

April 28, 1988

Docket Nos. 50-335
and 50-389

Mr. W. F. Conway
Acting Group Vice President
Nuclear Energy
Florida Power and Light Company
Post Office Box 14000
Juno Beach, Florida 33408

Dear Mr. Conway:

DISTRIBUTION

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NRC & Local PDRs
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S. Varga
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E. Tourigny
OGC-WF
D. Hagan
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J. Partlow

T. Barnhart (8)
W. Jones
E. Butcher
ACRS (10)
GPA/PA
ARM/LFMB
Gray File

SUBJECT: ST. LUCIE UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE:
ORGANIZATION CHARTS (TAC NOS. 67374 AND 67375)

The Commission has issued the enclosed Amendment Nos. 93 and 29 to Facility Operating License Nos. DPR-67 and NPF-16 for the St. Lucie Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated February 22, 1988, as supplemented April 20, 1988.

These amendments replace the organization charts in the Technical Specifications with more general organization requirements. Various title changes were also made to the Company's Nuclear Review Board members. A specification was added to require the Operations Supervisor to continue to hold a Senior Reactor Operator license.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

E. G. Tourigny, Project Manager
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 93 to DPR-67
2. Amendment No. 29 to NPF-16
3. Safety Evaluation

cc w/enclosures:
See next page

*See previous concurrence

*LA:PDII-2	*PM:PDII-2	*D:PDII-2	OGC
DMiller	ETourigny:bg	HBerkow	
3/28/88	3/29/88	3/ /88	3/ /88

Handwritten signature: J. G. Tourigny
Handwritten date: 4/26/88
Handwritten initials: JGS

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PDR ADDCK 05000335
P PDR

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The Commission has issued the enclosed Amendment Nos. and to Facility Operating License Nos. DPR-67 and NPF-16 for the St. Lucie Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated February 22, 1988.

These amendments replace the organization charts in the Technical Specifications with more general organization requirements. Various title changes were also made to the Company's Nuclear Review Board members.

A Technical Specification was added that was discussed with and agreed to by your staff. It requires the Operations Supervisor to hold a Senior Reactor Operator license. In addition, the proposed title of senior corporate nuclear officer will be defined in the Topical Quality Assurance Report, as discussed with your staff.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

E. G. Tourigny, Project Manager
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. to DPR-67
2. Amendment No. to NPF-16
3. Safety Evaluation

cc w/enclosures:
See next page

L:PDII-2
Dwyer
3/25/88

PM PDII2
ETourigny:bg
3/21/88

LPEB
3/ /88

D:PDII-2
HBerKow
3/27/88

OGC
3/ /88

Mr. W. F. Conway
Florida Power & Light Company

St. Lucie Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 93
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated February 22, 1988, as supplemented by letter dated April 20, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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P PDR

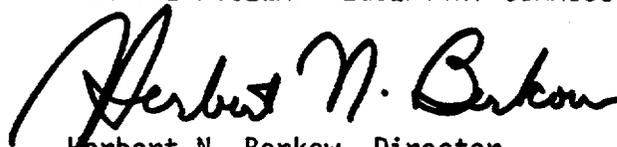
2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 93, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 28, 1988

ATTACHMENT TO LICENSE AMENDMENT NO.93
TO FACILITY OPERATING LICENSE NO. DPR-67
DOCKET NO. 50-335

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages

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6-5
6-7
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6-9
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6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Shift Supervisor, or during his absence from the control room a designated individual, shall be responsible for the control room command function. A management directive to this effect, signed by the Senior Vice President - Nuclear shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

ONSITE AND OFFSITE ORGANIZATION

6.2.1 An onsite and an offsite organization shall be established for unit operation and corporate management. This onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. Those relationships shall be documented and updated, as appropriate, in the form of organizational charts. These organizational charts will be documented in the Topical Quality Assurance Report and updated in accordance with 10 CFR 50.54(a)(3).
- b. The Senior Vice President - Nuclear shall be responsible for overall plant nuclear safety. This individual shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support in the plant so that continued nuclear safety is assured.
- c. The Plant Manager shall be responsible for overall unit safe operation and shall have control over those onsite resources necessary for safe operation and maintenance of the plant.
- d. Although the individuals who train the operating staff and those who carry out the quality assurance functions may report to the appropriate manager onsite, they shall have sufficient organizational freedom to be independent from operating pressures.
- e. Although health physics individuals may report to any appropriate manager onsite, for matters relating to radiological health and safety of employees and the public, the health physics manager shall have direct access to that onsite individual having responsibility for overall unit management. Health physics personnel shall have the authority to cease any work activity when worker safety is jeopardized or in the event of unnecessary personnel radiation exposures.

6.0 ADMINISTRATIVE CONTROLS

6.2 ORGANIZATION (Continued)

UNIT STAFF

6.2.2 The unit organization shall be subject to the following:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the reactor is in MODE 1, 2, 3, or 4, at least one licensed Senior Reactor Operator shall be in the control room.
- c. A health physics technician[#] shall be on site when fuel is in the reactor.
- d. All CORE ALTERATIONS shall be observed by a licensed operator and supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation. The SRO in charge of fuel handling normally supervises from the control room and has the flexibility to directly supervise at either the refueling deck or the spent fuel pool.
- e. A Site Fire Brigade[#] of at least five members shall be maintained onsite at all times[#]. The Fire Brigade shall not include the shift supervisor, the STA, nor the two other members of the

[#]The health physics technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.

DELETED

Table 6.2-1

MINIMUM SHIFT CREW COMPOSITION

TWO UNITS WITH TWO SEPARATE CONTROL ROOMS

WITH UNIT 2 IN MODE 5 OR 6 OR DEFUELED		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODE 1, 2, 3 or 4	MODE 5 or 6
SS (SRO)	1 ^a	1 ^a
SRO	1	None
RO	2	1 ^b
AO	2	2 ^b
STA	1	None

WITH UNIT 2 IN MODE 1, 2, 3, or 4		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODE 1, 2, 3, or 4	MODE 5 or 6
SS (SRO)	1 ^a	1 ^a
SRO	1	None
RO	2	1
AO	2 ^a	1
STA	1 ^a	None

- SS - Shift Supervisor with a Senior Reactor Operator's License on Unit 1
- SRO - Individual with a Senior Reactor Operator's License on Unit 1
- STA - Shift Technical Advisor
- RO - Individual with a Reactor Operator's License on Unit 1
- AO - Auxiliary Operator

Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 1, 2, 3 or 4, an individual (other than the Shift Technical Advisor) with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 5 or 6, an individual with a valid SRO or RO license shall be designated to assume the Control Room command function.

a/ Individual may fill the same position on Unit 2.

b/ One of the two required individuals may fill the same position on Unit 2.

ADMINISTRATIVE CONTROLS

minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.

- f. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions; e.g., senior reactor operators, reactor operators, health physicists, auxiliary operators, and key maintenance personnel.

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the plant is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance or major plant modification, on a temporary basis the following guidelines shall be followed:

1. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
2. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time.
3. A break of at least 8 hours should be allowed between work periods, including shift turnover time.
4. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager or his deputy, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Plant Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

- g. The Operations Supervisor shall hold a Senior Reactor Operator license.

SHIFT TECHNICAL ADVISOR

6.2.3 The Shift Technical Advisor function is to provide on shift advisory technical support in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit.

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS-3.1-1978 as endorsed by Regulatory Guide 1.8, September 1975 (reissued May 1977), except for (1) the Health Physics Supervisor who shall meet or exceed the qualifications of Regulatory Guide 1.8., September 1975, and (2) the Shift Technical Advisor who shall have a bachelor degree or equivalent in a scientific or engineering discipline with specific training in plant design and plant operating characteristics, including transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI/ANS-3.1-1978 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 FACILITY REVIEW GROUP (FRG)

FUNCTION

6.5.1.1 The Facility Review Group shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Facility Review Group shall be composed of the:

Member:	Plant Manager
Member:	Operations Superintendent
Member:	Operations Supervisor
Member:	Maintenance Superintendent
Member:	Instrument & Control Supervisor
Member:	Reactor Supervisor
Member:	Health Physics Supervisor
Member:	Technical Supervisor
Member:	Chemistry Supervisor
Member:	Quality Control Supervisor
Member:	Assistant Plant Supt. Mechanical
Member:	Assistant Plant Supt. Electrical

The Chairman shall be a member of the FRG and shall be designated in writing.

ADMINISTRATIVE CONTROLS

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the FRG Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in FRG activities at any one time.

MEETING FREQUENCY

6.5.1.4 The FRG shall meet at least once per calendar month and as convened by the FRG Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the FRG necessary for the performance of the FRG responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Facility Review Group shall be responsible for:

- a. Review of (1) all procedures required by Specification 6.8 and changes thereto, (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix A Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Senior Vice President - Nuclear and to the Chairman of the Company Nuclear Review Board.
- f. Review of all REPORTABLE EVENTS.
- g. Review of unit operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant Manager or the Company Nuclear Review Board.

ADMINISTRATIVE CONTROLS

- i. Review of the Security Plan and implementing procedures and submittal of recommended changes to the Company Nuclear Review Board.
- j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Company Nuclear Review Board.
- k. Review of every unplanned on-site release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Senior Vice President - Nuclear and to the Company Nuclear Review Board.
- l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL and RADWASTE TREATMENT SYSTEMS.
- m. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last FRG meeting.

AUTHORITY

6.5.1.7 The Facility Review Group shall:

- a. Recommend in writing to the Plant Manager, approval or disapproval of items considered under Specifications 6.5.1.6.a through d above.
- b. Render determinations in writing with regard to whether or not each item considered under Specifications 6.5.1.6 a through e above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Senior Vice President - Nuclear and the Company Nuclear Review Board of disagreement between the FRG and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.1.8 The Facility Review Group shall maintain written minutes of each FRG meeting that, at a minimum, document the results of all FRG activities performed under the responsibility and authority provisions of these Technical Specifications. Copies shall be provided to the Senior Vice President - Nuclear and the Chairman of the Company Nuclear Review Board.

ADMINISTRATIVE CONTROLS

6.5.2 COMPANY NUCLEAR REVIEW BOARD (CNRB)

FUNCTION

6.5.2.1 The Company Nuclear Review Board shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 The CNRB shall be composed of the following members:

- Member: Group Vice President
- Member: Senior Vice President - Nuclear
- Member: Vice President - Engineering, Projects & Construction
- Member: Vice President - Nuclear Energy
- Member: Director - Nuclear Licensing
- Member: Director - Quality Assurance
- Member: Chief Engineer - Power Plant Engineering
- Member: Manager - Nuclear Energy Services
- Member: Manager - Nuclear Fuel
- Member: Manager - Power Plant Engineering

The Chairman shall be a member of the CNRB and shall be designated in writing.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the CNRB Chairman to serve on temporary basis; however, no more than two alternates shall participate as voting members in CNRB activities at any one time.

ADMINISTRATIVE CONTROLS

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the CNRB Chairman to provide expert advice to the CNRB.

MEETING FREQUENCY

6.5.2.5 The CNRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter and as convened by the CNRB Chairman or his designated alternate.

QUORUM

6.5.2.6 The quorum of the CNRB necessary for the performance of the CNRB review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least four CNRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

REVIEW

6.5.2.7 The CNRB shall review:

- a. The safety evaluations for (1) changes to procedures, equipment, or systems and (2) tests or experiments completed under the provisions of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment, or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.

ADMINISTRATIVE CONTROLS

- g. All REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meeting minutes of the Facility Review Group.

AUDITS

6.5.2.8 Audits of unit activities shall be performed under the cognizance of the CNRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire unit staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months.
- e. Any other area of unit operation considered appropriate by the CNRB or the Executive Vice President.
- f. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel.
- g. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year.
- h. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- i. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
- j. The PROCESS CONTROL PROGRAM and implementing procedures for dewatering of radioactive bead resin at least once per 24 months.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.2.9 The CNRB shall report to and advise the Executive Vice President on those areas of responsibility specified in Specifications 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of CNRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each CNRB meeting shall be prepared, approved and forwarded to the Executive Vice President within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Executive Vice President within 14 days following completion of the review.
- c. Audit reports encompassed by Specification 6.5.2.8 above, shall be forwarded to the Executive Vice President and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the FRG, and the results of the review shall be submitted to the CNRB, and the Senior Vice President - Nuclear.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Senior Vice President - Nuclear and the CNBR shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the FRG. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

ADMINISTRATIVE CONTROLS

- c. The Safety Limit Violation Report shall be submitted to the Commission, the CNRB, and the Senior Vice President - Nuclear within 14 days of the violation.
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978, and those required for implementing the requirements of NUREG 0737.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Control Program for effluent monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974.
- j. Quality Control Program for environmental monitoring using the guidance in Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each procedure of Specification 6.8.1a through i. above, and changes thereto, shall be reviewed by the FRG and shall be approved by the Plant Manager prior to implementation and shall be reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of Specification 6.8.1a through i. above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.

ADMINISTRATIVE CONTROLS

- c. The change is documented, reviewed by the FRG and approved by the Plant Manager within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, maintained, and shall be audited under the cognizance of the CNRB at least once per 24 months:

- a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the Shutdown Cooling System, High Pressure Safety Injection System, Containment Spray System, and RCS Sampling. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.

- b. In-Plant Radioiodine Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for monitoring, and
- (iii) Provisions for maintenance of sampling and analysis equipment.

- c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- (i) Identification of a sampling schedule for the critical variables and control points for these variables,
- (ii) Identification of the procedures used to measure the values of the critical variables,
- (iii) Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in-leakage,



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY
ORLANDO UTILITIES COMMISSION OF
THE CITY OF ORLANDO, FLORIDA

AND

FLORIDA MUNICIPAL POWER AGENCY

DOCKET NO. 50-389

ST. LUCIE PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29
License No. NPF-16

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company, et al. (the licensee), dated February 22, 1988, as supplemented by letter dated April 20, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

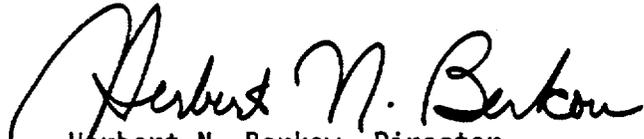
2. Accordingly, Facility Operating License No. NPF-16 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.2 to read as follows:

2. Technical Specifications

- The Technical Specifications contained in Appendices A and B, as revised through Amendment No. ²⁹, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Charges to the Technical
Specifications

Date of Issuance: April 28, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 29
TO FACILITY OPERATING LICENSE NO. NPF-16
DOCKET NO. 50-389

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages

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Insert Pages

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ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.

6.1.2 The Shift Supervisor, or during his absence, from the control room, a designated individual, shall be responsible for the control room command function. A management directive to this effect, signed by the Senior Vice President - Nuclear, shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

ONSITE AND OFFSITE ORGANIZATION

6.2.1 An onsite and an offsite organization shall be established for unit operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility and communication shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. Those relationships shall be documented and updated, as appropriate, in the form of organizational charts. These organizational charts will be documented in the Topical Quality Assurance Report and updated in accordance with 10 CFR 50.54(a)(3).
- b. The Senior Vice President - Nuclear shall be responsible for overall plant nuclear safety. This individual shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support in the plant so that continued nuclear safety is assured.
- c. The Plant Manager shall be responsible for overall unit safe operation and shall have control over those onsite resources necessary for safe operation and maintenance of the plant.
- d. Although the individuals who train the operating staff and those who carry out the quality assurance functions may report to the appropriate manager onsite, they shall have sufficient organizational freedom to be independent from operating pressures.
- e. Although health physics individuals may report to any appropriate manager onsite, for matters relating to radiological health and safety of employees and the public, the health physics manager shall have direct access to that onsite individual having responsibility for overall unit management. Health physics personnel shall have the authority to cease any work activity when worker safety is jeopardized or in the event of unnecessary personnel radiation exposures.

ADMINISTRATIVE CONTROLS

6.2 ORGANIZATION (Continued)

UNIT STAFF

6.2.2 The unit organization shall be subject to the following:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the reactor is in MODE 1, 2, 3, or 4, at least one licensed Senior Reactor Operator shall be in the control room.
- c. A health physics technician[#] shall be on site when fuel is in the reactor.
- d. All CORE ALTERATIONS shall be observed by a licensed operator and supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation. The SRO in charge of fuel handling normally supervises from the control room and has the flexibility to directly supervise at either the refueling deck or the spent fuel pool.
- e. A site Fire Brigade of at least five members shall be maintained onsite at all times.[#] The Fire Brigade shall not include the Shift Supervisor, the STA, nor the two other members of the minimum shift crew necessary for safe shutdown of the unit and any personnel required for other essential functions during a fire emergency.
- f. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions; e.g., senior reactor operators, reactor operators, health physicists, auxiliary operators, and key maintenance personnel. Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the plant is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance or major plant modification, on a temporary basis the following guidelines shall be followed:

[#]The health physics technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.

ADMINISTRATIVE CONTROLS

UNIT STAFF (Continued)

- a. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
- b. An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time.
- c. A break of at least 8 hours should be allowed between work periods, including shift turnover time.
- d. Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Plant Manager or his deputy, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Plant Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

- g. The Operations Supervisor shall hold a Senior Reactor Operator License.

DELETED

DELETED

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS (Continued)

or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and plant operating characteristics, including transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI/ANS 3.1-1978 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.1 FACILITY REVIEW GROUP (FRG)

FUNCTION

6.5.1.1 The Facility Review Group shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Facility Review Group shall be composed of the:

- Member: Plant Manager
- Member: Operations Superintendent
- Member: Operations Supervisor
- Member: Maintenance Superintendent
- Member: Instrument & Control Supervisor
- Member: Reactor Supervisor
- Member: Health Physics Supervisor
- Member: Technical Supervisor
- Member: Chemistry Supervisor
- Member: Quality Control Supervisor
- Member: Assistant Plant Supt. Electrical
- Member: Assistant Plant Supt. Mechanical

The Chairman shall be a member of the FRG and shall be designated in writing.

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the FRG Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in FRG activities at any one time.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.1.4 The FRG shall meet at least once per calendar month and as convened by the FRG Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the FRG necessary for the performance of the FRG responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Facility Review Group shall be responsible for:

- a. Review of (1) all procedures required by Specification 6.8 and changes thereto, (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix A Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Senior Vice President - Nuclear and to the Chairman of the Company Nuclear Review Board.
- f. Review of all REPORTABLE EVENTS.
- g. Review of unit operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Plant Manager or the Company Nuclear Review Board.
- i. Review of the Security Plan and implementing procedures and submittal of recommended changes to the Company Nuclear Review Board.
- j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Company Nuclear Review Board.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- k. Review of every unplanned onsite release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Senior Vice President - Nuclear and to the Company Nuclear Review Board.
- l. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL and RADWASTE TREATMENT SYSTEMS.
- m. Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last FRG meeting.

AUTHORITY

6.5.1.7 The Facility Review Group shall:

- a. Recommend in writing to the Plant Manager approval or disapproval of items considered under Specifications 6.5.1.6a. through d. and m. above.
- b. Render determinations in writing with regard to whether or not each item considered under Specifications 6.5.1.6a. through e. above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Senior Vice President - Nuclear and the Company Nuclear Review Board of disagreement between the FRG and the Plant Manager; however, the Plant Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1 above.

RECORDS

6.5.1.8 The Facility Review Group shall maintain written minutes of each FRG meeting that, at a minimum, document the results of all FRG activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the Senior Vice President - Nuclear and the Chairman of the Company Nuclear Review Board.

6.5.2 COMPANY NUCLEAR REVIEW BOARD (CNRB)

FUNCTION

6.5.2.1 The Company Nuclear Review Board shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy

ADMINISTRATIVE CONTROLS

FUNCTION (Continued)

- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

COMPOSITION

6.5.2.2 The CNRB shall be composed of the following members:

- Member: Group Vice President
- Member: Senior Vice President - Nuclear
- Member: Vice President - Engineering, Projects & Construction
- Member: Vice President - Nuclear Energy
- Member: Director - Nuclear Licensing
- Member: Director - Quality Assurance
- Member: Chief Engineer - Power Plant Engineering
- Member: Manager - Nuclear Energy Services
- Member: Manager - Nuclear Fuel
- Member: Manager - Power Plant Engineering

The Chairman shall be a member of the CNRB and shall be designated in writing.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the CNRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in CNRB activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the CNRB Chairman to provide expert advice to the CNRB.

MEETING FREQUENCY

6.5.2.5 The CNRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter and as convened by the CNRB Chairman or his designated alternate.

QUORUM

6.5.2.6 The quorum of the CNRB necessary for the performance of the CNRB review and audit functions of these Technical Specifications shall consist of the Chairman and his designated alternate and at least four CNRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

ADMINISTRATIVE CONTROLS

6.6 REPORTABLE EVENTS ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the FRG, and the results of this review shall be submitted to the CNRB, and the Senior Vice President - Nuclear.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Senior Vice President-Nuclear and the CNRB shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the FRG. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- c. The Safety Limit Violation Report shall be submitted to the Commission, the CNRB, and the Senior Vice President - Nuclear within 14 days of the violation.
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978, and those required for implementing the requirements of NUREG 0737.
- b. Refueling operations.
- c. Surveillance and test activities of safety-related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Control Program for effluent monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974.
- j. Quality Control Program for environmental monitoring using the guidance in Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each procedure of Specification 6.8.1a. through i. above, and changes thereto, shall be reviewed by the FRG and shall be approved by the Plant Manager prior to implementation and shall be reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of Specification 6.8.1a. through i. above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the FRG and approved by the Plant Manager within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, maintained, and shall be audited under the cognizance of the CNRB at least once per 24 months:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the Shutdown Cooling System, High Pressure Safety Injection System, Containment Spray System, and RCS Sampling. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radioiodine Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for monitoring, and
- (iii) Provisions for maintenance of sampling and analysis equipment.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 93 AND 29
TO FACILITY OPERATING LICENSE NOS. DPR-67 AND NPF-16
FLORIDA POWER & LIGHT COMPANY, ET AL.
ST. LUCIE PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-335 AND 50-389

INTRODUCTION

By letter dated February 22, 1988, Florida Power and Light Company (FP&L, the licensee) submitted a request for changes to Section 6, Administrative Controls, of the St. Lucie Plant, Unit Nos. 1 and 2, Technical Specifications. FP&L proposed deleting figures representing the offsite organizational structure (Figure 6.2-1) and the onsite organizational structure (Figure 6.2-2). FP&L proposed adding general organizational requirements, in place of the figures. The figures will be placed in the licensee's Topical Quality Assurance Report. Most of the elements contained in the figures are already required by regulation, other TS, the Updated Final Safety Analysis Report, or the Topical Quality Assurance Report, as described in the evaluation below.

FP&L also proposed other changes. The title "Vice President - Nuclear Operations" and "Group Vice President - Nuclear Energy" are used in various locations of the Administrative Controls section of the TS. FP&L proposed to delete these titles and use the title "Senior Vice President - Nuclear." Titles of personnel on the Company Nuclear Review Board (CNRB) are also proposed to be changed.

Because the first proposed change (deletion of organization charts from the TS) is the most significant of those in the amendment request, the next several pages of this Safety Evaluation are devoted to an evaluation of this issue, with a discussion of applicable regulatory requirements, past practice, and safety considerations. The remaining TS changes proposed by FP&L are then evaluated.

Additional information was submitted by letter dated April 20, 1988. The additional information did not alter, in any way, the staff's proposed no significant hazards consideration determination published in the Federal Register on March 23, 1988 (53 FR 9503).

On March 22, 1988, subsequent to the licensee's application, the staff issued Generic Letter 88-06 entitled "Removal of Organization Charts from Technical Specification Administrative Control Requirements." The staff used the Generic Letter as guidance in reviewing the licensee's proposed changes.

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2.0 EVALUATION

Section 6.0 of the licensee's TS is required to contain the details of those administrative controls necessary to assure safe operation of the facility. For the first proposed change, FP&L proposed to replace TS 6.2.1, TS Figure 6.2-1 (the figure showing the offsite organization), and TS Figure 6.2-2 (the figure showing the onsite organization) with more general organizational requirements. These general requirements capture the essence of those organizational features depicted on the charts that are important to the NRC for ensuring that the plant will be operated safely.

FP&L stated that the proposed changes are justified because they are administrative in nature and do not affect plant operation. FP&L notes that, in addition to being required by the TS, the important organizational features depicted on the organization charts are also required or controlled by other regulatory control mechanisms. For example, FP&L's Quality Assurance (QA) Program for St. Lucie is required by 10 CFR 50, Appendix B, to include similar information relating to the organizational structure.

Regulatory Requirements Applicable to Organizational Structure

10 CFR 50.36, "Technical Specifications," which implements Section 182a. of the Atomic Energy Act, was promulgated by the Commission on December 17, 1968 (33 FR 18610). This rule delineates requirements for determining the contents of the TS. Technical Specifications set forth the specific characteristics of the facility and the conditions for its operation that are required to provide adequate protection to the health and safety of the public. Specifically, 10 CFR 50.36 requires that:

Each license authorizing operation of a production or utilization facility of a type described in §50.21 or §50.22 will include Technical Specifications. The Technical Specifications will be derived from the analyses and evaluation included in the safety analysis report, and amendments thereto, submitted pursuant to 50.34. The Commission may include such additional TS as the Commission finds appropriate.

The regulation further states that the TS will include, among other things, items in the following category:

(5) Administrative Controls: Administrative controls are the provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner. Each licensee shall submit any reports to the Commission pursuant to approved Technical Specifications as specified in §50.4.

Past Practice

Review of the organization, personnel qualifications, education, experience, training, and an applicant's overall capacity to operate a plant safely has always been of concern to the NRC, and its predecessor, the AEC. Before a plant is licensed to operate, a finding is made that the applicant's staff is capable

of operating the plant safely. In the past, the organization charts were made TS so that changes made after operation began would require prior NRC approval. This was done to preserve certain specific features of the licensed organization.

While the regulation does not specifically require that the TS contain organization charts, the practice of including organization charts in the TS began in the late 1960s. These charts were used as an aid in depicting the organizational and management relationships thought to be needed to meet the provisions of 10 CFR 50.36(c)(5). The practice of including organization charts in the TS has continued since.

Organization charts do depict the reporting chain for some organizational functions that must be independent of scheduling and operating pressures. Until 10 CFR Part 50, Appendix B, - "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants," was adopted in 1970, organization charts were partially relied on by the staff for assuring this function.

As stated in 10 CFR 50, Appendix B, Criterion I, "Organization":

Such persons and organizations performing quality assurance functions shall report to a management level such that this required authority and organizational freedom, including sufficient independence from cost and schedule when opposed to safety considerations, are provided.

Appendix B further acknowledges that the organizational structures may take many forms, but emphasizes that regardless of structure, the individuals assigned the responsibility for execution of any portion of the program shall have access to such levels of management as may be necessary to perform this function. The licensee's required QA Program specifies and depicts these organizational relationships in greater detail than currently exists in the TS.

The practice of including organization charts in the TS was established before the advent of 10 CFR 50, Appendix B, - "Quality Assurance Criteria for Nuclear Power Plants and Fuel Processing Plants," and other associated guidance documents, such as the Regulatory Guide 1.70, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants," and NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants - LWR Edition." A general description of the features needed by the staff to make the finding that the applicant is capable of operating the plant safely are now mandated by Appendix B.

Safety Considerations

The fundamental safety issue in the proposal to delete organization charts from the TS is whether there can be reasonable assurance that the organization will operate the plant safely and remain effective without requiring prior staff approval for changes reflected in organization charts.

It has been the staff's experience that organization charts by themselves have been little help to reviewers in assessing the safety significance of changes to the plant and license. Nevertheless, because the charts are in the TS, license amendment requests have been required to effect organizational changes as simple as combining some minor functions under one organizational element shown on the chart. The usefulness of the charts to the staff in recent years has been minimal and the safety relevance of the charts themselves is small.

Specific operational requirements that bear more directly on the safety matters of concern to the staff than the organization charts are required elsewhere in the TS. For example, the organizational element responsible for control room command function is identified separately in the TS, as are the requirements for minimum staffing under various operating modes. The organizational management functions for independent reviews and audits, unit review group and independent safety engineering groups, and shift technical advisor are also specified in other TS. Thus, the organization charts themselves are not needed to support the staff's finding that the organization will operate the plant safely.

In summary, the specific details of the operating organization are not essential to the safe operation of the facility, and the staff concludes that the details can be modified in many ways while maintaining adequate operational safety. Over the years of experience with the details of operating organizations, the staff has been able to distill those organizational characteristics which are important to assure plant safety.

The important features of a licensee's organization (currently depicted on the TS organization charts, but not already included in other TS) necessary for the staff to find that the organization will operate the plant safely are stated below. These amendments revise the TS to add statements incorporating these features to replace the organization charts.

- a. Lines of authority, responsibility and communications shall be established and defined from the highest management levels through intermediate levels to and including all operating organization positions. Those relationships shall be documented and updated, as appropriate, in the form of organizational charts, functional descriptions of departmental responsibilities and relationships and job descriptions for key personnel positions, or in equivalent forms of documentation. These organizational relationships will be maintained in a document such as the FSAR or QA Manual.
- b. There shall be an individual executive position (corporate officer) in the offsite organization having corporate responsibility for overall plant nuclear safety. This individual shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant so that continued nuclear safety is assured.

- c. There shall be an individual management position in the onsite organization having responsibilities for overall unit safe operation and shall have control over those onsite resources necessary for safe operation and maintenance of the plant.
- d. Although the individuals who train the operating staff and those who carry out the health physics and quality assurance functions may report to the appropriate manager on site, they shall have sufficient organizational freedom to be independent from operating pressures.
- e. Senior Reactor Operator (SRO) and Reactor Operator (RO) licenses shall continue to be required for the positions so indicated on the current TS organization charts.
- f. Other TS which reference the current organization charts shall be revised to reference the appropriate functional responsibility or position.

The proposed changes (TS 6.2.1.a. through e.) incorporate these features. The significant aspects of the changes are further discussed as follows.

In regard to proposed TS 6.2.1.a. (a. above), the licensee proposes the same words and will place the offsite and onsite organization charts in the St. Lucie Plant Topical Quality Assurance Report. This report is a licensee-controlled document and cannot be changed unilaterally if the contained commitments would be reduced. The governing regulation is 10 CFR 50.54(a)(3). Proposed TS 6.2.1.b. through e. contain the same recommended requirements as b. through d. above. In regard to b. and c. above, the Senior Vice President - Nuclear is specified as the responsible offsite executive and the Plant Manager is specified as the responsible onsite manager. In connection with e. above (SRO/RO licenses), the licensee proposed a TS to continue to require the Operations Supervisor to hold an SRO license.

The licensee's letter of April 20, 1988 clarified the request to continue the requirement that the Operations Supervisor hold an SRO license. This does not affect the substance of the amendments as noticed nor the staff's proposed no significant hazards consideration determination.

Lastly, the licensee proposed changes to the TS to implement recommendation f. above.

Based upon the above discussion, the staff concludes that the removal of the organization charts from the TS will not prevent the licensee from meeting the standards of 10 CFR 50.36 and the underlying statutory requirements. Moreover, the deletion of the unnecessary detail of organization charts will save resources for both the NRC and the licensee and will allow the staff to focus on issues of importance to the plant's safety. The proposed changes are acceptable.

The licensee proposed to delete the titles "Vice President - Nuclear Operations" and "Group Vice President - Nuclear Energy" and replace these titles with the title "Senior Vice President - Nuclear." These titles are located in various places in Section 6 of the TS. The change was requested because of an upcoming reorganization. The title "Group Vice President - Nuclear Energy" will become "Senior Vice President - Nuclear." This proposed change is acceptable, since a person on the vice president level will continue to be responsible for these activities.

In the February 22, 1988 application, the licensee initially proposed to identify the management officers by general terminology. However, after discussions with the NRC staff, the licensee by letter dated April 20, 1988 modified the request to identify the Senior Vice President-Nuclear and the Plant Manager-Nuclear by their titles in accordance with the recommendations of GL 88-06. This change merely adds clarity to the more general terminology used in the February 22, 1988 request and does not affect the substance of the amendments as noticed nor the staff's proposed no significant hazards consideration determination.

The licensee proposed three title changes to the membership of the CNRB. The licensee stated that there will be no changes in collective talents on the CNRB and that the quality and scope of independent review will be maintained. The titles also need to be changed because of an upcoming offsite organizational change. The title "Group Vice President - Nuclear Energy" will be changed to "Senior Vice President - Nuclear." The person occupying this position will also change. The title "Vice President - Nuclear Operations" will be changed to "Vice President - Nuclear Energy." The person occupying this position will not change. The title "Senior Project Manager - Power Plant Engineering" will be deleted, and the title "Manager-Power Plant Engineering" will be added. This will still require a senior person from Power Plant Engineering to be a member of the CNRB. The staff agrees that the collective talents on the CNRB will not be changed and that the quality and scope of independent review will be maintained. Therefore, the changes are acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, or administrative procedures and requirements. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 28, 1988

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