

DCS MS-016

AUG 24 1981

Docket No. 50-335

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Dr. Robert E. Uhrig
 Vice President
 Advanced Systems & Technology
 Florida Power & Light Company
 P. O. Box 529100
 Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 42 to Facility Operating License No. DPR-67 for St. Lucie Unit No. 1. The amendment consists of a temporary change to the Technical Specifications in response to your August 3, 1981 application, telecopied to us on August 3, 1981. This amendment was authorized by telephone on August 4, 1981 and was confirmed by letter dated August 4, 1981.

This amendment modifies the Technical Specifications to permit until October 1, 1981, isolation of a primary relief line by using a de-energized shut power operated relief valve (PORV) rather than an inoperable and shut PORV block valve. This amendment was issued on an expedited basis to allow isolation of a packing leak on the inoperable PORV block valve by opening (backseating) the block valve.

Changes made to your proposed change were discussed with and agreed to by your staff. Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:

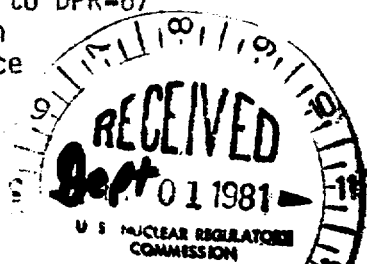
Christian C. Nelson, Project Manager
 Operating Reactors Branch #3
 Division of Licensing

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Enclosures:

1. Amendment No. 42 to DPR-67
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
 See next page



*Noted by [unclear] to form of
 revised Amendment
 or FR Note C*

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DATE	8/6/81	8/6/81	8/6/81	8/9/81	8/10/81		

Florida Power & Light Company

cc:

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Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301

Mr. Weldon B. Lewis
County Administrator
St. Lucie County
2300 Virginia Avenue, Room 104
Fort Pierce, Florida 33450

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N.E.
Atlanta, Georgia 30308

Mr. Charles B. Brinkman
Manager - Washington Nuclear Operations
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Resident Inspector/St. Lucie
Nuclear Power Station
c/o U.S.N.R.C.
P. O. Box 400
Jensen Beach, Florida 33457

cc w/enclosure(s) and incoming
dated: 8/4/81

Bureau of Intergovernmental
Relations
660 Apalachee Parkway
Tallahassee, Florida 32304



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company (the licensee) dated August 3, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provision of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License No. DPR-67 is amended by changes to the Technical Specifications as indicated in the Attachment to this license amendment, and by amending paragraph 2.C(2) to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 42, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark for

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 24, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 42

FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change.

Page

3/4 4-58

REACTOR COOLANT SYSTEM

PORV BLOCK VALVES

LIMITING CONDITION FOR OPERATION

3.4.12 Each Power Operated Relief Valve (PORV) Block Valve shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

With one or more block valve(s) inoperable, within 1 hour either restore the block valve(s) to OPERABLE status or close the block valve(s) and remove power from the block valve(s)*; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.12 Each block valve shall be demonstrated OPERABLE at least once per 92 days by operating the valve through one complete cycle of full travel.

* Until October 1, 1981, in lieu of closing and removing power to the block valve V-1403, the PORV, V-1402, may be deenergized in the closed position such that it is incapable of being opened.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 42 TO FACILITY OPERATING LICENSE NO. DPR-67

FLORIDA POWER & LIGHT COMPANY

ST. LUCIE PLANT, UNIT NO. 1

DOCKET NO. 50-335

Introduction

Technical Specification 3/4.4.12 for St. Lucie Unit No. 1 requires that the power operated relief valve (PORV) block valves be operable and tested every 92 days. On August 2, 1981 during the periodic operability check, one block valve (V-1403) failed to close when operated from the Control room. The block valve was declared inoperable, shut manually from inside containment and its power removed. This satisfied the ACTION statements of specification 3.4.12 permitting continued power operation of the reactor.

With the block valve closed, however, the valve packing developed a leak of approximately 1-1.5 gpm. To stop or greatly reduce this leak Florida Power and Light Company (FPL or the licensee) has proposed to open (backseat) the block valve, thereby isolating the valve packing from system pressure. To isolate the primary coolant relief path the associated PORV would be deenergized in the closed position. By letter dated August 3, 1981, FPC proposed a change to specification 3.4.12 which would permit isolation of the affected PORV relief line in this manner until October 1, 1981, at which time St. Lucie Unit No. 1 should be shut down for core reload.

Evaluation

The requirement for block valve operability was implemented to assure the capability to isolate a malfunctioning PORV. With the PORV deenergized in the shut position it will not open manually from the control room or automatically in response to increasing primary pressure. Therefore, with the PORV deenergized shut, there should be no need for the block valve. Actually, the proposed arrangement of block valve open and PORV deenergized shut is comparable to having the block valve shut with a malfunctioning PORV, for which continued operation is acceptable.

The pilot valve of the PORV is operated by an electric solenoid. The power source will be removed from the solenoid by lifting leads at a contact location outside containment. Precautions will be taken to insure that the correct leads have been lifted prior to opening the block valve. The lifting of leads will be independently verified. In addition FPL will attempt to open the PORV after the leads are lifted and confirm, by monitoring valve position indication which indicates solenoid position, that the solenoid does not operate.

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Once the PORV is deenergized shut and the block valve opened, flow indication parameters will continue to be monitored. If leakage is indicated immediate actions will be taken to isolate the leak (shut block valve manually inside containment if necessary) or the plant will be shut down.

Other factors considered in our review are:

1. The capability will exist for restoring power to the PORV from outside containment in the unlikely event the PORV relief path is needed to mitigate a loss of all feedwater transient.
2. There is a redundant, operable PORV to aid in mitigating overpressure transients.
3. Previous analyses have shown the plant capable of mitigating a small break LOCA via the PORV relief path without core damage.

Based on the above we conclude that the PORV, V-1402, deenergized in the shut position provides an adequate isolation of the PORV relief path and the technical specification proposed change, as modified to meet our requirements, is acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of the amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 24, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-335FLORIDA POWER & LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 42 to Facility Operating License No. DPR-67, issued to Florida Power & Light Company (the licensee), which revised the Technical Specifications for operation of the St. Lucie Plant, Unit No. 1 (the facility), located in St. Lucie County, Florida. The amendment was effective on August 4, 1981.

This amendment modifies the Technical Specifications to permit until October 1, 1981, isolation of a primary relief line by using a deenergized shut power operated relief valve (PORV) rather than an inoperable and shut PORV block valve. This amendment was issued on an expedited basis to allow isolation of a packing leak on the inoperable PORV block valve by opening (backseating) the block valve.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated August 3, 1981, (2) Amendment No. 42 to License No. DPR-67, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. and at the Indian River Junior College Library, 3209 Virginia Avenue, Ft. Pierce, Florida. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 24th day of August, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Eben L. Conner

Eben L. Conner, Acting Chief
Operating Reactors Branch #3
Division of Licensing