

February 19, 1987

DMB o/k

Docket No. 50-335

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Mr. C. O. Woody  
Vice President  
Nuclear Energy Department  
Florida Power & Light Company  
P. O. Box 14000  
Juno Beach, Florida 33408

Dear Mr. Woody:

The Commission has issued the enclosed Amendment No. 77 to Facility Operating License No. DPR-67 for the St. Lucie Plant, Unit No. 1. This amendment consists of changes to the Technical Specifications in response to your application dated December 18, 1986.

This amendment lowers the maximum allowable power level for linear heat rate monitoring using the excore detector system from 88 percent to 85 percent when the peripheral axial shape index is between 0.02 and 0.08.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

Original signed by

E. G. Tourigny, Project Manager  
PWR Project Directorate #8  
Division of PWR Licensing-B

Enclosures:

1. Amendment No. 77 to DPR-67
2. Safety Evaluation

cc w/enclosures:

See next page

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Mr. C. O. Woody  
Florida Power & Light Company

St. Lucie Plant,

cc:

Mr. Jack Shreve  
Office of the Public Counsel  
Room 4, Holland Building  
Tallahassee, Florida 32304

Resident Inspector  
c/o U.S. NRC  
7585 S. Hwy A1A  
Jensen Beach, Florida 33457

State Planning & Development  
Clearinghouse  
Office of Planning & Budget  
Executive Office of the Governor  
The Capitol Building  
Tallahassee, Florida 32301

Harold F. Reis, Esq.  
Newman & Holtzinger  
1615 L Street, N.W.  
Washington, DC 20036

Norman A. Coll, Esq.  
McCarthy, Steel, Hector and Davis  
14th Floor, First National Bank Building  
Miami, Florida 33131

Administrator  
Department of Environmental Regulation  
Power Plant Siting Section  
State of Florida  
2600 Blair Stone Road  
Tallahassee, Florida 32301

Mr. Weldon B. Lewis, County  
Administrator  
St. Lucie County  
2300 Virginia Avenue, Room 104  
Fort Pierce, Florida 33450

Mr. Charles B. Brinkman, Manager  
Washington - Nuclear Operations  
Combustion Engineering, Inc.  
7910 Woodmont Avenue  
Bethesda, Maryland 20814

Mr. Allan Schubert, Manager  
Public Health Physicist  
Department of Health and  
Rehabilitative Services  
1323 Winewood Blvd.  
Tallahassee, Florida 32301

Regional Administrator, Region II  
U.S. Nuclear Regulatory Commission  
Executive Director for Operations  
101 Marietta Street N.W., Suite 2900  
Atlanta, Georgia 30323



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77  
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power & Light Company, (the licensee) dated December 18, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and by amending paragraph 2.C.(2) to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 77, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Ashok C. Thadani, Director  
PWR Project Directorate #8  
Division of PWR Licensing-B

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 19, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 77  
TO FACILITY OPERATING LICENSE NO. DPR-67  
DOCKET NO. 50-335

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove Page

3/4 2-4

Insert Page

3/4 2-4

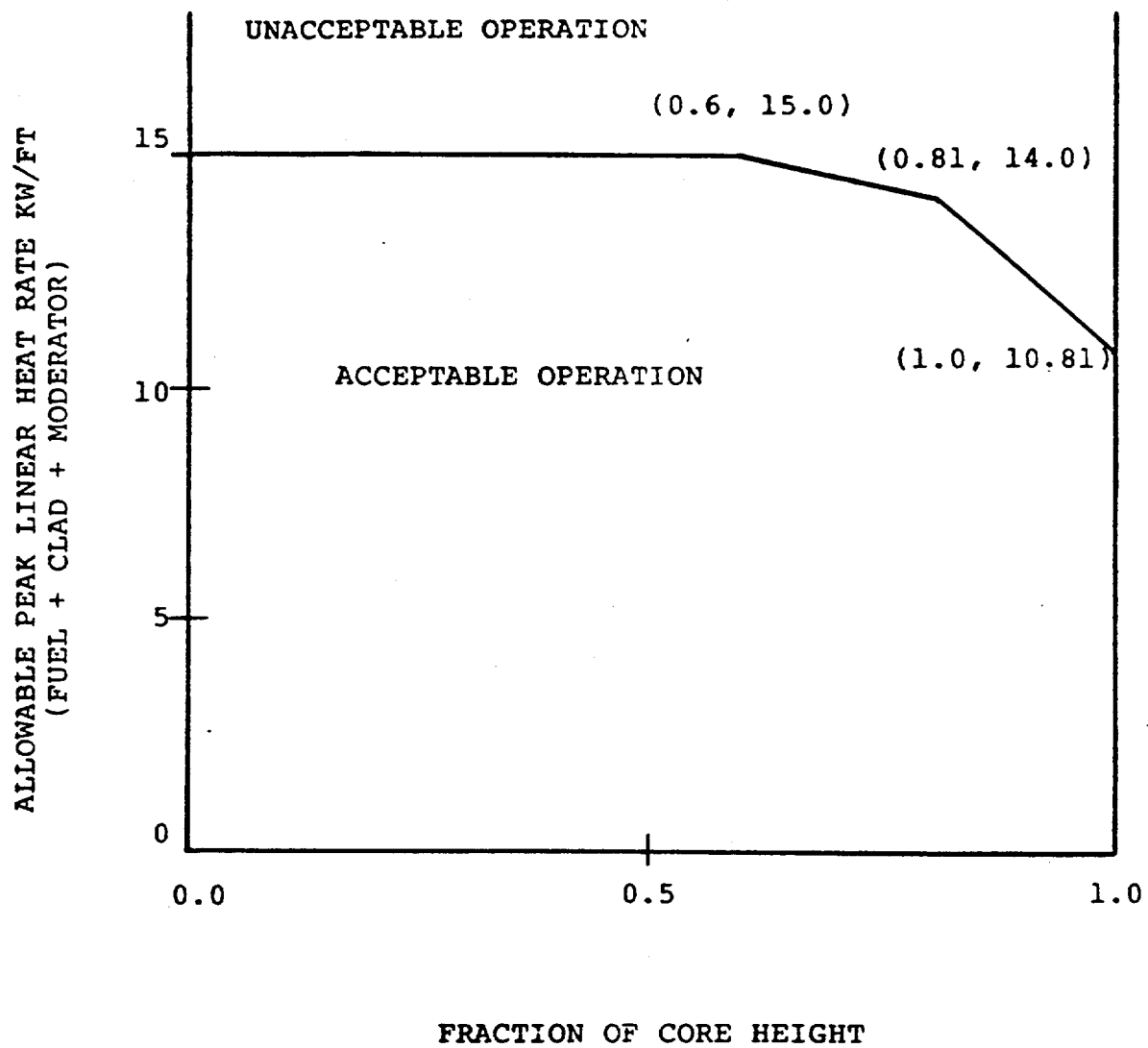


FIGURE 3.2-1 ALLOWABLE PEAK LINEAR HEAT RATE  
VERSUS FRACTION OF CORE HEIGHT

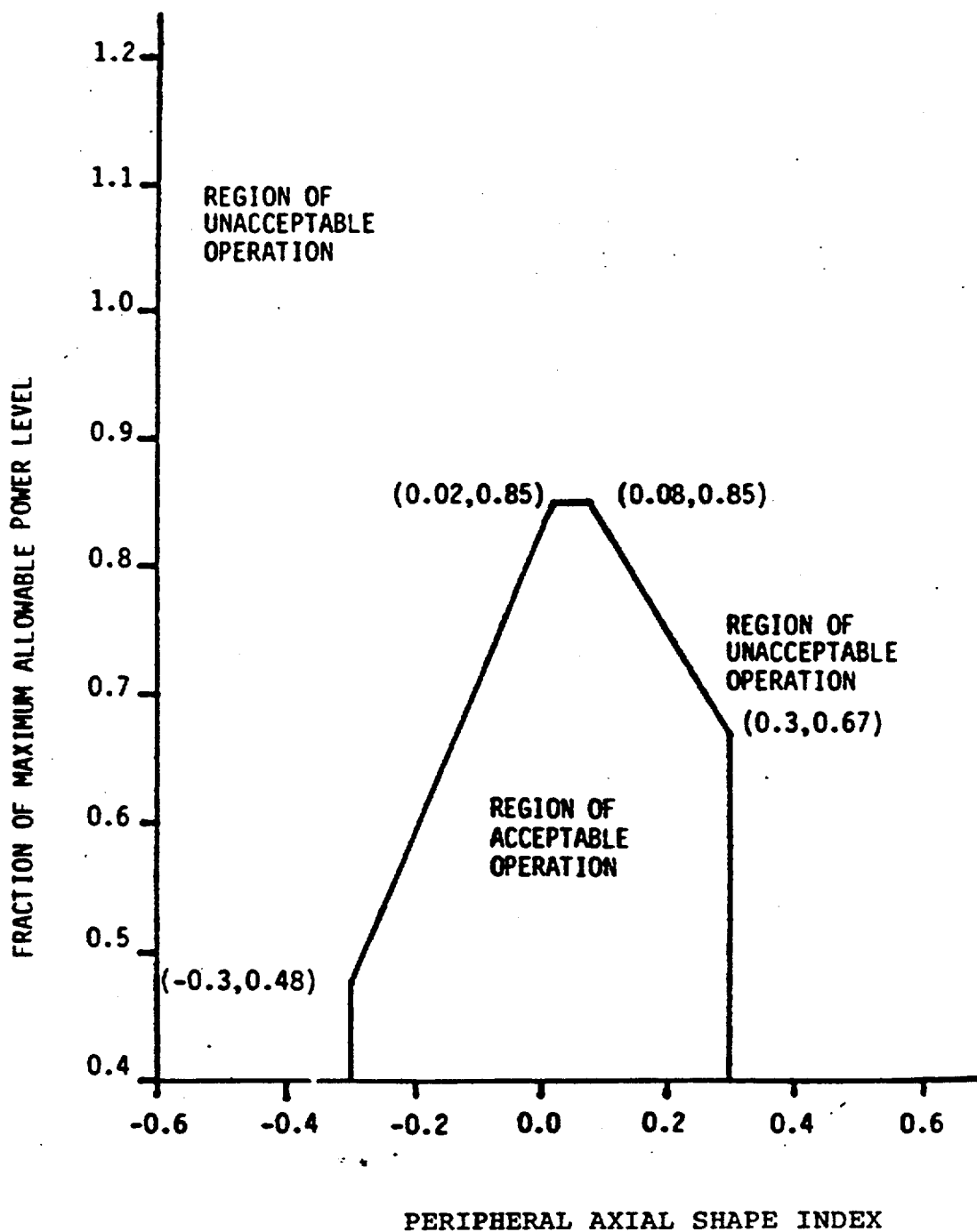


FIGURE 3.2-2

AXIAL SHAPE INDEX VS. MAXIMUM ALLOWABLE POWER  
LEVEL PER SPECIFICATION 4.2.1.3



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 77

TO FACILITY OPERATING LICENSE NO. DPR-67

FLORIDA POWER & LIGHT COMPANY

ST. LUCIE PLANT, UNIT NO. 1

DOCKET NO. 50-335

INTRODUCTION

By letter L-86-510 dated December 18, 1986, Florida Power and Light Company (FPL) submitted a request to decrease the maximum allowable power level from 0.88 to 0.85 in Figure 3.2-2, "Axial Shape Index Versus Maximum Allowable Power Level Per Specification 4.2.1.3" of the St. Lucie Unit 1 Technical Specifications. This proposed reduction in maximum allowable power level has been made to accommodate the potential for more restrictive axial power shapes in future Unit 1 fuel cycles.

EVALUATION

Either of the two core power distribution monitoring systems, the incore detector monitoring system and the excore detector monitoring system, provide adequate monitoring of the core power distribution and are capable of verifying that the linear heat rate does not exceed its limits. The limitation on linear heat rate ensures that in the event of a loss of coolant accident, the peak temperature of the fuel cladding will not exceed 2200°F. When the incore detector monitoring system becomes inoperable, monitoring of the linear heat rate is performed with the excore detector monitoring system by continuously monitoring the axial shape index (ASI) with the operable quadrant symmetric excore neutron flux detectors and verifying that the ASI is maintained within the allowable limits of Figure 3.2-2.

The proposed Figure 3.2-2 has been made more restrictive by decreasing the maximum allowable power level fraction from 0.88 to 0.85. Operation under the proposed Figure 3.2-2 meets all the NRC acceptance criteria stated in Chapter 15 of the Standard Review Plan. In addition, these analyses were performed with assumptions and methodology which have been approved by the NRC. Therefore, the staff finds the proposed change acceptable.

The staff has reviewed the proposed change to Figure 3.2-2 of the St. Lucie Unit 1 Technical Specifications. Based on the above safety evaluation, the change is acceptable.

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### ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: February 19, 1987

Principal Contributor:

L. Kopp