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Docket No. 50-335

Dr. Robert E. Uhrig
Vice President, Advanced
Systems & Technology
Florida Power & Light Company
P. O. Box 529100
Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 31 to Facility Operating License No. DPR-67 for the St. Lucie Plant, Unit No. 1. This amendment consists of changes to the Technical Specifications in response to your request dated March 10, 1978, as revised April 3 and 19, 1978, and March 8, 1979; and your request dated November 16, 1978.

This amendment revises the Technical Specifications to allow replacement of selected safety-related hydraulic snubbers with mechanical snubbers and to revise snubber surveillance requirements. This amendment also allows relocation of the sample lines for the safety injection tanks and revises the Technical Specifications to add surveillance requirements for the containment isolation valves in the new sample lines.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:
1. Amendment No. 31 to DPR-67
2. Safety Evaluation
3. Notice

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cc w/enclosures: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 5, 1979

Docket No. 50-335

Dr. Robert E. Uhrig
Vice President, Advanced
Systems & Technology
Florida Power & Light Company
P. O. Box 529100
Miami, Florida 33152

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Sincerely,

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:
1. Amendment No. 31 to DPR-67
2. Safety Evaluation
3. Notice

cc w/enclosures: See next page

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3/10/78, 4/3 & 19/78, 3/8/79 & 11/16/7
Bureau of Intergovernmental
Relations
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Tallahassee, Florida 32304

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER & LIGHT COMPANY

DOCKET NO. 50-335

ST. LUCIE PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 31
License No. DPR-67

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power & Light Company (the licensee) dated March 10, 1978, as revised April 13 and 19, 1978, and March 8, 1979, and the application for amendment dated November 16, 1978, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-67 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 31, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 5, 1979

ATTACHMENT TO LICENSE AMENDMENT NO. 31

FACILITY OPERATING LICENSE NO. DPR-67

DOCKET NO. 50-335

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 6-5
3/4 6-22
3/4 7-32
3/4 7-33
3/4 7-34
3/4 7-35
3/4 7-36
3/4 7-37

TABLE 3.6-1

CONTAINMENT LEAKAGE PATHS

Penetration	System	Valve Tag Number	Location to Containment	Service	Test Type *
7	Makeup Water	Gate (I-MV-15-1) Check (I-V-15-1347)	Outside Inside	Primary Makeup Water	Bypass
8	Station Air	Globe (I-V-18-947) Globe (I-V-18-947)	Outside Outside	Station Air Supply	Bypass
9	Instrument Air	Gate (I-MV-18-1) Check (I-V-18-957)	Outside Inside	Instrument Air Supply	Bypass
10	Containment Purge	Butterfly (I-FCV-25-4) Butterfly (I-FCV-25-5)	Inside Outside	Containment Purge Exhaust	Type C
11	Containment Purge	Butterfly (I-FCV-25-3) Butterfly (I-FCV-25-2)	Inside Outside	Containment Purge Supply	Type C
14	Waste Management	Globe (V-6741) Check (V-6779)	Outside Outside	Nitrogen supply to SI Tanks	Bypass
23	Component Cooling	Butterfly (I-HCV-14-7) Butterfly (I-HCV-14-1)	Outside Outside	RC Pump CW supply	Bypass
24	Component Cooling	Butterfly (I-HCV-14-6) Butterfly (I-HCV-14-2)	Outside Outside	RC Pump CW Return	Bypass
25	Fuel Transfer Tube	Blind Flange	Inside	Fuel Transfer	Bypass
26	CVCS	Globe (V-2515) Globe (V-2516)	Inside Inside	Letdown Line	Bypass
28	Sampling	Globe (V-5200) Globe (V-5203) Globe (I-FCV-03-1E) Globe (I-FCV-03-1F)	Outside Outside Outside Outside	Reactor Coolant Sample SI Tank Sample SI Tank Sample	Bypass Bypass

TABLE 3.6-1 (Continued)

Penetration	System	Valve Tag Number	Location to Containment	Service	Test Type*
29	Sampling	Globe (V-5202) Globe (V-5205)	Outside Outside	Pressurizer Steam Space Sample	Bypass
29	Sampling	Globe (V-5201) Globe (V-5204)	Outside Outside	Pressurizer Surge Line Sample	Bypass
31	Waste Management	Gate (V-6554) Gate (V-6555)	Outside Outside	Containment Vent Header	Bypass
41	Safety Injection Tank Test Lines	Gate (V-3463) Gate (I-V-03-1307)	Outside Outside	Safety Injection Tank Fill and Sampling	Bypass
42	Waste Management	Gate (I-LCV-07-11A) Gate (I-LCV-07-11B)	Outside Outside	Reactor Cavity Sump Pump Discharge	Bypass
43	Waste Management	Gate (V-6301) Gate (V-6302)	Outside Outside	Reactor Drain Tank Pump Suction	Bypass
44	CVCS	Gate (V-2505) Gate (I-SE-01-1)	Outside Inside	RC Pump Controlled Bleedoff	Bypass
46	Fuel Pool Cleanup	Gate (I-V-07-206) Gate (I-V-07-189)	Outside Inside	Refueling Cavity Purification Flow Inlet	Bypass
47	Fuel Pool Cleanup	Gate (I-V-07-170) Gate (I-V-07-188)	Outside Inside	Refueling Cavity Purification Flow Outlet	Bypass
48	Sampling	Globe (I-FSE-27-01,02, 03,04) Globe (I-FSE-27-08)	Inside Outside	H_2 Sampling	Type C

TABLE 3.6-2 (Continued)

Valve Tag Number	Penetration Number	Function	Testable During Plant Operation	Isolation Time (Sec)
B. MANUAL OR REMOTE MANUAL				
1. I-V-18-947	8	Station air supply, Manual	Yes	NA
2. I-V-25-11,12	56	Hydrogen purge outside air make-up, Manual (NC)	Yes	NA
3. I-V-25-13,14, 15,16	57 & 58	Hydrogen purge exhaust, Manual (NC)	Yes	NA
4. V-3463	41	Safety injection tank test line, Manual (NC)	Yes	NA*
5. I-V-03-1307	41	Safety injection tank test line, Manual (NC)	Yes	NA*
6. V-07206, V-07189	46	Refueling cavity purification flow inlet, Manual (NC)	Yes	NA
7. V-07170, V-07188	47	Refueling cavity purification flow outlet, Manual (NC)	Yes	NA
8. I-FSE-27-1,2,3, 4,8,10	48	Hydrogen sampling line, Remote manual	Yes	NA*
9. I-FSE-27-5,6,7, 9,11	51	Hydrogen sampling line, Remote manual	Yes	NA*

TABLE 3.6-2 (Continued)

Valve Tag Number	Penetration Number	Function	Testable During Plant Operation	Isolation Time (Sec)
10. I-FCV-26-1 & 2	52a	Radiation monitoring	Yes	NA
11. I-FCV-26-3 & 4	52b	Radiation monitoring	Yes	NA
12. I-FCV-26-5 & 6	52c	Radiation monitoring, return	Yes	NA
13. I-V00140(1325) I-V00143(1325)	52d	ILRT test tap	Yes	NA
14. I-V00139(1322) I-V00144(1322)	52e	ILRT test tap	Yes	NA
15. I-V00101(612)	54	ILRT pressure connection	Yes	NA
16. I-FCV-03-1E & 1F	28	SI Tank Sample	Yes	NA**

NA - Manual Valve-Isolation time not applicable.
* May be opened on an intermittent basis under administrative control.
** Normally closed valves - Isolation time not applicable.

TABLE 3.7-2

SAFETY RELATED HYDRAULIC SNUBBERS*

FPL LOCATION NO.	MARK NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
001	SS-1 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
002	SS-2 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
003	SS-3 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
004	SS-4 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
005	SS-5 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
006	SS-6 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
007	SS-7 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
008	SS-8 1A	MS, Steam Gen. 1A, Elev. 62'	I	No	Yes
009	SS-1 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
010	SS-2 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
011	SS-3 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
012	SS-4 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
013	SS-5 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
014	SS-6 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
015	SS-7 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes
016	SS-8 1B	MS, Steam Gen. 1B, Elev. 62'	I	No	Yes

TABLE 3.7-2 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

FPL LOCATION NO.	MARK NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
017	1A1	RC, RCP Motor 1A1, Elev. 57'	I	No	No
018	1A2	RC, RCP Motor 1A2, Elev. 57'	I	No	No
019	1B1	RC, RCP Motor 1B1, Elev. 57'	I	No	No
020	1B2	RC, RCP Motor 1B2, Elev. 57'	I	No	No

TABLE 3.7-2 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

FPL LOCATION NO.	MARK NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICUT TO REMOVE (Yes or No)
033	MS 649-319	MS, Reactor Bldg, Elev. 82'	A	No	No
034	MS 548-5	MS, Reactor Bldg, Elev. 82'	A	No	No
035	MS 1076-3164	MS, M.S. Trestle, Elev. 62'	A	No	No
036	MS 649-314	MS, Reactor Bldg, Elev. 55'	I	No	No
037	MS 649-314	MS, Reactor Bldg, Elev. 55'	I	No	No
038	MS 649-310	MS, Reactor Bldg, Elev. 50'	I	No	No
039	MS 649-304A	MS, Reactor Bldg, Elev. 30'	A	No	Yes
040	MS 548-9	MS, Reactor Bldg, Elev. 50'	I	No	Yes
041	MS 548-9	MS, Reactor Bldg, Elev. 50'	I	No	Yes
042	BF 549-7	BF, Reactor Bldg, Elev. 40'	I	No	No
043	BF 549-7	BF, Reactor Bldg, Elev. 40'	I	No	No
044	BF 549-8	BF, Reactor Bldg, Elev. 40'	I	No	Yes
047	BF 549-17	BF, Reactor Bldg, Elev. 36'	A	No	Yes

TABLE 3.7-2 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

FPL LOCATION NO.	MARK NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
052	BF 549-17	BF, Reactor Bldg, Elev. 36'	A	No	No
053	SI 968-210	SI, Reactor Bldg, Elev. 16'	I	No	No
058	SI 969-1216	SI, Reactor Bldg, Elev. 18'	A	No	No
061	MS 549-11	SI, Reactor Bldg, Elev. 18'	A	No	No
066	MS 549-11	SI, Reactor Bldg, Elev. 20'	I	No	No

TABLE 3.7-2 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

FPL LOCATION NO.	MARK NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
073	SI 972-6240	SI, Reactor Bldg, Elev. 16'	I	No	No
074	SI 973-240	SI, Reactor Bldg, Elev. 18'	A	No	No
076	SI 973-6224	SI, Reactor Bldg, Elev. 18'	A	No	No
077	SI 868-64	SI, RAB, Elev. 4'	A	No	No
079	SI 868-163	SI, RAB, Elev. 4'	A	No	No
080	SI 868-410	SI, RAB, Elev. 4'	A	No	No
081	SI 676-67	SI, RAB, Elev. 4'	A	No	No
082	SI 676-67	SI, RAB, Elev. 4'	A	No	No
083	SI 676-105	SI, RAB, Elev. 4'	A	No	No

TABLE 3.7-2 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

FPL LOCATION NO.	MARK NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
084	SI 676-105	SI, RAB, Elev. 4'	A	No	No
086	SI 676-129	SI, RAB, Elev. 4'	A	No	No
087	SI 676-2481	SI, RAB, Elev. 24'	A	No	No
110	SI 676-247	SI, RAB, Elev. 30'	A	No	No
111	SI 676-2475A	SI, RAB, Elev. 30'	A	No	No
112	SI 676-4505	SI, RAB, Elev. 7'	A	No	No
114	SI 972-6240	SI, RAB, Elev. 4'	A	No	No
091	SPS-417	Pressurizer Spray, Reactor Bldg, Elev, 50'	I	No	No
090	SPS-27	Pressurizer Spray, Reactor Bldg, Elev, 50'	I	No	No
092	SPS 467	Pressurizer Spray, Reactor Bldg, Elev. 80'	A	No	No
093	SPS-777	Pressurizer Spray, Reactor Bldg, Elev. 80'	A	No	No

TABLE 3.7-2 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

FPL LOCATION NO.	MARK NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULt TO REMOVE (Yes or No)
096	CC-1865-9	CC, Reactor Bldg, Elev. 25'	A	No	No
088	CC-1899-48	CC, Reactor Bldg, Elev. 25'	A	No	No
089	CC-1852-6241	CC, Reactor Bldg, Elev. 25'	A	No	No
101	CC-17-1	CC, RAB, Elev. 20'	A	No	No
102	MS-649-313	CC, RAB, Elev. 26'	A	No	No
104	CC-21-1	CC, RAB, Elev. 20'	A	No	No
103	BF-549-7	CC, RAB, Elev. 26'	A	No	No
105	CC-23-2	CC, RAB, Elev. 26'	A	No	No
106	CH-3-40	CH, RAB, Elev. 34'	A	No	No
107	CH-3-75	CH, RAB, Elev. 23'	A	No	No

TABLE 3.7-2 (Continued)

SAFETY RELATED HYDRAULIC SNUBBERS*

FPL LOCATION NO.	MARK NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
108	MS-649-313	MS, Reactor Bldg, Elev. 80'	I	No	No
109	MS-649-313	MS, Reactor Bldg, Elev. 80'	I	No	No
097	MS-649-314	MS, Reactor Bldg, Elev. 80'	I	No	No
099	MS-649-314	MS, Reactor Bldg, Elev. 80'	I	No	No

*Snubbers may be added to safety related systems without prior License Amendment to Table 3.7-2 provided a revision to Table 3.7-2 is included with the next License Amendment request.

**Modifications to this column due to changes in high radiation areas may be made without prior License Amendment provided that a revision to Table 3.7-2 is included with the next License Amendment request.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NO. DPR-67

FLORIDA POWER & LIGHT COMPANY

ST. LUCIE PLANT, UNIT NO. 1

DOCKET NO. 50-335

Introduction

By application dated March 10, 1978, as revised April 3 and 19, 1978, and March 8, 1979, Florida Power and Light Company (FPL) requested an amendment to Facility Operating License No. DPR-67 to allow the replacement of certain hydraulic snubbers with mechanical snubbers and to delete Technical Specification (TS) testing requirements for those snubbers. By application dated November 16, 1978, FPL requested an amendment to Facility Operating License No. DPR-67 to relocate sample lines for the safety injection tanks and to add TS surveillance requirements for containment isolation valves in the new sample lines.

Discussion and Evaluation

Mechanical Snubbers

Forty-two hydraulic snubbers would be replaced with mechanical snubbers. Twenty-two of the hydraulic snubbers to be replaced are rated at 10,000 inch-pounds (10 kips) and 20 are rated at 3,000 inch-pounds (3 kips). All of these snubbers are used for seismic restraints. The proposed mechanical snubbers have ratings equivalent to the corresponding hydraulic snubbers. The mechanical snubbers have an activation level of 0.02 g in both directions and will, therefore, limit seismic induced acceleration to 0.02 g. The mechanical snubbers allow less total movement than the hydraulic snubbers with a peak-to-peak axial displacement of less than 0.12 inches under a seismic load. The performance of these snubbers has been verified by a test in a working range of 3 to 33 hz. Also, reliability of these mechanical snubbers has been demonstrated through experience at other reactors.

By letter dated March 8, 1978, FPL committed to visual inspection of linkages and anchorage of the mechanical snubbers at least once every 18 months. The NRC is developing Standard Technical Specifications (STS) for mechanical snubber testing and surveillance. During 1979 FPL will be requested to propose surveillance TS for mechanical snubbers that are consistent with STS. We have determined that the inspection commitment as stated in FPL letter of March 8, 1978, will provide adequate assurance of mechanical snubber operability in the interim until STS surveillance requirements

have been established because of the demonstrated performance and reliability of the proposed snubbers.

Therefore, the use of mechanical snubbers as proposed by FPL will not result in any decrease in safety margin or result in any increase in accident consequences or an introduction of any new accidents.

Safety Injection Tank Sample Line Relocation

Each safety injection tank sample line presently terminates in a sample sink inside the reactor containment building. Therefore, there are considerable man hours and significant radiation exposure associated with containment entries to take samples. The proposed change involves routing these sample lines through electrically operated valves to a manifold. A single line would then go through an existing containment penetration to the reactor auxiliary building.

Penetration through containment will be through two isolation valves which will both be normally closed. These isolation valves will fail closed on loss of power and also close on a containment isolation signal. Therefore, no single failure of the proposed system would jeopardize plant safety. Any sampling line failure inside containment would be detected by a low level indication on a safety injection tank or by increased flow rate to the sump. Line size would limit the maximum leakage rate to 1.5 gpm for any sampling line failure. A single active failure in one of the containment isolation valves would not cause a leakage path since the other isolation valve would be closed. The containment isolation valves are added to the TS surveillance requirements by the proposed change. We have determined that the proposed surveillance requirements are adequate to assure isolation valve operability.

Therefore, the proposed change to the sampling lines and the additional surveillance requirements will not result in any decrease in safety margin, any increase in accident consequences, or introduction of any new accidents. In addition, the proposed action will result in a decrease in potential radiation exposure to employees.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 5, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-335

FLORIDA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 31 to Facility Operating License No. DPR-67 issued to Florida Power & Light Company (the licensee), which revised Technical Specifications for operation of St. Lucie Plant, Unit No. 1 (the facility), located in St. Lucie County, Florida. The amendment is effective as of its date of issuance.

This amendment revises the Technical Specifications to allow replacement of selected safety-related hydraulic snubbers with mechanical snubbers and to revise snubber surveillance requirements. The amendment also allows relocation of sample lines for the safety injection tanks and revises the Technical Specifications to add surveillance requirements for the containment isolation valves in the new sample lines.

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated March 10, 1978, as revised April 3 and 19, 1978, and March 8, 1979, and the application for amendment dated November 16, 1978, (2) Amendment No. 31 to License No. DPR-67, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Indian River Junior College Library, 3209 Virginia Avenue, Ft. Pierce, Florida. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 5th day of April 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors