October 22, 1987

Docket No. 50-339

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Mr. W. L. Stewart Vice President - Nuclear Operations

Richmond, Virginia 23261

Virginia Electric and Power Company Post Office Box 26666

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JPartlow

Dear Mr. Stewart:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 65496)

The Commission has issued the enclosed Amendment No. 82 to Facility Operating License No. NPF-7 for the North Anna Power Station, Unit No. 2 (NA-2). The amendment revises the Technical Specifications (TS) in partial response to your letter dated May 27, 1987.

The amendment revises NA-2 TS 3/4.6.3, Table 3.6-1 to correct an inconsistency between your response to NUREG 0737 dated December 10, 1980 and the NA-2 TS for specifying the containment isolation signal to be Phase B instead of Phase A.

Your May 27, 1987 letter also requested a change to allow the placing of the hydrogen recombiner containment isolation valves for NA-1&2 under administrative control in order to permit functional testing in Modes 1 through 4 at six month intervals. Our review of this matter is continuing at this time and issuance of the appropriate amendments is being deferred to a later date. We will keep you informed on this matter.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Leon B. Engle, Project Manager Project Directorate II-2 Division of Reactor Projects-I/II Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 82 to NPF-7

Safety Evaluation

cc w/enclosures: See next page

\*See previous concurrence

\*LA:PD22 DMiller

10/15/87

\*PM:PD22 LEngle:bd

10/10/87

\*PSB JWermeil

10/15/87

\*OGC MYoung 10/16/87



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

October 22, 1987

Docket No. 50-339

Mr. W. L. Stewart
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Dear Mr. Stewart:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 65496)

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Leon B. Engle, Project Manager

Project Directorate II-2

Division of Reactor Projects-I/II Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 82 to NPF-7

2. Safety Evaluation

cc w/enclosures:
See next page

Mr. W. L. Stewart Virginia Electric & Power Company

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Atomic Safety and Licensing Appeal Board Panel U.S. Nuclear Regulatory Commission Washington, DC 20555

Regional Administrator, Region II U.S. Nuclear Regulatory Commission Office of Executive Director for Operations 101 Marietta Street N.W., Suite 2900 Atlanta, Georgia 30323

Mr. E. W. Harrell P. O. Box 402 Mineral, Virginia 23117

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Mr. William C. Porter, Jr. County Administrator Louisa County P. O. Box 160 Louisa, Virginia 23093

James B. Kenley, M.D., Commissioner Department of Health 109 Governor Street Richmond, Virginia 23219



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

## VIRGINIA ELECTRIC AND POWER COMPANY

## OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

# AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 82 License No. NPF-7

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated May 27, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

## (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 82, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Hembert N. Berkow, Director Project Directorate II-2

Division of Reactor Projects-I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: October 22, 1987

# ATTACHMENT TO LICENSE AMENDMENT NO. 82

## TO FACILITY OPERATING LICENSE NO. NPF-7

# DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

## Page

3/4 6-20

3/4 6-22

# TABLE 3.6-1 (Cont.)

			•
	VALVE NUMBER	FUNCTION	MAXIMUM ISOLATION TIME (SEC.)
47.	TV-CV250A	Containment Vacuum Pump Suction	60
48.	TV-CV250B	Containment Vacuum Pump Suction	60
49.	TV-CV250C	Containment Vacuum Pump Suction	60
50.	TV-CV250D	Containment Vacuum Pump Suction	60
51.	TV-SS203A	Residual Heat Removal System Sample Lines	60
52.	TV-SS203B	Residual Heat Removal System Sample Lines	60
53.	TV-LM201A	Reactor Containment Leakage Monitoring Lines to Reference System	60
54.	TV-LM201B	Reactor Containment Leakage Monitoring Lines to Reference System	60
55.	TV-LM201C	Reactor Containment Leakage Monitoring Lines to Reference System	60
56.	TV-LM201D	Reactor Containment Leakage Monitoring Lines to Reference System	60
57.	TV-2859	Safety Injection Test Line	10
58.	TV-2842	Safety Injection Test Line	10
59.	TV-SS212A	Steam Generator Surface Sample	60
60.	TV-SS212B	Steam Generator Surface Sample	60
61,.	TV-MS209#	Main Steam Drains to Condenser	60
62.	TV-MS210#	Main Steam to Blowdown	60
63.	TV-SV202-2#	Condenser Air Ejector Vent	60
64.	FCV-AS200A#	Condenser Air Ejector Steam Supply	60
65.	FCV-AS200B#	Condenser Air Ejector Steam Supply	60

# TABLE 3.6-1 (Cont.)

	VALVE NUMBER	FUNCTION	MAXIMUM ISOLATION TIME (SEC)
66.	TV-1A201A	Containment Instrument Air Supply	60
67.	TV-1A201B	Containment Instrument Air Supply	60
68.	DELETED		60
69.	DELETED		60
70.	TV-DA203A	Post Accident Sample System Containment Return Line	60
71.	TV-DA2O3B	Post Accident Sample System Containment Return Line	60
В.	PHASE "B" ISOLATION	<b>,</b>	
1.	TV-CC203A	Component Cooling Water from RHR System and Excess Letdown Heat Exchanger	60
2.	TV-CC203B	Component Cooling Water From RHR System and Excess Letdown Heat Exchanger	60
3.	TV-CC201A	Reactor Coolant Pump Thermal Barrier Cooling Water Return	60
4.	TV-CC201B	Reactor Coolant Pump Thermal Barrier Cooling Water Return	60

# <u>TABLE 3.6-1</u> (Cont.)

VALVE NUMBER		FUNCTION	MAXIMUM ISOLATION TIME (SEC.)
5.	TV-CC200A	Chilled Water From Containment Air Recir- culation Coils	60
6.	TV-CC200B	Chilled Water From Containment Air Recir- culation Coils	60
7.	TV-CC200C	Chilled Water From Containment Air Recir- culation Coils	60
8.	TV-CC205A	Chilled Water From Containment Air Recir- culation Coils	60
9.	TV-CC205B	Chilled Water From Containment Air Recir- culation Coils	60
10.	TV-CC205C	Chilled Water From Containment Air Recir- culation Coils	60
11.	TV-CC204A	Reactor Coolant Pumps, Cooling Water In	60
12.	TV-CC204B	Reactor Coolant Pumps, Cooling Water In	60
13.	TV-CC204C	Reactor Coolant Pumps, Cooling Water In	60
14.	TV-CC202A	Reactor Coolant Pumps and Shroud Cooling	60
		Cooling Water Out	
15.	TV-CC202B	Reactor Coolant Pumps and Shroud Cooling, Cooling Water Out	60
16.	TV-CC202C	Reactor Coolant Pumps and Shroud Cooling, Cooling Water Out	60

# TABLE 3.6-1 (Cont.)

NORTH ANNA		VALVE NUMBER	<u>FUNCTION</u>	MAXIMUM ISOLATION TIME (SEC.)
ı	17	. TV-CC202D	Reactor Coolant Pumps and Shroud Cooling, Cooling Water Out	60
UNIT 2	18	. TV-CC202E	Reactor Coolant Pumps and Shroud Cooling, Cooling Water Out	60
	19	. TV-CC202F	Reactor Coolant Pumps and Shroud Cooling, Cooling Water Out	60
	20	. TV-BD200A	Steam Generator Blowdown	60
	21	. TV-BD200B	Steam Generator Blowdown	60
(.)	22.	TV-BD200C	Steam Generator Blowdown	60
3/4	23.	TV-BD200D	Steam Generator Blowdown	60
6-22	24.	TV-BD200E	Steam Generator Blowdown	60
	25.	TV-BD200F	Steam Generator Blowdown	60
	26.	TV-1A202A	Instrument Air to Reactor Containment	60
	27.	TV-IA202B#	Instrument Air to Reactor Containment	60
·>	C. CON	TAINMENT PURGE AND E)	(HAUST (VENTILATION DUCTS)	
îmendmen+	1.	MOV-HV200A*	Purge Supply	NA
<b>9</b>	2.	MOV-HV200B*	Purge Supply	NA
5	3.	MQV-HV202*	Alternate Supply	, NA
82	4.	MOV-HV200C*	Purge Exhaust	NA NA
	5.	MOV-HV200D*	Purge Exhaust	NA NA
	6.	MOV-HV201*	Bypass	NA NA



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

### SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 82TO

FACILITY OPERATING LICENSE NO. NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

NORTH ANNA POWER STATION, UNIT NO. NO. 2

DOCKET NO. 50-339

### INTRODUCTION

By letter dated May 27, 1987, the Virginia Electric and Power Company (the licensee) requested a change to the North Anna Power Station, Units No. 1 and No. 2 (NA-1 & 2) Technical Specifications (TS). The amendment is in partial response to the licensee's request. The proposed change would modify the NA-2 TS 3/4.6.3, Table 3.6-1 by correcting an inconsistency between the licensee's response to NUREG 0737 and the NA-2 TS with regard to the isolation signal for the containment instrument air system. At present, instrument air isolates on a Phase B signal for NA-1 and a Phase A signal for NA-2. This request will change the containment isolation signal for NA-2 from Phase A to Phase B in compliance with NUREG 0737.

Our discussion is provided below.

#### DISCUSSION:

TS 3/4.6.3, Table 3.6.1 for NA-2 lists the Containment Isolation Valves for containment instrument air supply (TV-IA 202A and TV-IA 202B) as shutting on receipt of a Phase A containment isolation signal. Although the licensee's letter dated October 24, 1979 designated NA-1 containment instrument air as an essential Level 2 system which isolates on a Phase B signal, the similar letter for NA-2 dated October 25, 1979 makes no mention of the containment air system. The licensee's response to NUREG 0737, Rev. 0 dated December 10, 1980 lists containment instrument air as a Phase B isolation system for both units, indicating that the omission of mention in the NA-2 letter was an oversight. Therefore, the licensee has proposed to change the containment isolation signal from Phase A to Phase B to correct this oversight and bring NA-2 instrument air containment isolation signal into compliance with NUREG 0737 and provide consistency with the homologous valves in NA-1.

#### **EVALUATION**

As discussed above, the proposed change would change the NA-2 containment instrument air supply valves from Phase A containment isolation to Phase B containment isolation. The acceptability of Phase B isolation for the containment instrument air supply for NA-1&2 is documented in the NRC SE dated January 12, 1984. The revised designation would bring NA-2 into compliance with the licensee's response to NUREG 0737, Section II.E.4.2 and provide consistency with the NA-1 valves. Therefore, we find this change to be acceptable.

#### **ENVIRONMENTAL CONSIDERATION**

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 22, 1987

Principal Contributor:

Leon Engle