December 5, 1991

Docket Nos. 50-335
and 50-389

DISTRIBUTION
See attached sheet

Mr. J. H. Goldberg
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

Dear Mr. Goldberg:

SUBJECT: ST. LUCIE UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: ADMINISTRATIVE CHANGES (TAC NOS. M79982 AND M79983)

The Commission has issued the enclosed Amendment Nos. 112 and 53 to Facility Operating License Nos. DPR-67 and NPF-16 for the St. Lucie Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated February 26, 1991.

These amendments make administrative changes to achieve consistency throughout the Technical Specifications by removing outdated material, making minor text changes, and correcting errors.

A copy of the Safety Evaluation be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/
Jan A. Norris, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 112 to DPR-67
2. Amendment No. 53 to NPF-16
3. Safety Evaluation

NRC FILE CENTER COPY

cc w/enclosures:
See next page

OFC	:LA:PDII-2	:PM:PDII-2	:D:PDII-2	:OGC	:
NAME	:D. Mauer	:J. Norris	:H. Berkow	: Bm Borchardt	:
DATE	:11/27 91	:11/24 91	:11/27 91	:11/26/ 91	:

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Mr. J. H. Goldberg
Florida Power & Light Company

St. Lucie Plant

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Juno Beach, Florida 33408-0420

DATED: December 5, 1991

AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. DPR-67 - ST. LUCIE, UNIT 1
AMENDMENT NO. 53 TO FACILITY OPERATING LICENSE NO. NPF-16 - ST. LUCIE, UNIT 2

Docket File
NRC & Local PDRs
PDII-2 Reading
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated February 26, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 112, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 5, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 112

TO FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages	Insert Pages
3/4 3-28	3/4 3-28
3/4 3-42	3/4 3-42
3/4 4-4	3/4 4-4
3/4 8-1	3/4 8-1
3/4 8-2	3/4 8-2
3/4 8-3	3/4 8-3
3/4 8-4	3/4 8-4
3/4 8-6a	3/4 8-6a
3/4 8-6b	3/4 8-6b
3/4 10-2	3/4 10-2
6-8	6-8

TABLE 3.3-7

SEISMIC MONITORING INSTRUMENTATION

INSTRUMENT CHANNEL	SENSOR LOCATION	MEASUREMENT RANGE	MINIMUM CHANNELS OPERABLE
1. STRONG MOTION TRIAXIAL ACCELEROGRAPHS			
a. SMR-42-1	R.B. Elev. 23.0′	0-1 g	1
b. SMR-42-2	R.B. Elev. 62.0′	0-1 g	1
c. SMR-42-3	R.A.B. Elev. -0.5′	0-1 g	1
d. SMR-42-4	R.A.B. Elev. 43.0′	0-1 g	1
e. SMR-42-5	R.A.B. Elev. 19.5′	0-1 g	1
2. PEAK RECORDING ACCELEROGRAPHS			
a. SMR-42-6	R.B. Piping from S.I.T.1A2-c Elev. 46′ 10 9/16"	0-2 g	1
b. SMR-42-7	R.B. Equipment on S.I.T.1A2	0-2 g	1
c. SMR-42-8	R.A.B.-Sh. Dn. Ht. XCHR Supports	0-2 g	1
3. PEAK SHOCK RECORDERS			
a. SMR-42-9	R.B. Elev. 23.0′	-	1
b. SMR-42-10	R.B. M.S. Pipe Restraints - S.G.1B1	-	1
4. EARTHQUAKE FORCE MONITOR			
a. SMI-42-11	Control Room	0-0.2 g	1
5. SEISMIC SWITCH			
a. SMS-42-12	R.B. Elev. 23.0′	-	1

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.8 The accident monitoring instrumentation channels shown in Table 3.3-11 shall be OPERABLE

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

a. Actions per Table 3.3-11.

b. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.8 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

TABLE 3.3-11

ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	TOTAL NO. OF CHANNELS	MINIMUM CHANNELS OPERABLE	ACTION
1. Pressurizer Water Level	3	1	1
2. Auxiliary Feedwater Flow Rate	1/pump	1/pump	1
3. RCS Subcooling Margin Monitor	2	1	1
4. PORV Position Indicator Acoustic Flow Monitor	1/valve	1/valve	2
5. PORV Block Valve Position Indicator	1/valve	1/valve	2
6. Safety Valve Position Indicator	1/valve	1/valve	3
7. Incore thermocouples	4/core quadrant	2/core quadrant	1
8. Containment Sump Water Level (Narrow Range)	1*	1*	4, 5
9. Containment Sump Water Level (Wide Range)	2	1	4, 5
10. Reactor Vessel Level Monitoring System	2**	1**	4, 5
11. Containment Pressure	2	1	1

*The non-safety grade containment sump water level instrument may be substituted.

**Definition of OPERABLE: A channel is composed of eight (8) sensors in a probe, of which four (4) sensors must be OPERABLE.

PRESSURIZER

LIMITING CONDITION FOR OPERATION

3.4.4 The pressurizer shall be OPERABLE with a steam bubble, and with at least 150 kw of pressurizer heaters capable of being supplied by emergency power.

APPLICABILITY: MODES 1 and 2.

ACTION:

With the pressurizer inoperable, be in at least HOT STANDBY with the reactor trip breakers open within 6 hours.

SURVEILLANCE REQUIREMENTS

4.4.4 In accordance with 4.8.1.1.2.

3/4.8 ELECTRICAL POWER SYSTEMS

3/4.8.1 A.C. SOURCES

OPERATING

LIMITING CONDITION FOR OPERATION

3.8.1.1 As a minimum, the following A.C. electrical power sources shall be OPERABLE:

a. Two physically independent circuits between the offsite transmission network and the onsite Class 1E distribution system, and

b. Two separate and independent diesel generator sets each with:

   1. Engine-mounted fuel tanks containing a minimum of 152 gallons of fuel,

   2. A separate fuel storage system containing a minimum of 16,450 gallons of fuel, and

   3. A separate fuel transfer pump.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With one offsite circuit of 3.8.1.1.a inoperable, except as provided in Action f. below, demonstrate the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirement 4.8.1.1.1.a within 1 hour and at least once per 8 hours thereafter. If either EDG has not been successfully tested within the past 24 hours, demonstrate its OPERABILITY by performing Surveillance Requirement 4.8.1.1.2.a.4 separately for each such EDG within 24 hours. Restore the offsite circuit to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and COLD SHUTDOWN within the following 30 hours.

b. With one diesel generator of 3.8.1.1.b inoperable, demonstrate the OPERABILITY of the A.C. sources by performing Surveillance Requirement 4.8.1.1.1.a within 1 hour and at least once per 8 hours thereafter; and if the EDG became inoperable due to any cause other than preplanned preventative maintenance or testing, demonstrate the OPERABILITY of the remaining OPERABLE EDG by performing Surveillance Requirement 4.8.1.1.2.a.4 within 24 hours*; restore the diesel generator to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Additionally, verify within 2 hours or be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours that:

*This test is required to be completed regardless of when the inoperable EDG is restored to OPERABILITY.

ACTION (Continued)

1. all required systems, subsystems, trains, components and devices that depend on the remaining OPERABLE diesel generator as a source of emergency power are also OPERABLE, and

2. When in MODE 1, 2 or 3, the steam-driven auxiliary feed pump is OPERABLE.

c. With one offsite A.C. circuit and one diesel generator inoperable, demonstrate the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirement 4.8.1.1.1.a within one hour and at least once per 8 hours thereafter; and if the EDG became inoperable due to any cause other than preplanned preventative maintenance or testing, demonstrate the OPERABILITY of the remaining OPERABLE EDG by performing Surveillance Requirement 4.8.1.1.2.a.4 within 8 hours*. Restore at least one of the inoperable sources to OPERABLE status within 12 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Restore the other A.C. power source (offsite circuit or diesel generator) to OPERABLE status in accordance with the provisions of Section 3.8.1.1 ACTION Statement a or b, as appropriate, with the time requirement of that ACTION Statement based on the time of the initial loss of the remaining inoperable A.C. power source. Additionally, verify within 2 hours or be in HOT STANDBY within the next 6 hours and COLD SHUTDOWN within the following 30 hours that:

1. all required systems, subsystems, trains, components and devices that depend on the remaining OPERABLE diesel generator as a source of emergency power are also OPERABLE, and

2. When in MODE 1, 2 or 3, the steam-driven auxiliary feed pump is OPERABLE.

d. With two of the required offsite A.C. circuits inoperable, demonstrate the OPERABILITY of two diesel generators by sequentially performing Surveillance Requirement 4.8.1.1.2.a.4 on both diesels within 8 hours, unless the diesel generators are already operating; restore one of the inoperable offsite sources to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours. Following restoration of one offsite source, follow ACTION Statement a. with the time requirement of that ACTION Statement based on the time of the initial loss of the remaining inoperable offsite A.C. circuit.

*This test is required to be completed regardless of when the inoperable EDG is restored to OPERABILITY.

ACTION (Continued)

e. With two of the above required diesel generators inoperable, demonstrate the OPERABILITY of two offsite A.C. circuits by performing Surveillance Requirement 4.8.1.1.1.a within 1 hour and at least once per 8 hours thereafter; restore one of the inoperable diesel generators to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Following restoration of one diesel generator unit, follow ACTION Statement b. with the time requirement of that ACTION Statement based on the time of initial loss of the remaining inoperable diesel generator.

f. With one Unit 1 startup transformer (1A or 1B) inoperable and with a Unit 2 startup transformer (2A or 2B) connected to the same A or B offsite power circuit and administratively available to both units, then should Unit 2 require the use of the startup transformer administratively available to both units, Unit 1 shall demonstrate the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirement 4.8.1.1.1.a within 1 hour and at least once per 8 hours thereafter. If either EDG has not been successfully tested within the past 24 hours, demonstrate its OPERABILITY by performing Surveillance Requirement 4.8.1.1.2.a.4 separately for each such EDG within 24 hours. Restore the inoperable startup transformer to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.1.1.1 Each of the above required independent circuits between the offsite transmission network and the onsite Class 1E distribution system shall be:

a. Determined OPERABLE at least once per 7 days by verifying correct breaker alignments, indicated power availability; and

b. Demonstrated OPERABLE at least once per 18 months by transferring (manually and automatically) unit power supply from the auxiliary transformer to the startup transformer.

4.8.1.1.2 Each diesel generator shall be demonstrated OPERABLE:

a. In accordance with the frequency specified in Table 4.8-1 on a STAGGERED TEST BASIS by:

1. Verifying fuel level in the engine-mounted fuel tank,

2. Verifying the fuel level in the fuel storage tank,

3. Verifying the fuel transfer pump can be started and transfers fuel from the storage system to the engine-mounted tank,

SURVEILLANCE REQUIREMENTS (Continued)

4. Verifying the diesel starts from ambient condition and accelerates to approximately 900 rpm in less than or equal to 10 seconds**. The generator voltage and frequency shall be 4160 ± 420 volts and 60 ± 1.2 Hz within 10 seconds after the start signal**. The diesel generator shall be started for this test by using one of the following signals:

   a) Manual/Local

   b) Simulated loss-of-offsite power by itself.

   c) Simulated loss-of-offsite power in conjunction with an ESF actuation test signal.

   d) An ESF actuation test signal by itself.

5. Verifying the generator is synchronized, loaded to greater than or equal to 3500 kW in less than or equal to 60 seconds***, and operates within a load band of 3300 to 3500 kW*** for at least an additional 60 minutes, and

6. Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.

b. By removing accumulated water:

   1. From the engine-mounted fuel tank at least once per 31 days and after each occasion when the diesel is operated for greater than 1 hour, and

   2. From the storage tank at least once per 92 days.

**The diesel generator start (10 sec.) from ambient conditions shall be performed at least once per 184 days in these surveillance tests. All other diesel generator starts for the purposes of this surveillance testing may be preceded by an engine prelube period and may also include warmup procedures (e.g., gradual acceleration) as recommended by the manufacturer so that mechanical stress and wear on the diesel generator is minimized.

***Generator loading in less than or equal to 60 seconds shall be performed at least once every 184 days; timing for this loading test shall start upon the closing the diesel generator breaker. All other loading for the purpose of this surveillance test may be performed according to manufacturer's recommendations. The indicated load band is meant as guidance to avoid routine overloading. Variations in loads in excess of the band due to changing bus loads shall not invalidate this test.

SURVEILLANCE REQUIREMENTS (Continued)

6. Verifying the diesel generator operates for at least 24 hours****. During the first 2 hours of this test, the diesel generator shall be loaded within a load band of 3800 to 3960 kW# and during the remaining 22 hours of this test, the diesel generator shall be loaded within a load band of 3300 to 3500 kW#. The generator voltage and frequency shall be 4160 ± 420 volts and 60 ± 1.2 Hz within 10 seconds after the start signal; the steady state generator voltage and frequency shall be maintained within these limits during this test.

7. Verifying that the auto-connected loads do not exceed the 2000-hour rating of 3730 kW.

8. Verifying the diesel generator's capability to:

   a) Synchronize with the offsite power source while the generator is loaded with its emergency loads upon a simulated restoration of offsite power.

   b) Transfer its loads to the offsite power source, and

   c) Be restored to its standby status.

9. Verifying that with the diesel generator operating in a test mode (connected to its bus), a simulated safety injection signal overrides the test mode by (1) returning the diesel generator to standby operation and (2) automatically energizes the emergency loads with offsite power.

10. Verifying that the fuel transfer pump transfers fuel from each fuel storage tank to the engine-mounted tanks of each diesel via the installed cross connection lines.

11. Verifying that the automatic load sequence timers are operable with the interval between each load block within ± 1 second of its design interval.

f. At least once per ten years or after any modification which could affect diesel generator independence by starting**** the diesel generators simultaneously, during shutdown, and verifying that the diesel generators accelerate to approximately 900 rpm in less than or equal to 10 seconds.

#This band is meant as guidance to avoid routine overloading of the engine. Variations in load in excess of this band due to changing bus loads shall not invalidate this test.

****This test may be conducted in accordance with the manufacturer's recommendations concerning engine prelube period.

SURVEILLANCE REQUIREMENTS (Continued)

g. At least once per ten years by:

1. Draining each fuel storage tank, removing the accummulated sediment and cleaning the tank using an appropriate cleaning compound, and

2. Performing a pressure test of those portions of the diesel fuel oil system designed to USAS B31.7 Class 3 requirements at a test pressure equal to 110% of the system design pressure.

4.8.1.1.3 Reports - All diesel generator failures, valid or non-valid, shall be reported to the Commission pursuant to Specification 6.9.2. Reports of diesel generator failures shall include the information recommended in Regulatory Position C.3.b of Regulatory Guide 1.108, Revision 1, August 1977. If the number of failures in the last 100 valid tests (on a per nuclear unit basis) is greater than or equal to 7, the report shall be supplemented to include the additional information recommended in Regulatory Position C.3.b of Regulatory Guide 1.108, Revision 1, August 1977.

4.8.1.1.4 The Class 1E underground cable system shall be demonstrated OPERABLE within 30 days after the movement of any loads in excess of 80% of the ground surface design basis load over the cable ducts by pulling a mandrel with a diameter of at least 80% of the duct's inside diameter through a duct exposed to the maximum loading (duct nearest the ground's surface) and verifying that the duct has not been damaged.

3/4.10 SPECIAL TEST EXCEPTIONS

SHUTDOWN MARGIN

LIMITING CONDITION FOR OPERATION

3.10.1 The SHUTDOWN MARGIN requirement of Specification 3.1.1.1 may be suspended for measurement of CEA worth and shutdown margin provided reactivity equivalent to at least the highest estimated CEA worth is available for trip insertion from OPERABLE CEA(s).

APPLICABILITY: MODE 2.

ACTION:

a. With any full length CEA not fully inserted and with less than the above reactivity equivalent available for trip insertion, immediately initiate and continue boration at $\geq$ 40 gpm of 1720 ppm boric acid solution or its equivalent until the SHUTDOWN MARGIN required by Specification 3.1.1.1 is restored.

b. With all full length CEAs inserted and the reactor subcritical by less than the above reactivity equivalent, immediately initiate and continue boration at $\geq$ 40 gpm of 1720 ppm boric acid solution or its equivalent until the SHUTDOWN MARGIN required by Specification 3.1.1.1 is restored.

SURVEILLANCE REQUIREMENTS

4.10.1.1 The position of each full length CEA required either partially or fully withdrawn shall be determined at least once per 2 hours.

4.10.1.2 Each CEA not fully inserted shall be demonstrated capable of full insertion when tripped from at least the 50% withdrawn position within 7 days prior to reducing the SHUTDOWN MARGIN to less than the limits of Specification 3.1.1.1.

GROUP HEIGHT, INSERTION AND POWER DISTRIBUTION LIMITS

LIMITING CONDITION FOR OPERATION

3.10.2 The group height, insertion and power distribution limits of Specifications 3.1.1.4, 3.1.3.1, 3.1.3.2, 3.1.3.5, 3.1.3.6, 3.2.3, and 3.2.4 may be suspended during the performance of PHYSICS TESTS provided:

a. The THERMAL POWER is restricted to the test power plateau which shall not exceed 85% of RATED THERMAL POWER, and

b. The limits of Specification 3.2.1 are maintained and determined as specified in Specification 4.10.2.2 below.

APPLICABILITY: MODES 1 and 2.

ACTION:

With any of the limits of Specification 3.2.1 being exceeded while the requirements of Specifications 3.1.1.4, 3.1.3.1, 3.1.3.2, 3.1.3.5, 3.1.3.6, 3.2.3 and 3.2.4 are suspended, either:

a. Reduce THERMAL POWER sufficiently to satisfy the requirements of Specification 3.2.1, or

b. Be in HOT STANDBY within 6 hours.

SURVEILLANCE REQUIREMENTS

4.10.2.1 The THERMAL POWER shall be determined at least once per hour during PHYSICS TESTS in which the requirements of Specifications 3.1.1.4, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.2.3 or 3.2.4 are suspended and shall be verified to be within the test power plateau.

4.10.2.2 The linear heat rate shall be determined to be within the limits of Specification 3.2.1 by monitoring it continuously with the Incore Detector Monitoring System pursuant to the requirements of Specifications 4.2.1.3 and 3.3.3.2 during PHYSICS TESTS above 5% of RATED THERMAL POWER in which the requirements of Specifications 3.1.1.4, 3.1.3.1, 3.1.3.5, 3.1.3.6, 3.2.3 or 3.2.4 are suspended.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the FRG Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in FRG activities at any one time.

MEETING FREQUENCY

6.5.1.4 The FRG shall meet at least once per calendar month and as convened by the FRG Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the FRG necessary for the performance of the FRG responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Facility Review Group shall be responsible for:

a. Review of (1) all procedures required by Specification 6.8 and changes thereto, (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.

b. Review of all proposed tests and experiments that affect nuclear safety.

c. Review of all proposed changes to Appendix A Technical Specifications.

d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.

e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the President - Nuclear Division and to the Chairman of the Company Nuclear Review Board.

f. Review of all REPORTABLE EVENTS.

g. Review of unit operations to detect potential nuclear safety hazards.

h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant Manager or the Company Nuclear Review Board.

i. Review of the Security Plan and implementing procedures and submittal of recommended changes to the Company Nuclear Review Board.

j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Company Nuclear Review Board.

k. Review of every unplanned on-site release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the President - Nuclear Division and to the Company Nuclear Review Board.

l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL and RADWASTE TREATMENT SYSTEMS.

m. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last FRG meeting.

AUTHORITY

6.5.1.7 The Facility Review Group shall:

a. Recommend in writing to the Plant Manager, approval or disapproval of items considered under Specifications 6.5.1.6.a through d above.

b. Render determinations in writing with regard to whether or not each item considered under Specifications 6.5.1.6 a, b, d, and e above constitutes an unreviewed safety question.

c. Provide written notification within 24 hours to the President-Nuclear Division and the Company Nuclear Review Board of of disagreement between the FRG and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.1.8 The Facility Review Group shall maintain written minutes of each FRG meeting that, at a minimum, document the results of all FRG activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the President-Nuclear Division and the Chairman of the Company Nuclear Review Board.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

ORLANDO UTILITIES COMMISSION OF

THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 53
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated February 26, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

   2. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 53, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 5, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 53

TO FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages	Insert Pages
XXI	XXI
XXII	XXII
XXIV	XXIV
XXV	XXV
3/4 6-21	3/4 6-21
3/4 8-4	3/4 8-4
3/4 8-18	3/4 8-18
3/4 8-19	3/4 8-19
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CONTAINMENT ISOLATION VALVES

Valve Tag Number	Penetration Number	Function	Testable During Plant Operation	Maximum Isolation Time (Sec)
A) Containment Isolation				
I-HCV-15-1	7	Primary Makeup Water (CIS)	Yes	5
I-HCV-18-2	8	Station Air Supply	Yes	5
I-HCV-18-1	9	Instrument Air Supply (CIS)	No	5
I-FCV-25-5,4	10	Containment Purge Exhaust (CIS)	No	3
I-FCV-25-2,3	11	Containment Purge Makeup (CIS)	No	3
V-6741	14	Nitrogen Supply to Safety Injection Tanks (CIS)	Yes	5
I-HCV-14-7 I-HCV-14-1	23	Reactor Coolant Pump Cooling Water Supply (SIAS)	No	5
I-HCV-14-6 I-HCV-14-2	24	Reactor Coolant Pump Cooling Water Return (SIAS)	No	5
I-V-2516 I-V-2522	26	Letdown Line (CIS)	No	5
I-SE-05-1A,1B, 1C,1D,1E	28A	Safety Injection Tank Sample	Yes	5
I-V-5200 I-V-5203	28B	Reactor Coolant System Hot Leg Sample (CIS)	Yes	5
I-V-5204 I-V-5201	29A	Pressurizer Surge Sample (CIS)	Yes	5
I-V-5205 I-V-5202	29B	Pressurizer Steam Sample (CIS)	Yes	5

TABLE 3.6-2 (Continued)

CONTAINMENT ISOLATION VALVES

Valve Tag Number	Penetration Number	Function	Testable During Plant Operation	Maximum Isolation Time (Sec)
I-V-6718 I-V-6750	31	Containment Vent Header (CIS)	Yes	5
I-SE-03-2A,2B	41	Safety Injection Tank Test Line (CIS/SIAS)	Yes	5
I-LCV-07-11A I-LCV-07-11B	42	Reactor Cavity Sump Pump Discharge (CIS/SIAS)	Yes	5
I-V-6341 I-V-6342	43	RCDT Pump Suction (CIS)	Yes	5
I-V-2524 I-V-2505	44	RCP Controlled Bleed-off (CIS)	No	5
I-FCV-26-1 I-FCV-26-2	52A	Containment Radiation Monitoring (CIS)	Yes	10
I-FCV-26-3 I-FCV-26-4	52B	Containment Radiation Monitoring (CIS)	Yes	10
I-FCV-26-5 I-FCV-26-6	52C	Containment Radiation Monitoring (CIS)	Yes	10
I-FCV-25-26 I-FCV-25-36	56	Cont. Containment/H_2 Purge Makeup Inlet (CIS)	Yes	3
I-FCV-25-20 I-FCV-25-21	57	Cont. Containment/H_2 Purge Exhaust (CIS)	Yes	3

ACTION (Continued)

d. With two of the required offsite A.C. circuits inoperable, demonstrate the OPERABILITY of two diesel generators by sequentially performing Surveillance Requirement 4.8.1.1.2a.4 on both diesels within 8 hours, unless the diesel generators are already operating; restore one of the inoperable offsite sources to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours. Following restoration of one offsite source, follow ACTION Statement a. with the time requirement of that ACTION Statement based on the time of initial loss of the remaining inoperable offsite A.C. circuit.

e. With two of the above required diesel generators inoperable, demonstrate the OPERABILITY of two offsite A.C. circuits by performing Surveillance Requirement 4.8.1.1.1.a within 1 hour and at least once per 8 hours thereafter; restore one of the inoperable diesel generators to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Following restoration of one diesel generator unit, follow ACTION Statement b. with the time requirement of that ACTION Statement based on the time of initial loss of the remaining inoperable diesel generator.

f. With one Unit 2 startup transformer (2A or 2B) inoperable and with a Unit 1 startup transformer (1A or 1B) connected to the same A or B offsite power circuit and administratively available to both units, then should Unit 1 require the use of the startup transformer administratively available to both units, Unit 2 shall demonstrate the operability of the remaining A.C. sources by performing Surveillance Requirement 4.8.1.1.1a. within 1 hour and at least once per 8 hours thereafter. If either EDG has not been successfully tested within the past 24 hours, demonstrate its OPERABILITY by performing Surveillance Requirement 4.8.1.1.2a.4 separately for each such EDG within 24 hours. Restore the inoperable startup transformer to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.1.1.1 Each of the above required independent circuits between the offsite transmission network and the onsite Class 1E distribution system shall be:

a. Determined OPERABLE at least once per 7 days by verifying correct breaker alignments, indicated power availability; and

b. Demonstrated OPERABLE at least once per 18 months by transferring (manually and automatically) unit power supply from the normal circuit to the alternate circuit.

4.8.1.1.2 Each diesel generator shall be demonstrated OPERABLE:

a. In accordance with the frequency specified in Table 4.8-1 on a STAGGERED TEST BASIS BY:

SURVEILLANCE REQUIREMENTS (Continued)

1. Verifying the fuel level in the engine-mounted fuel tank,

2. Verifying the fuel level in the fuel storage tank,

3. Verifying the fuel transfer pump can be started and transfers fuel from the storage system to the engine-mounted tank,

4. Verifying the diesel starts form ambient condition and accelerates to approximately 900 rpm in less than or equal to 10 seconds**. The generator voltage and frequency shall be 4160 ± 420 volts and 60 ± 1.2 Hz within 10 seconds after the start signal**. The diesel generator shall be started for this test by using one of the following signals:

   a) Manual/Local.

   b) Simulated loss-of-offsite power by itself.

   c) Simulated loss-of-offsite power in conjunction with an ESF actuation test signal.

   d) An ESF actuation test signal by itself.

5. Verifying the generator is synchronized, loaded*** to greater than or equal to 3685 kW in less than or equal to 60 seconds, and operates within a load band of 3450 to 3685 kW at least an additional 60 minutes, and

6. Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.

b. By removing accumulated water:

   1. From the engine-mounted fuel tank at least once per 31 days and after each occasion when the diesel is operated for greater than 1 hour, and

   2. From the storage tank at least once per 31 days.

** The diesel generator start (10 sec) from ambient conditions shall be performed at least once per 184 days in these surveillance tests. All other diesel generator starts for purposes of this surveillance testing may be preceded by an engine prelube period and may also include warmup procedures (e.g., gradual acceleration) as recommended by the manufacturer so that mechanical stress and wear on the diesel generator is minimized.

*** Generator loading in less than or equal to 60 seconds shall be performed at least once every 184 days; timing for this loading test shall start upon the closing of the diesel generator breaker. All other loading for the purpose of this surveillance test may be performed according to manufacturer's recommendations. The indicated load band is meant as guidance to avoid routine overloading. Variations in loads in excess of the band due to changing bus loads shall not invalidate this test.

3/4.8.4 ELECTRICAL EQUIPMENT PROTECTIVE DEVICES

MOTOR-OPERATED VALVES THERMAL OVERLOAD PROTECTION BYPASS DEVICES

LIMITING CONDITION FOR OPERATION

3.8.4 The thermal overload protection bypass devices, integral with the motor starter, of each valve listed in Table 3.8-1 shall be OPERABLE.

APPLICABILITY: Whenever the motor-operated valve is required to be OPERABLE.

ACTION:

With one or more of the thermal overload protection bypass devices inoperable, declare the affected valve(s) inoperable and apply the appropriate ACTION Statement(s) for the affected valve(s).

SURVEILLANCE REQUIREMENTS

4.8.4 The above required thermal overload protection bypass devices shall be demonstrated OPERABLE.

a. At least once per 18 months, by visually verifying the bypass switch to be in the bypass position for those thermal overload devices which are either:

   1. Continuously bypassed and temporarily placed in force only when the valve motors are undergoing periodic or maintenance testing, or

   2. Normally in force during plant operation and bypassed under accident conditions.

b. At least once per 18 months by the performance of a CHANNEL CALIBRATION of a representative sample of at least 25% of:

   1. All thermal overload devices which are not bypassed, such that each non-bypassed device is calibrated at least once per 6 years.

   2. All thermal overload devices which are continuously bypassed and temporarily placed in force only when the valve motors are undergoing periodic or maintenance testing, and thermal overload devices normally in force and bypassed under accident conditions such that each thermal overload is calibrated and each valve is cycled through at least one complete cycle of full travel with the motor-operator when the thermal overload is OPERABLE and not bypassed, at least once per 6 years.

TABLE 3.8-1

MOTOR-OPERATED VALVES THERMAL OVERLOAD PROTECTION BYPASS DEVICES

VALVE NUMBER	FUNCTION	BYPASS (YES/NO)
RCS: V-1476	LTOP ISOLATION	YES
V-1477	LTOP ISOLATION	YES
CVCS: V-2508	BAMT ISOL.	YES
V-2509	BAMT ISOL.	YES
V-2514	BAMP DISCH.	YES
V-2525	PMW SUPPLY	YES
V-2553	CHARGING PUMP BYPASS	YES
V-2554	CHARGING PUMP BYPASS	YES
V-2555	CHARGING PUMP BYPASS	YES
V-2501	VCT ISOL.	YES
V-2504	RWT ISOL.	YES
SIS: FCV-3301	SHUTDOWN COOLING	YES
FCV-3306	SHUTDOWN COOLING	YES
HCV-3512	SHUTDOWN COOLING	YES
HCV-3657	SHUTDOWN COOLING	YES
V-3456	SDC ISOL.	YES
V-3457	SDC ISOL.	YES
V-3517	SDC ISOL.	YES
V-3658	SDC ISOL.	YES
V-3540	HOT LEG INJECTION	YES
V-3550	HOT LEG INJECTION	YES
V-3523	HOT LEG INJECTION	YES
V-3551	HOT LEG INJECTION	YES
V-3656,3654	HPSI ISOL.	YES
V-3659	SI RECIRCULATION	YES
V-3660	SI RECIRCULATION	YES
V-3615,25,35,45	LPSI INJ.	YES
V-3616,26,36,46	HPSI INJ.	YES
V-3617,27,37,47	HPSI INJ.	YES
V-3480	SDC ISOL.	YES
V-3481	SDC ISOL.	YES
V-3651	SDC ISOL.	YES
V-3652	SDC ISOL.	YES
V-3545	SDC X-TIE	YES
V-3664	SDC ISOL.	YES
V-3665	SDC ISOL.	YES
V-3536	SDC WARMUP	YES
V-3539	SDC WARMUP	YES
V-3614,24,34,44	SIT ISOL.	YES
V-3432	RWT ISOL.	YES
V-3444	RWT ISOL.	YES

TABLE 3.8.1 (Continued)

MOTOR-OPERATED VALVES THERMAL OVERLOAD PROTECTION BYPASS DEVICES

VALVE NUMBER		FUNCTION	BYPASS (YES/NO)
MAIN STEAM:			
	MV-08-1A,1B	MSIV BYPASS	YES
	MV-08-18A,18B	A.D.V.	YES
	MV-08-19A,19B	A.D.V.	YES
	MV-08-12,13	AFW TURBINE INLET	YES
	MV-08-3	AFW TURBINE INLET	YES
	MV-08-14,15,16,17	A.D.V. ISOL.	YES
MAIN FEEDWATER:			
	MV-09-9,10,11,12	AUX. FEED ISOL.	YES
	MV-09-13,14	AUX. FEED X-TIE	YES
ICW:	MV-21-2,3	ICW ISOL.	YES
	MV-21-4A,4B	ICW ISOL.	YES
CCW:	MV-14-17,18,19,20	FUEL POOL ISOL.	YES
	MV-14-9,10,11,12,13,14,15, 16	CONT. FAN ISOL.	YES
	MV-14-1,2,3,4	CCW PUMP ISOL.	YES
C.S.:	MV-07-1A,1B	RWT ISOL.	YES
	MV-07-2A,2B	SUMP ISOL.	YES
	MV-07-3,4	SYSTEM ISOL.	YES
HVAC:	FCV-25-14,15,16,17,18,19	CRECS ISOL.	YES
	FCV-25-24,25	CRECS ISOL.	YES
	FCV-25-11,12	SBVS ISOL.	YES
	FCV-25-35	VENT ISOL.	YES
	FCV-25-29,34	H2 CONT. PURGE	YES
	FCV-25-30,31	SFP EXHAUST	YES
	FCV-25-32,33	SBVS INLET	YES

RESPONSIBILITIES (Continued)

k. Review of every unplanned onsite release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the President - Nuclear Division and to the Company Nuclear Review Board.

l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL and RADWASTE TREATMENT SYSTEMS.

m. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last FRG meeting.

n. Review of the Fire Protection Program and implementing procedures and submittal of recommended changes to the Company Nuclear Review Board.

AUTHORITY

6.5.1.7 The Facility Review Group shall:

a. Recommend in writing to the Plant Manager approval or disapproval of items considered under Specifications 6.5.1.6a. through d. and m. above.

b. Render determinations in writing with regard to whether or not each item considered under Specifications 6.5.1.6a, b, d and e above constitutes an unreviewed safety question.

c. Provide written notification within 24 hours to the President-Nuclear Division and the Company Nuclear Review Board of disagreement between the FRG and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.1.8 The Facility Review Group shall maintain written minutes of each FRG meeting that, at a minimum, document the results of all FRG activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the President - Nuclear Division and the Chairman of the Company Nuclear Review Board.

6.5.2 COMPANY NUCLEAR REVIEW BOARD (CNRB)

FUNCTION

6.5.2.1 The Company Nuclear Review Board shall function to provide independent review and audit of designated activities in the areas of:

a. nuclear power plant operations
b. nuclear engineering
c. chemistry and radiochemistry
d. metallurgy

FUNCTION (Continued)

e. instrumentation and control

f. radiological safety

g. mechanical and electrical engineering

h. quality assurance practices

COMPOSITION

6.5.2.2 The President - Nuclear Division shall appoint, in writing, a minimum of five members to the CNRB and shall designate from this membership, in writing, a Chairman. The membership shall function to provide independent review and audit in the areas listed in Specification 6.5.2.1. The Chairman shall meet the requirements of ANSI/ANS-3.1-1987, Section 4.7.1. The members of the CNRB shall meet the educational requirements of the ANSI/ANS-3.1-1987, Section 4.7.2, and have at least 5 years of professional level experience in one or more of the fields listed in Specification 6.5.2.1. CNRB members who do not possess the educational requirements of ANSI/ANS-3.1-1987, Section 4.7.2 (up to a maximum of 2 members) shall be evaluated, and have their membership approved and documented, in writing, on a case-by-case basis by the President - Nuclear Division, considering the alternatives to educational requirements of ANSI/ANS-3.1-1987, Sections 4.1.1 and 4.1.2.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the CNRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in CNRB activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the CNRB Chairman to provide expert advice to the CNRB.

MEETING FREQUENCY

6.5.2.5 The CNRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter and as convened by the CNRB Chairman or his designated alternate.

QUORUM

6.5.2.6 The quorum of the CNRB necessary for the performance of the CNRB review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least a majority of CNRB members including alternates. No more than a minority of the quorom shall have line responsibility for operation of the unit.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 112 AND 53

TO FACILITY OPERATING LICENSE NOS. DPR-67 AND NPF-16

FLORIDA POWER & LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-335 AND 50-389

1.0 INTRODUCTION

By application dated February 26, 1991, Florida Power & Light Company (FPL, the licensee) requested various changes to the Technical Specifications (TS) for the St. Lucie Plant, Unit Nos. 1 and 2. The licensee proposed to make administrative changes to achieve consistency throughout the TS by removing outdated material, making minor text changes and correcting errors.

2.0 EVALUATION

St. Lucie Unit 1 TS only

a) On page 3/4 3-28, Table 3.3-7, the licensee proposed to add strong motion triaxial accelograph SMR-42-5. This instrument was always part of the plant and is not the result of a plant change or modification. Since this change corrects an erroneous omission, the staff finds it acceptable.

b) On page 3/4 3-42, Table 3.3-11, the licensee proposed to change the number of reactor coolant system (RCS) subcooling margin monitors from 1 to 2. This change corrects an erroneous omission made when Amendment No. 79 revised Table 3.3-11. With this correction, Table 3.3-11 will now accurately reflect the current instrumentation in the plant. The staff finds this correction acceptable.

c) On page 3/4 4-4, Surveillance Requirement 4.4.4, the licensee proposed to correct the reference by changing "4.8.1.1" to "4.8.1.1.2." This is not a change but a correction to cite the proper surveillance requirement. For that reason the staff finds this correction acceptable.

d) The licensee proposed the following minor changes in text:

1) On pages 3/4 8-1 and 8-3, capitalize the word "OPERABILITY."
2) On page 3/4 8-2, capitalize the "s" in the word "Statement."
3) On page 3/4 8-6a, add the word "with" to the sentence.
4) On page 3/4 8-6b, correct spelling of "Specification" and "information."

The above changes correct grammar, spelling and standardize capitalization in the TS. Therefore, the staff finds the proposed changes acceptable.

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e) On page 3/4 10-2, the licensee proposed to delete the reference to specification 3.1.3.2 from specifications 4.10.2.1 and 4.10.2.2. The above deletion should have been made when Amendment No. 27, issued May 26, 1978, deleted specification 3.1.3.2. Since the proposed deletion corrects an error of omission, the staff finds the deletion acceptable.

St. Lucie Unit 2 TS only

a) Over the years, a number of figures and tables in the TS have been changed by amendments without making corresponding changes on the appropriate pages of the TS index. The licensee proposed to update the various index pages as follows:

1. On page XXI, Figures 3.1-1, 4.2-1 and 3.4-2 are corrected in accordance with Amendment Nos. 40, 17 and 46, respectively.

2. On page XXII, Figures 3.4-3, 3.4-4, and B 3/4.4-1 are corrected in accordance with Amendment Nos. 46, 46 and 31, respectively. Figure 5.6-1 is added in accordance with Amendment No. 7.

3. On page XXIV, Table 4.4-5 is corrected in accordance with Amendment No. 31. Tables 3.4-3 and 3.4-4 are added in accordance with Amendment No. 31.

4. On page XXV, Tables 3.7-3a and 3.7-3b are deleted in accordance with Amendment No. 22.

The staff has reviewed all of the above proposals and finds that the index pages should be updated. Therefore, the staff finds the proposed updates acceptable.

b) On page 3/4 6-21, the licensee proposed to delete "HC" from valve identification on valves 2516 and 2522 to read I-V-2516 and I-V-2522. This change corrects the TS to reflect the actual valve identification as it exists in the plant, and for that reason the staff finds the proposed changes acceptable.

c) The licensee proposed to clarify and make the format of listing multiple valves consistent by the following changes:

1. On page 3/4 8-18, replace:
   "-3645" with ",25,35,45"
   "-3646" with ",26,36,46"
   "-3647" with ",27,37,47"
   "-3644" with ",24,34,44"

2. On page 3/4 8-19, replace:
   "TO 16" with ",10,11,12,13,14,15,16"
   "TO 19" with ",15,16,17,18,19"

The staff has reviewed all of the above proposals and agrees that they are clarifying in nature. For these reasons, the staff finds them acceptable.

St. Lucie Units 1 and 2

a) On pages 3/4 8-4, the licensee proposed to add "Local" to the diesel generator start option for the diesel generator surveillance requirements. Amendments No. 103 for Unit 1, and No. 39 for Unit 2 allowed warmup and slow acceleration of the diesel generator for certain surveillance requirements. In accordance with the guidance contained in CEN 152, "Emergency Procedure Guidelines," Revision 3, the term "manual" means to operate equipment from a location other than its physical location, normally from the control room. The term "local" means to operate equipment at its physical location. A warmup and slow acceleration of the diesel generator can only be accomplished by starting the diesel locally. It is, therefore, necessary to add the "Local" option to the TS to avoid confusing and inconsistent instructions for the operators. The staff finds the proposed addition acceptable.

b) On page 6-8 for Unit 1 and page 6-9 for Unit 2, in specification 6.5.1.7b, the licensee proposed to replace "a through e" with "a,b,d, and e." In essence, this change deletes item "c" from specification 6.5.1.7b. Item "c" reads: "Review of all proposed changes to Appendix A Technical Specifications."

   Specification 6.5.1.7 delineates the authority of the Facility Review Group, and item b currently reads: "Render determinations in writing with regard to whether or not each item considered under Specifications 6.5.1.6a through e. above constitutes an unreviewed safety question." The intent of this specification is to determine whether the particular changes to the procedures, tests, experiments etc., listed in specifications 6.5.1.6a through e need to be submitted to NRC for prior approval. Since 10 CFR 50.59 already requires all changes to Appendix A Technical Specifications be submitted to NRC for prior approval, item "c" is not necessary. Therefore, the staff finds this deletion acceptable.

3.0 STATE CONSULTATION

Based upon the written notice of the proposed amendments, the Florida State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in

individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (56 FR 15641). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). These amendments also relate to changes in administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Jan A. Norris

Date: December 5, 1991

December 5, 1991

Docket Nos. 50-335
and 50-389

DISTRIBUTION
See attached sheet

Mr. J. H. Goldberg
President-Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

Dear Mr. Goldberg:

SUBJECT: ST. LUCIE UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: MAIN STEAM LINE AND MAIN FEEDWATER ISOLATION VALVES (TAC NOS. M74113 AND M74114)

The Commission has issued the enclosed Amendment Nos. 111 and 52 to Facility Operating License Nos. DPR-67 and NPF-16 for the St. Lucie Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated July 26, 1989.

These amendments revise Technical Specifications Section 3.7 for both units to clarify testing requirements for the main steam line isolation valves and for the main feedwater line isolation valves.

A copy of the Safety Evaluation be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/
Jan A. Norris, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 111 to DPR-67
2. Amendment No. 52 to NPF-16
3. Safety Evaluation

cc w/enclosures:
See next page

OFC	:LA:PDII-2	:PM:PDII-2	:SPLB	:D:PDII-2	: OGC
NAME	:D. Miller	:J. Norris	:C. McCracken	:H. Berkow	:
DATE	:11/13 91	:11/14 91	:11/15 91	:11/18 91	:11/27

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Mr. J. H. Goldberg
Florida Power & Light Company

St. Lucie Plant

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DATED: December 5, 1991

AMENDMENT NO. 111 TO FACILITY OPERATING LICENSE NO. DPR-67 - ST. LUCIE, UNIT 1
AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. NPF-16 - ST. LUCIE, UNIT 2

Docket File
NRC & Local PDRs
PDII-2 Reading
S. Varga, 14/E/4
G. Lainas, 14/H/3
H. Berkow
D. Miller
J. Norris
OGC-WF
D. Hagan, 3302 MNBB
G. Hill (8), P-137
Wanda Jones, MNBB-7103
C. Grimes, 11/F/23
C. McCracken
ACRS (10)
GPA/PA
OC/LFMB
PD Plant-specific file [Gray File]
M. Sinkule, R-II
Others as required

cc: Plant Service list

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 111
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated July 26, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 111, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 5, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 111

TO FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove Page	Insert Page
3/4 7-9	3/4 7-9

MAIN STEAM LINE ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.7.1.5 Each main steam line isolation valve shall be OPERABLE.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

MODE 1 - With one main steam line isolation valve inoperable, POWER OPERATION may continue provided the inoperable valve is either restored to OPERABLE status or closed within 4 hours; otherwise, be in HOT STANDBY within the next 6 hours.

MODES 2 and 3 - With one or both main steam isolation valve(s) inoperable, subsequent operation in MODES 2 or 3 may proceed provided the isolation valve(s) is (are) maintained closed. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 24 hours.

The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.1.5 Each main steam line isolation valve that is open shall be demonstrated OPERABLE by verifying full closure within 6.0 seconds when tested pursuant to Specification 4.0.5.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

ORLANDO UTILITIES COMMISSION OF

THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated July 26, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

   2. Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 52 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 5, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 52

TO FACILITY OPERATING LICENSE NO. NPF-16

DOCKET NO. 50-389

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages	Insert Pages
3/4 7-9	3/4 7-9
3/4 7-10	3/4 7-10

MAIN STEAM LINE ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.7.1.5 Each main steam line isolation valve shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

MODE 1	-	With one main steam line isolation valve inoperable but open, POWER OPERATION may continue provided the inoperable valve is restored to OPERABLE status within 4 hours; otherwise, be in at least HOT STANDBY within the next 6 hours.
MODES 2, 3 and 4	-	With one or both main steam line isolation valve(s) inoperable, subsequent operation in MODES 2, 3 or 4 may proceed provided the isolation valve(s) is (are) maintained closed. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 24 hours. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.1.5 Each main steam line isolation valve shall be demonstrated OPERABLE by verifying full closure within 6.75 seconds when tested pursuant to Specification 4.0.5.

MAIN FEEDWATER LINE ISOLATION VALVES

LIMITING CONDITION FOR OPERATION

3.7.1.6 Each main feedwater line isolation valve shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

MODE 1 -	With one main feedwater line isolation valve inoperable but open, POWER OPERATION may continue provided the inoperable valve is restored to OPERABLE status within 4 hours; otherwise, be in at least HOT STANDBY within the next 6 hours.
MODES 2, 3 - and 4	With one or both main feedwater line isolation valve(s) inoperable, subsequent operation in MODE 2, 3, or 4 may proceed provided the isolation valve(s) is (are) maintained closed. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 24 hours. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.1.6 Each main feedwater line isolation valve shall be demonstrated OPERABLE by verifying full closure within 5.15 seconds when tested pursuant to Specification 4.0.5

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 111 AND 52

TO FACILITY OPERATING LICENSE NO. DPR-67 AND NO. NPF-16

FLORIDA POWER AND LIGHT COMPANY, ET AL.

ST. LUCIE PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-335 AND 50-389

1.0 INTRODUCTION

By letter dated July 26, 1989, Florida Power and Light Company (FPL, the licensee) proposed to amend Facility Operating Licenses DPR-67 and NPF-16 for St. Lucie Units 1 and 2, respectively. These amendments would revise Technical Specifications Section 3.7 for both units to clarify testing requirements for the main feedwater line isolation valves and the main steam line isolation valves. The NRC staff proposed minor clarifying changes to the revised Technical Specification pages. These changes were discussed with and agreed to by the licensee's staff.

2.0 EVALUATION

2.1 Main Steam Line Isolation Valves - Units 1 and 2

The operability of the main steam isolation valves (MSIVs) ensures that no more than one steam generator will blow down in the event of a steam line rupture. This restriction is required to minimize the positive reactivity effects of the reactor coolant system cooldown associated with blowdown, and to limit the pressure rise within containment in the event the steam line rupture occurs within containment.

Present Specification 3.7.1.5 could be interpreted to require surveillance testing prior to entering and/or changing modes. The proposed changes will clarify requirements for the plant to achieve the required modes to conduct testing necessary to meet the limiting condition for operation. The corresponding action statements will also be clarified to indicate that the MSIVs perform their required safety function when closed.

The licensee proposed to modify Specification 3.7.1.5 for Units 1 and 2 as follows:

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a. In the action statement applicable to mode 1 for Unit 1, replace ". . . be in HOT SHUTDOWN within the next 12 hours" with ". . . be in HOT STANDBY within the next 6 hours." This change renders the action statement more conservative in that it is preferable to be in mode 3 within 6 hours than in mode 4 within 12 hours. Also, this change makes this action statement the same as for Unit 2. In the action statement applicable in mode 1 for Unit 2, delete "and in COLD SHUTDOWN within the following 24 hours." This deletion eliminates a confusing statement, because once in HOT STANDBY the action statement for Modes 2, 3 and 4 is operative, and that already requires going down to COLD SHUTDOWN within the following 24 hours. For these reasons, the staff finds the proposed changes acceptable.

b. In the action statement applicable to modes 2 and 3 for Unit 1, replace "With one main steam line isolation valve. . ." with "With one or both main steam line isolation valve(s). . ." Also, add "The provisions of Specification 3.0.4 are not applicable." The first change clarifies the action statement to indicate that when both MSIVs are maintained closed they perform their required safety function. The statement regarding the inapplicability of Specification 3.0.4 is conservative in nature because it requires that during heatup all MSIVs be maintained closed until surveillance testing is complete. Also, it renders the Unit 1 specification the same as that of Unit 2. For these reasons, the staff finds these proposed changes acceptable.

c. In the action statement applicable to modes 2, 3, and 4 for Unit 2, replace "With one main steam line isolation valve inoperable. . ." with "With one or both main steam line isolation valve(s) inoperable. . ." Also, move "The provisions of Specification 3.0.4 are not applicable" to the end of the paragraph. The first change clarifies the action statement to indicate that when both MSIVs are maintained closed they perform their required safety function. Moving the statement regarding the inapplicability of Specification 3.0.4 to the end of the paragraph clarifies the intent of this action statement and eliminates otherwise confusing requirements. For these reasons, the staff finds these proposed changes acceptable.

2.2 <u>Main Feedwater Isolation Valves (MFIVs) - Unit 2</u>

The MFIVs are required to be operable to ensure that feedwater is terminated to the affected steam generator following a steam line break and that auxiliary feedwater is delivered to the intact steam generator following a feedwater line break.

The licensee proposed changes to Specification 3.7.1.6 for Unit 2, similar to the changes proposed for MSIVs, to clarify similar operational action statements.

a. In the action statement applicable to mode 1, delete ". . . and in COLD SHUTDOWN within the following 24 hours." This deletion eliminates a confusing statement, because once in HOT STANDBY the action statement for modes 2, 3 and 4 is applicable, and that already requires going down to COLD SHUTDOWN within the following 24 hours. For this reason, the staff finds the proposed change acceptable.

b. In the action statement applicable to modes 2, 3 and 4, replace "With one main feedwater line isolation valve inoperable. . ." with "With one or both main steam line isolation valve(s) inoperable. . ." Also, move "The provisions of Specification 3.0.4 are not applicable" to the end of the paragraph. The first change clarifies the action statement to indicate that when both MFIVs are maintained closed they perform their required safety function. Moving the statement regarding the inapplicability of Specification 3.0.4 to the end of paragraph clarifies the intent of this action statement and eliminates otherwise confusing requirements. For these reasons, the staff finds the proposed changes acceptable.

3.0 STATE CONSULTATION

Based upon the written notice of the proposed amendments, the Florida State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (54 FR 35102). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. A. Norris

Date: December 5, 1991