

June 9, 1992

Docket No. 50-335

DISTRIBUTION
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Mr. J. H. Goldberg
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

Dear Mr. Goldberg:

SUBJECT: ST. LUCIE UNIT 1 - ISSUANCE OF AMENDMENT RE: FUEL ASSEMBLIES
(TAC NO. M82953)

The Commission has issued the enclosed Amendment No. 114 to Facility Operating License No. DPR-67 for the St. Lucie Plant, Unit No. 1. This amendment consists of changes to the Technical Specifications in response to your application dated March 13, 1992.

This amendment revises Design Features Section 5.3.1, Fuel Assemblies, by deleting the maximum weight of uranium in a fuel rod and providing alternative requirements for fuel assemblies in the design features section.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed by)

Jan A. Norris, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 114 to DPR-67
2. Safety Evaluation

cc w/enclosures:
See next page

OFC	:LA:PDII-2	:PM:PDII-2	:D:PDII-2	:OGC	:	:
NAME	:D. Miller	:J. Norris	:H. Berlow	:W. Young	:	:
DATE	:5/15/92	:5/19/92	:5/19/92	:5/27/92	:	:

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Mr. J. H. Goldberg
Florida Power and Light Company

St. Lucie Plant

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DATED: June 9, 1992

AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. DPR-67 - ST. LUCIE, UNIT 1

Docket File

NRC & Local PDRs

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, (the licensee) dated March 13, 1992 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 114, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 9, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 114

TO FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove Page

5-4

Insert Page

5-4

FIGURE 5.1-2
(Deleted)

DESIGN FEATURES

5.2.1.2 SHIELD BUILDING

- a. Minimum annular space = 4 feet.
- b. Annulus nominal volume = 543,000 cubic feet.
- c. Nominal outside height (measured from top of foundation base to the top of the dome) = 230.5 feet.
- d. Nominal inside diameter = 148 feet.
- e. Cylinder wall minimum thickness = 3 feet.
- f. Dome minimum thickness = 2.5 feet.
- g. Dome inside radius - 112 feet.

DESIGN PRESSURE AND TEMPERATURE

5.2.2 The containment vessel is designed and shall be maintained for a maximum internal pressure of 44 psig and a temperature of 264°F.

PENETRATIONS

5.2.3 Penetrations through the containment structure are designed and shall be maintained in accordance with the original design provisions contained in Sections 3.8.2.1.10 and 6.2.4 of the FSAR with allowance for normal degradation pursuant to the applicable Surveillance Requirements.

5.3 REACTOR CORE

FUEL ASSEMBLIES

5.3.1 The reactor core shall contain 217 fuel assemblies with each fuel assembly containing a maximum of 176 fuel rods clad with Zircaloy-4. Each fuel rod shall have a nominal active fuel length of between 134.1 and 136.7 inches. Individual fuel assemblies shall contain fuel rods of the same nominal active fuel length. Fuel assemblies shall be limited to those designs that have been analyzed using NRC approved methodology and shown by tests or analyses to comply with fuel design and safety criteria. The initial core loading shall have a maximum enrichment of 2.83 weight percent U-235. Reload fuel shall be similar in physical design to the initial core loading.

5.3.2 Except for special test as authorized by the NRC, all fuel assemblies under control element assemblies shall be sleeved with a sleeve design previously approved by the NRC.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 114

TO FACILITY OPERATING LICENSE NO. DPR-67

FLORIDA POWER & LIGHT COMPANY

ST. LUCIE PLANT, UNIT NO. 1

DOCKET NO. 50-335

1.0 INTRODUCTION

By letter dated March 13, 1992, Florida Power and Light Company (FPL) requested an amendment to Facility Operating License DPR-67 for St. Lucie Unit 1. The proposed amendment would revise Technical Specifications Design Features Section 5.3.1, Fuel Assemblies, to delete the maximum weight of uranium in a fuel rod and provide alternative requirements for fuel assemblies in the design features section. Reload fuel assemblies would be limited to those designs that have been analyzed using an NRC-approved methodology and shown by tests or analyses to comply with all applicable design and safety criteria.

2.0 EVALUATION

The fuel assembly description in the Design Features Section of the Technical Specifications provides a description of the required characteristics of reload fuel, and requires that each rod in a fuel assembly shall contain a maximum total weight of 2250 grams uranium. However, the requirement for maximum fuel rod uranium weight is unnecessary because changes to the characteristics of the fuel rod (including uranium weight) that can impact design and safety criteria are specifically analyzed during the reload evaluation process. These analyses, using an NRC-approved methodology, assure that plant operation with the reload fuel assemblies comply with the safety limits and limiting conditions for operation in the Technical Specifications. In addition, any changes in the characteristics of the reload fuel assemblies will be limited to those designs that have been analyzed using an NRC-approved methodology and shown by tests or analyses to comply with all applicable design and safety criteria.

Therefore, the deletion of the maximum fuel rod weight in the Design Features Section 5.3.1 of the Technical Specifications on Fuel Assemblies will

permit changes in rod uranium weight while maintaining similarity in physical design to that of the initial core. Since compliance of the design with the limiting safety system settings and the limiting conditions for operation in the St. Lucie Unit 1 Technical Specifications is demonstrated during the reload evaluation process, the staff concludes that the proposed amendment will not adversely impact the safe operation of the plant and finds it acceptable.

3.0 STATE CONSULTATION

Based upon the written notice of the proposed amendment, the Florida State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (57 FR 11109). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: L. Kopp

Date: June 9, 1992