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Docket No. 50-335

FEBRUARY 17 1978

Florida Power & Light Company
ATTN: Dr. Robert E. Uhrig
Vice President
Nuclear & General Engineering
P. O. Box 013100
Miami, Florida 33101

Gentlemen:

The Commission has issued the enclosed Amendment No. 21 to Facility License No. DPR-67 for St. Lucie Plant Unit No. 1. The amendment consists of a revision to License No. DPR-67 in response to your request dated May 20, 1976.

The amendment revises the Technical Specifications related to shock suppressors (snubbers). Your proposed amendment has been modified and the modifications have been discussed with a representative of your staff.

Copies of the related Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original signed by
M. Grotenhuis

for Don K. Davis, Acting Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 21 to DPR-67
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures: See next page

*Const 1
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*EAK 2/3/78
modified after discussion with PDL
1/25/78 #2*

OFFICE	ORB#2:DOR	STA:DOR	OELD	ORB#2:DRO	ORB#2:DOR
SURNAME	EReeves:nm	JMcGough	W.D. Paton	DDavis	RDiggs
DATE	1/17/77	12/1/77	2/14/78	2/17/77	1/17/77

February 17, 1978

cc w/enclosures:

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660 Apalachee Parkway
Tallahassee, Florida 32304

Mr. Hamilton Oven, Jr., Administrator - w/ cy of FPL 5/20/76 filing
Florida Department of Environmental Reg.
Power Plant Siting Section
Montgomery Building
2562 Executive Center Circle
Tallahassee, Florida 32301

Mr. Weldon B. Lewis
County Administrator
St. Lucie County
Post Office Box 700
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Chief, Energy Systems Analyses
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U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N. E.
Atlanta, Georgia 30308



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 21
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company (the licensee) dated May 20, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

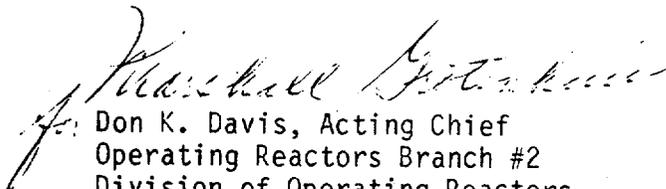
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility Operating License No. DPR-67 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 21, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Don K. Davis, Acting Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 17, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 21

FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Pages

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3/4 7-31
3/4 7-32
3/4 7-33
3/4 7-34
3/4 7-35
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PLANT SYSTEMS

3/4.7.10 HYDRAULIC SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.10.1 All hydraulic snubbers listed in Table 3.7-2 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more hydraulic snubbers inoperable, replace or restore the inoperable snubber(s) to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.10.1.1 Each hydraulic snubber with seal material fabricated from ethylene propylene or other materials demonstrated compatible with the operating environment and approved as such by the NRC, shall be determined OPERABLE at least once after not less than 4 months but within 6 months of initial criticality and in accordance with the inspection schedule of Table 4.7-3 thereafter, by a visual inspection of the snubber. Visual inspections of the snubbers shall include, but are not necessarily limited to, inspection of the hydraulic fluid reservoirs, fluid connections, and linkage connections to the piping and anchors. Initiation of the Table 4.7-3 inspection schedule shall be made assuming the unit was previously at the 6 month inspection interval.

4.7.10.1.2 Each hydraulic snubber with seal material not fabricated from ethylene propylene or other materials demonstrated compatible with the operating environment shall be determined OPERABLE at least once per 31 days by a visual inspection of the snubber. Visual inspections of the snubbers shall include, but are not necessarily limited to, inspection of the hydraulic fluid reservoirs, fluid connections, and linkage connections to the piping and anchors.

PLANT SYSTEMS

HYDRAULIC SNUBBERS (Continued)

SURVEILLANCE REQUIREMENTS (Continued)

4.7.10.1.3 At least once per 18 months during shutdown, a representative sample of at least 10 snubbers or at least 10% of all snubbers listed in Table 3.7-2, whichever is less, shall be selected and functionally tested to verify correct piston movement, lock up and bleed. Snubbers greater than 50,000 lb. capacity may be excluded from functional testing requirements. Snubbers selected for functional testing shall be selected on a rotating basis. Snubbers identified as either "Especially Difficult to Remove" or in "High Radiation Zones" may be exempted from functional testing provided these snubbers were demonstrated OPERABLE during previous functional tests. Snubbers found inoperable during functional testing shall be restored to OPERABLE status prior to resuming operation. For each snubber found inoperable during these functional tests, an additional minimum of 10% of all snubbers or 10 snubbers, whichever is less, shall also be functionally tested until no more failures are found or all snubbers have been functionally tested.

TABLE 3.7-2
SAFETY RELATED HYDRAULIC SNUBBERS*

<u>FPL LOCATION NO.</u>	<u>MARK NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
001	SS-1 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
002	SS-2 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
003	SS-3 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
004	SS-4 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
005	SS-5 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
006	SS-6 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
007	SS-7 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
008	SS-8 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
009	SS-1 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
010	SS-2 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
011	SS-3 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
012	SS-4 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
013	SS-5 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
014	SS-6 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
015	SS-7 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
016	SS-8 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes

TABLE 3.7-2 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>FPL LOCATION NO.</u>	<u>MARK NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
017	1A1	RC, RCP Motor 1A1, Elev. 57'	I	No	No
018	1A2	RC, RCP Motor 1A2, Elev. 57'	I	No	No
019	1B1	RC, RCP Motor 1B1, Elev. 57'	I	No	No
020	1B2	RC, RCP Motor 1B2, Elev. 57'	I	No	No
021	RC 005-34A	RC, Reactor Bldg, Elev. 68'	A	No	No
022	RC 005-34B	RC, Reactor Bldg, Elev. 68'	A	No	No
023	RC 005-36	RC, Reactor Bldg, Elev. 68'	A	No	No
025	RC 005-12B	RC, Reactor Bldg, Elev. 80'	A	No	No
026	RC 005-12B	RC, Reactor Bldg, Elev. 80'	A	No	No
024	RC 005-12A	RC, Reactor Bldg, Elev. 80'	A	No	No
028	RC 005-55C	RC, Reactor Bldg, Elev. 80'	A	No	No
027	RC 005-55B	RC, Reactor Bldg, Elev. 80'	A	No	No
029	RC 005-62A	RC, Reactor Bldg, Elev. 80'	A	No	No
030	RC 005-89	RC, Reactor Bldg, Elev. 80'	A	No	No
031	RC 005-90	RC, Reactor Bldg, Elev. 80'	A	No	No
032	RC 005-98	RC, Reactor Bldg, Elev. 80'	A	No	No

ST. LUCIE - UNIT 1

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Amendment No. 21

TABLE 3.7-2 (Continued)
SAFETY RELATED HYDRAULIC SNUBBERS*

<u>FPL LOCATION NO.</u>	<u>MARK NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
033	MS 649-319	MS, Reactor Bldg, Elev. 82'	A	No	No
034	MS 548-5	MS, Reactor Bldg, Elev. 82'	A	No	No
035	MS 1076-3164	MS, M.S. Trestle, Elev. 62'	A	No	No
036	MS 649-314	MS, Reactor Bldg, Elev. 55'	I	No	No
037	MS 649-314	MS, Reactor Bldg, Elev. 55'	I	No	No
038	MS 649-310	MS, Reactor Bldg, Elev. 50'	I	No	No
039	MS 649-304A	MS, Reactor Bldg, Elev. 30'	A	No	Yes
040	MS 548-9	MS, Reactor Bldg, Elev. 50'	I	No	Yes
041	MS 548-9	MS, Reactor Bldg, Elev. 50'	I	No	Yes
042	BF 549-7	BF, Reactor Bldg, Elev. 40'	I	No	No
043	BF 549-7	BF, Reactor Bldg, Elev. 40'	I	No	No
044	BF 549-8	BF, Reactor Bldg, Elev. 40'	I	No	Yes
045	BF 549-11	BF, Reactor Bldg, Elev. 50'	I	No	No
046	BF 549-11	BF, Reactor Bldg, Elev. 50'	I	No	No
047	BF 549-17	BF, Reactor Bldg, Elev. 36'	A	No	Yes
048	BF 661-407	BF, Reactor Bldg, Elev. 40'	I	No	No
049	BF 661-407	BF, Reactor Bldg, Elev. 40'	I	No	No

TABLE 3.7-2 (Continued)
SAFETY RELATED HYDRAULIC SNUBBERS*

<u>FPL LOCATION NO.</u>	<u>MARK NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
050	BF 661-416	BF, Reactor Bldg, Elev, 50'	I	No	No
051	BF 661-416	BF, Reactor Bldg, Elev, 50'	I	No	No
052	BF 549-17	BF, Reactor Bldg, Elev. 36'	A	No	No
053	SI 968-210	SI, Reactor Bldg, Elev. 16'	I	No	No
054	SI 968-565	SI, Reactor Bldg, Elev. 25'	A	No	No
055	SI 968-1205	SI, Reactor Bldg, Elev. 30'	A	No	No
056	SI 968-1207	SI, Reactor Bldg, Elev. 18'	A	No	No
057	SI 969-1190	SI, Reactor Bldg, Elev. 20'	I	No	No
058	SI 969-1216	SI, Reactor Bldg, Elev. 18'	A	No	No
059	SI 969-6193	SI, Reactor Bldg, Elev. 18'	A	No	No
060	SI 969-6195	SI, Reactor Bldg, Elev. 18'	A	No	No
061	MS 549-11	SI, Reactor Bldg, Elev. 18'	A	No	No
062	SI 969-6201	SI, Reactor Bldg, Elev. 18'	A	No	No
063	SI 969-6217	SI, Reactor Bldg, Elev. 18'	A	No	No
064	SI 969-6217	SI, Reactor Bldg, Elev. 18'	A	No	No
065	SI 970-1210	SI, Reactor Bldg, Elev. 33'	I	No	No
066	MS 549-11	SI, Reactor Bldg, Elev. 20'	I	No	No

TABLE 3.7-2 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

<u>FPL LOCATION NO.</u>	<u>MARK NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
067	SI 970-1251	SI, Reactor Bldg, Elev. 20'	A	No	No
068	SI 971-6	SI, Reactor Bldg, Elev. 20'	I	No	No
069	SI 971-1229	SI, Reactor Bldg, Elev. 20'	I	No	No
070	SI 971-6229	SI, Reactor Bldg, Elev. 20'	I	No	No
071	SI 971-6236	SI, Reactor Bldg, Elev. 20'	I	No	No
072	SI 972-1243	SI, Reactor Bldg, Elev. 25'	A	No	No
073	SI 972-6240	SI, Reactor Bldg, Elev. 16'	I	No	No
074	SI 973-240	SI, Reactor Bldg, Elev. 18'	A	No	No
075	SI 973-6219	SI, Reactor Bldg, Elev. 36'	I	No	No
076	SI 973-6224	SI, Reactor Bldg, Elev. 18'	A	No	No
077	SI 868-64	SI, RAB, Elev. 4'	A	No	No
078	SI 868-111	SI, RAB, Elev. 4'	A	No	No
079	SI 868-163	SI, RAB, Elev. 4'	A	No	No
080	SI 868-410	SI, RAB, Elev. 4'	A	No	No
081	SI 676-67	SI, RAB, Elev. 4'	A	No	No
082	SI 676-67	SI, RAB, Elev. 4'	A	No	No
083	SI 676-105	SI, RAB, Elev. 4'	A	No	No

ST. LUCIE - UNIT 1

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Amendment No. 21

TABLE 3.7-2 (Continued)
SAFETY RELATED HYDRAULIC SNUBBERS*

<u>FPL LOCATION NO.</u>	<u>MARK NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
084	SI 676-105	SI, RAB, Elev. 4'	A	No	No
085	SI 676-127	SI, RAB, Elev. 4'	A	No	No
086	SI 676-129	SI, RAB, Elev. 4'	A	No	No
087	SI 676-2481	SI, RAB, Elev. 24'	A	No	No
110	SI 676-247	SI, RAB, Elev. 30'	A	No	No
111	SI 676-2475A	SI, RAB, Elev. 30'	A	No	No
112	SI 676-4505	SI, RAB, Elev. 7'	A	No	No
113	SI 971-6236	SI, RAB, Elev. 4'	A	No	No
114	SI 972-6240	SI, RAB, Elev. 4'	A	No	No
091	SPS-417	Pressurizer Spray, Reactor Bldg, Elev, 50'	I	No	No
090	SPS-27	Pressurizer Spray, Reactor Bldg, Elev, 50'	I	No	No
092	SPS 467	Pressurizer Spray, Reactor Bldg, Elev. 80'	A	No	No
093	SPS-777	Pressurizer Spray, Reactor Bldg, Elev. 80'	A	No	No
094	CS-832-118	CS, Reactor Bldg, Elev. 125'	A	No	Yes

TABLE 3.7-2 (Continued)
SAFETY RELATED HYDRAULIC SNUBBERS*

<u>FPL LOCATION NO.</u>	<u>MARK NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
095	CS-878-115	CS, Reactor Bldg, Elev. 18'	A	No	Yes
096	CC-1865-9	CC, Reactor Bldg, Elev. 25'	A	No	No
088	CC-1899-48	CC, Reactor Bldg, Elev. 25'	A	No	No
098	CC-1899-2208	CC, Reactor Bldg, Elev. 59'	A	No	No
089	CC-1852-6241	CC, Reactor Bldg, Elev. 25'	A	No	No
100	CC-1865-2207	CC, Reactor Bldg, Elev. 59'	A	No	No
101	CC-17-1	CC, RAB, Elev. 20'	A	No	No
102	MS-649-313	CC, RAB, Elev. 26'	A	No	No
104	CC-21-1	CC, RAB, Elev. 20'	A	No	No
103	BF-549-7	CC, RAB, Elev. 26'	A	No	No
105	CC-23-2	CC, RAB, Elev. 26'	A	No	No
106	CH-3-40	CH, RAB, Elev. 34'	A	No	No
107	CH-3-75	CH, RAB, Elev. 23'	A	No	No

TABLE 3.7-2 (Continued)
SAFETY RELATED HYDRAULIC SNUBBERS*

<u>FPL LOCATION NO.</u>	<u>MARK NO.</u>	<u>SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION</u>	<u>ACCESSIBLE OR INACCESSIBLE (A or I)</u>	<u>HIGH RADIATION ZONE** (Yes or No)</u>	<u>ESPECIALLY DIFFICULT TO REMOVE (Yes or No)</u>
108	MS-649-313	MS, Reactor Bldg, Elev. 80'	I	No	No
109	MS-649-313	MS, Reactor Bldg, Elev. 80'	I	No	No
097	MS-649-314	MS, Reactor Bldg, Elev. 80'	I	No	No
099	MS-649-314	MS, Reactor Bldg, Elev. 80'	I	No	No

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-2 provided a revision to Table 3.7-2 is included with the next License Amendment request.

**Modifications to this column due to changes in high radiation areas may be made without prior License Amendment provided that a revision to Table 3.7-2 is included with the next License Amendment request.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 21 TO LICENSE NO. DPR-67

FLORIDA POWER & LIGHT COMPANY

ST. LUCIE PLANT UNIT NO. 1

DOCKET NO. 50-335

INTRODUCTION

By letter dated May 20, 1976, Florida Power & Light Company (FPL) requested an amendment to Facility License No. DPR-67 for the St. Lucie Plant Unit No. 1.

The amendment request would revise the Technical Specifications related to shock suppressors (snubbers). The amendment proposed by FPL has been modified and the modifications have been discussed with representatives of the FPL staff.

DISCUSSION AND EVALUATION

The revision proposed by FPL would revise Table 3.7-2 regarding Safety related hydraulic snubbers to: (1) add the location number for each snubber, (2) correct the Mark Number (identification number) for ten snubbers, (3) add two snubbers to the list, (4) revise the designation of one snubber to "especially difficult to remove", (5) delete two snubbers from the Table, and (6) revise a footnote to permit changes in the Table without prior license amendment.

The addition of the location numbers, correction of the Mark Numbers, and addition of two snubbers are revisions which are editorial changes to assure that the snubbers subject to surveillance are accurately identified. The changes do not affect existing surveillance requirements.

FPL has requested that one additional snubber be designated as "especially difficult to remove". The Bases associated with the snubber specifications (section 3/4.7.10) state that snubbers in especially difficult to remove locations (as identified in Table 3.7-2) may be exempted from periodic functional testing requirements provided the operability of these snubbers was demonstrated during functional testing at either the completion of their fabrication or at a subsequent date. FPL has confirmed that the snubber to be designated "especially difficult to remove" has been functionally tested. We agree that this situation justifies the designation "especially difficult to remove."

Since the change would not significantly affect FPL's ability to functionally test a representative sample of snubbers, as required by the Technical Specifications, the change will not have any significant effect on assurance of snubber operability.

FPL reported that two hydraulic type snubbers had been replaced with the mechanical type. On this basis FPL requested deletion of snubbers (Mark No. CH-3-54 and CH-3-54A) from Table 3.7-2. During a discussion with the FPL staff on December 15, 1977, we were advised that the snubbers had ratings of 1.0 and 0.25 kips, respectively and had always been the mechanical type. On this basis, the deletion of the identification numbers for mechanical snubbers from the table of hydraulic snubbers is an editorial correction and is acceptable.

FPL requested that the footnote of Table 3.7-2 which permits snubbers to be added to safety related systems without prior license amendment be revised to permit changes to Table 3.7-2 without prior license amendment provided the conditions of 10 CFR Part 50.59 are met and the table is revised in a subsequent license amendment. Since the table is an integral part of the limiting conditions for operation listing the safety related snubbers, we would not authorize deletions from the table without prior review and approval of a licensee safety analysis. Otherwise, such action may reduce the assurance that structural integrity of safety related systems would be maintained during dynamic loading. We have added a footnote allowing modifications in designation of snubbers listed in a "High Radiation Zone" due to changes in radiation levels. Such changes are allowed but must be included in the next license amendment request. Since the Technical Specifications have been established on the basis that snubbers in high radiation areas are to be considered separately from others, no prior evaluation of changes in category is considered to be necessary. We have also made minor editorial changes to the existing footnote. We have discussed the changes with representatives of FPL and they are in agreement with the change to the existing footnote and the addition of the second footnote.

We have also revised specification 3.7.10.1 and 4.7.10.1.3 to clarify our current licensing requirements. Snubbers greater than 50,000 lb. capacity would be excluded from functional test requirements due to the impracticality of on-site testing of these large snubbers.

Visual inspections are being maintained for the large snubbers. The revisions have been discussed with FPL representatives who are in agreement with the change.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

DATE: February 17, 1978

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-335FLORIDA POWER AND LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 21 to Facility Operating License No. DPR-67, issued to Florida Power & Light Company (the licensee), which revised the license and its appended Technical Specifications for operation of the St. Lucie Plant Unit No. 1 (the facility) located in St. Lucie County, Florida. The amendment is effective as of its date of issuance.

The amendment changed the Technical Specification table for shock suppressors (snubbers) to add two snubbers to the table, add location numbers, to correct identification numbers, and to make other minor changes relating to snubbers to assure that snubbers subject to surveillance are accurately identified. These changes do not alter existing surveillance requirements for snubbers.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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The Commission had determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated May 20, 1976, (2) Amendment No. 21 to License No. DPR-67, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Indian River Junior College Library, 3209 Virginia Avenue, Ft. Pierce, Florida 33450.

A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 17th day of February, 1978.

FOR THE NUCLEAR REGULATORY COMMISSION

Marshall Grotenhuis
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Operating Reactors Branch #2
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