

June 13, 1988

Docket No. 50-335

Mr. W. F. Conway  
Senior Vice President - Nuclear  
Florida Power and Light Company  
Post Office Box 14000  
Juno Beach, Florida 33408

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Dear Mr. Conway:

SUBJECT: ST. LUCIE UNIT 1 - ISSUANCE OF AMENDMENT RE: STATION AIR SYSTEM -  
CHANGES TO VALVE TABLES (TAC NO. 65002)

The Commission has issued the enclosed Amendment No. 95 to Facility Operating License No. DPR-67 for the St. Lucie Plant, Unit No. 1. This amendment consists of changes to the Technical Specifications in response to your application dated March 17, 1987.

This amendment authorizes the installation of additional containment isolation valves in the station air system and changes the valve tag numbers associated with the station air supply header through containment (TS Tables 3.6-1 and 3.6-2). Upon completion of the modification to the station air system, you are requested to notify the NRC and you should initiate action to revise the pages of Tables 3.6-1 and 3.6-2 to incorporate the new valve tag numbers and delete the associated footnotes.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

E. G. Tourigny, Project Manager  
Project Directorate II-2  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 95 to DPR-67
2. Safety Evaluation

cc w/enclosures:  
See next page

\*See previous concurrences

LA:PDII-2\*      PM:PDII-2\*      D:PDII-2\*      OGC-WF\*  
DAMiller      ETourigny:bd      HBerkow      SHLewis  
03/28/88      03/28/88      03/28/88      04/22/88

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Mr. W. F. Conway  
Florida Power & Light Company

St. Lucie Plant

cc:

Mr. Jack Shreve  
Office of the Public Counsel  
Room 4, Holland Building  
Tallahassee, Florida 32304

Resident Inspector  
c/o U.S. NRC  
7585 S. Hwy A1A  
Jensen Beach, Florida 34957

State Planning & Development  
Clearinghouse  
Office of Planning & Budget  
Executive Office of the Governor  
The Capitol Building  
Tallahassee, Florida 32301

Harold F. Reis, Esq.  
Newman & Holtzinger  
1615 L Street, N.W.  
Washington, DC 20036

John T. Butler, Esq.  
Steel, Hector and Davis  
4000 Southeast Financial Center  
Miami, Florida 33131-2398

Administrator  
Department of Environmental Regulation  
Power Plant Siting Section  
State of Florida  
2600 Blair Stone Road  
Tallahassee, Florida 32301

Mr. Weldon B. Lewis, County  
Administrator  
St. Lucie County  
2300 Virginia Avenue, Room 104  
Fort Pierce, Florida 33450

Mr. Charles B. Brinkman, Manager  
Washington - Nuclear Operations  
Combustion Engineering, Inc.  
7910 Woodmont Avenue  
Bethesda, Maryland 20814

Jacob Daniel Nash  
Office of Radiation Control  
Department of Health and  
Rehabilitative Services  
1317 Winewood Blvd.  
Tallahassee, Florida 32399-0700

Regional Administrator, Region II  
U.S. Nuclear Regulatory Commission  
101 Marietta Street N.W., Suite 2900  
Atlanta, Georgia 30323



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 95  
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power & Light Company, (the licensee) dated March 17, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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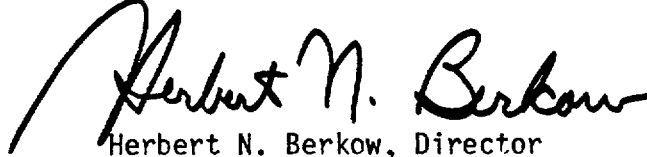
2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 95, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 13, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 95  
TO FACILITY OPERATING LICENSE NO. DPR-67  
DOCKET NO. 50-335

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages

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3/4 6-20

3/4 6-21

Insert Pages

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3/4 6-20

3/4 6-21

TABLE 3.6-1

CONTAINMENT LEAKAGE PATHS

<u>Penetration</u>	<u>System</u>	<u>Valve Tag Number</u>	<u>Location to Containment</u>	<u>Service</u>	<u>Test Type*</u>
7	Makeup Water	Gate (I-MV-15-1) Check (I-V-15-1347)	Outside Inside	Primary Makeup Water	Bypass
8	Station Air	Globe (I-V-18-947)+ Globe (I-V-18-947)++ Check (I-V-181118)+++ Globe (I-SH-18797)+++	Outside Outside Inside+++ Annulus+++	Station Air Supply	Bypass
9	Instrument Air	Gate (I-MV-18-1) Check (I-V-18-957)	Outside Inside	Instrument Air Supply	Bypass
10	Containment Purge	Butterfly (I-FCV-25-4) Butterfly (I-FCV-25-5)	Inside Outside	Containment Purge Exhaust	Type C
11	Containment Purge	Butterfly (I-FCV-25-3) Butterfly (I-FCV-25-2)	Inside Outside	Containment Purge Supply	Type C
14	Waste Management	Globe (V-6741) Check (V-6779)	Outside Outside	Nitrogen supply to SI Tanks	Bypass
23	Component Cooling	Butterfly (I-HCV 14-7) Butterfly (I-HCV-14-1)	Outside Outside	RC Pump CW Supply	Bypass
24	Component Cooling	Butterfly (I-HCV-14-6) Butterfly (I-HCV-14-2)	Outside Outside	RC Pump CW Return	Bypass
25	Fuel Transfer Tube	Double Gasket Flange	Inside	Fuel Transfer	Bypass
26	CVCS	Globe (V-2515) Globe (V-2516)	Inside Inside	Letdown Line	Bypass
28	Sampling	Globe (V-5200) Globe (V-5203) Globe (I-FCV-03-1E) Globe (I-FCV-03-1F)	Outside Outside Outside Outside	Reactor Coolant Sample SI Tank Sample SI Tank Sample	Bypass Bypass

+To become I-HCV-18-2 upon completion of the modification described in L-87-123.

++To become I-V-18795 upon completion of the modification described in L-87-123.

+++To become effective upon completion of the modification described in L-87-123.

ST. LUCIE - UNIT 1

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TABLE 3.6-1 (Continued)

<u>Penetration</u>	<u>System</u>	<u>Valve Tag Number</u>	<u>Location to Containment</u>	<u>Service</u>	<u>Test Type*</u>
29	Sampling	Globe (V-5202) Globe (V-5205)	Outside Outside	Pressurizer Steam Space Sample	Bypass
29	Sampling	Globe (V-5201) Globe (V-5204)	Outside Outside	Pressurizer Surge Line Sample	Bypass
31	Waste Management	Gate (V-6554) Gate (V-6555)	Outside Outside	Containment Vent Header	Bypass
41	Safety Injection Tank Test Lines	Gate (V-3463) Gate (I-V-03-1307)	Outside Outside	Safety Injection Tank Fill and Sampling	Bypass
42	Waste Management	Gate (I-LCV-07-11A) Gate (I-LCV-07-11B)	Outside Outside	Reactor Cavity Sump Pump Discharge	Bypass
43	Waste Management	Gate (V-6301) Gate (V-6302)	Outside Outside	Reactor Drain Tank Pump Suction	Bypass
44	CVCS	Gate (V-2505) Gate (I-SE-01-1)	Outside Inside	RC Pump Controlled Bleedoff	Bypass
46	Fuel Pool Cleanup	Gate (I-V-07-206) Gate (I-V-07-189)	Outside Inside	Refueling Cavity Purification Flow Inlet	Bypass
47	Fuel Pool Cleanup	Gate (I-V-07-170) Gate (I-V-07-188)	Outside Inside	Refueling Cavity Purification Flow Outlet	Bypass
48	Sampling	Globe (I-FSE-27-01,02, 03,04) Globe (I-FSE-27-08)	Inside Outside	H <sub>2</sub> Sampling	Type C

## CONTAINMENT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

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4.6.3.1.2 Each isolation valve specified in Table 3.6-2 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

- a. Verifying that on a Containment Isolation test signal, and/or SIAS test signal, each isolation valve actuates to its isolation position.

4.6.3.1.3 The isolation time of each power operated or automatic valve of Table 3.6-2 shall be determined to be within its limits when tested pursuant to Specification 4.0.5.



ST. LUCIE - UNIT 1

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Amendment No. 95

TABLE 3.6-2

CONTAINMENT ISOLATION VALVES

<u>Valve Tag Number</u>	<u>Penetration Number</u>	<u>Function</u>	<u>Testable During Plant Operation</u>	<u>Isolation Time (Sec)</u>
A. CONTAINMENT ISOLATION				
1. I-FCV-25-4,5	10	Containment purge air exhaust, CIS	No	5
2. I-FCV-25-2,3	11	Containment purge supply, CIS	No	5
3. I-MV-15-1	7	Primary makeup water, CIS	Yes	19
4. I-MV-18-1	9	Instrument air supply, CIS	No	28
5. V-6741	14	Nitrogen supply to safety injection tanks, CIS	Yes	5
6. I-HCV-14-1 & 7	23	Reactor coolant pump cooling water supply, SIAS	No	5
7. I-HCV-14-6 & 2	24	Reactor coolant pump cooling water return, SIAS	No	5
8. V-2515,2516	26	Letdown line, CIS, SIAS	No	5
9. V-5200,5203	28	Reactor coolant sample, CIS	Yes	5
10. V-5201,5204	29	Pressurizer surge line sample, CIS	Yes	5
11. V-5202,5205	29	Pressurizer steam space sample, CIS	Yes	5
12. V-6554,6555	31	Containment vent header, CIS	Yes	5
13. I-LCV-07-11A,11B	42	Reactor cavity sump pump discharge, CIS	Yes	10
14. V-6301,6302	43	Reactor drain tank pump suction, CIS	Yes	5
15. V-2505	44	Reactor coolant pump controlled bleedoff, CIS	No	5
16. I-SE-01-1	44	Reactor coolant pump controlled bleedoff, CIS	No	5
17. <sup>+</sup> I-HCV-18-2	8	Station air supply, CIS	Yes	5

<sup>+</sup>Item 17 to become effective upon completion of the modification described in L-87-123.

TABLE 3.6-2 (Continued)

<u>Valve Tag Number</u>	<u>Penetration Number</u>	<u>Function</u>	<u>Testable During Plant Operation</u>	<u>Isolation Time (Sec)</u>
B. MANUAL OR REMOTE MANUAL				
1. I-V-18-947 <sup>+</sup>	8	Station air supply, Manual	Yes	NA
2. I-V-25-11,12	56	Hydrogen purge outside air make-up, Manual (NC)	Yes	NA
3. I-V-25-13,14,15,16	57 & 58	Hydrogen purge exhaust, Manual (NC)	Yes	NA
4. V-3463	41	Safety injection tank test line, Manual (NC)	Yes	NA*
5. I-V-03-1307	41	Safety injection tank test line, Manual (NC)	Yes	NA*
6. V-07206, V-07189	46	Refueling cavity purification flow inlet, Manual (NC)	Yes	NA
7. V-07170, V-07188	47	Refueling cavity purification flow outlet, Manual (NC)	Yes	NA
8. I-FSE-27-1,2,3,4,8,10	48	Hydrogen sampling line, Remote manual	Yes	NA*
9. I-FSE-27-5,6,7,9,11	51	Hydrogen sampling line, Remote manual	Yes	NA*

<sup>+</sup>To become I-V-18795 upon completion of the modification described in L-87-123.

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TABLE 3.6-2 (Continued)

<u>Valve Tag Number</u>	<u>Penetration Number</u>	<u>Function</u>	<u>Testable During Plant Operation</u>	<u>Isolation Time (Sec)</u>
10. I-FCV-26-1 & 2	52a	Radiation monitoring	Yes	NA
11. I-FCV-26-3 & 4	52b	Radiation monitoring	Yes	NA
12. I-FCV-26-5 & 6	52c	Radiation monitoring, return	Yes	NA
13. I-V00140(1325) I-V00143(1325)	52d	ILRT test tap	Yes	NA
14. I-V00139(1322) I-V00144(1322)	52e	ILRT test tap	Yes	NA
15. I-V00101(612)	54	ILRT pressure connection	Yes	NA
16. I-FCV-03-1E & 1F	28	SI Tank Sample	Yes	NA**

NA - Manual Valve-Isolation time not applicable.

\* May be opened on an intermittent basis under administrative control.

\*\* Normally closed valves - Isolation time not applicable.

Amendment No. 3f



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO.95

TO FACILITY OPERATING LICENSE NO. DPR-67

FLORIDA POWER & LIGHT COMPANY

ST. LUCIE PLANT, UNIT NO. 1

DOCKET NO. 50-335

INTRODUCTION

By letter dated March 17, 1987 (L-87-123), Florida Power and Light Company (FP&L or the licensee) proposed changes to the St. Lucie Unit 1 facility Technical Specifications to enhance the station air system by adding new containment isolation valves on the station air header to containment, and by adding a breathing air aftercooler. The breathing air aftercooler is classified as non-nuclear safety-related and its addition will not affect any safety-related structures, systems, or components. These changes will permit the station air system to be also used as a supply of breathable air inside containment under any mode of plant operation.

The new containment isolation valves will have valve tag numbers different from the valve tag numbers of the presently installed containment isolation valves. Therefore, Technical Specification Tables 3.6-1 "Containment Leakage Paths" and Table 3.6-2 "Containment Isolation Valves" must be revised to reflect the new tag numbers.

EVALUATION

FP&L has planned a plant improvement modification for St. Lucie Unit 1, which will enhance the station air system by adding a manual isolation valve and a check valve inside containment and an air-operated isolation valve outside containment. The pneumatic-operated valve will normally be maintained in the closed position during normal plant operations and is designed to fail in the closed position in the event of a loss in air pressure or upon receiving a containment isolation signal.

The station air system is not considered in any accident analysis nor does it affect any other safety-related equipment. However, since one of its headers penetrate containment, containment isolation and leakage paths must be considered. The licensee stated that the modification to the plant does not involve an unreviewed safety question, and the resulting revisions to the Technical Specifications are administrative in nature. The licensee based this determination on the fact that the new containment isolation valves will be installed in

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accordance with the containment isolation criteria specified in the St. Lucie Unit 1 FSAR, and the new valves are of similar design or better than the valves presently installed. The staff does not take issue with the licensee's conclusion of no unreviewed safety question and analysis in support of it. However, the proposed changes to the tables describing containment leakage pathways and containment isolation valves are not administrative in nature. The proposed containment related changes do affect the Technical Specifications and require prior review and approval by the staff.

The staff reviewed the planned containment isolation valve configuration against General Design Criterion (GDC) 56, Primary Containment Isolation. One method to meet GDC-56 is to have one automatic isolation valve inside containment and one automatic isolation valve outside containment. A simple check valve may not be used as the automatic isolation valve outside containment.

The licensee plans to use a check valve inside containment and an air-operated isolation valve outside containment (which will close on a containment isolation signal) as automatic isolation valves. Thus, the licensee meets GDC-56, and the licensee's planned valve changes are acceptable.

As stated above, a number of changes need to be made to TS Tables 3.6-1 and 3.6-2 because some existing valves and their tags will be removed and some new valves and their tags will be added.

The proposed changes to the tables are as follows:

1. Page 3/4 6-5

Penetration 8, Valve Tag Number

Globe (I-V-18-947) should read Globe (I-HCV-18-2)

The addition of this valve meets GDC 56 as evaluated above.

Globe (I-V-18-947) should read Globe (I-V-18795)

This change corrects a generic valve designation to a plant-specific tag number.

Check (I-V-181118)

The addition of this valve meets GDC 56 as evaluated above.

Globe (I-SH-18797)

The addition of this valve to the Technical Specifications as a leakage pathway formalizes this valve as the leakage pathway.

2. Page 3/4 6-20

Penetration 8, Valve Tag Number

Globe (I-HCV-18-2)

The addition of this valve meets GDC 56 as evaluated above.

3. Page 3/4 6-21

Penetration 8, Valve Tag Number

Globe (I-V-18-947) should read Globe (I-V-18795)

This change corrects a generic valve designation to a plant-specific tag number.

Based upon the above considerations, the changes to the valve tables are acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: June 13, 1988

Principal Contributors:

T. MacArthur, RII  
E. Tourigny