Norris

Date: June 22, 1995