

December 6, 1988

Docket No. 50-339

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Mr. W. R. Cartwright
Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. Cartwright:

SUBJECT: NORTH ANNA UNIT 2 - CORRECTION TO AMENDMENT NO. 92
ISSUED NOVEMBER 10, 1988

On November 10, 1988, the Commission issued Amendment Nos. 106 and 92 to Facility Operating License Nos. NPF-4 and NPF-7, respectively, for the North Anna Power Station, Unit Nos. 1 and 2 (NA-1&2). These amendments modified the requirement to conduct a Type A test at 40 ± 10 month intervals during each 10-year inservice inspection (ISI) and allow the third Type A test of the first 10-year service interval to take place during the NA-1&2 1989/10-year inservice inspection outages.

By letter dated November 23, 1988, you informed us that your submittal dated October 6, 1988, incorrectly referred to the the current Type A test interval for NA-2 as the third interval test to be conducted during the 10-year inservice inspection outage. You further stated that the current interval for NA-2 is the second interval with the second interval Type A test scheduled for the NA-2 1989 refueling outage. Therefore, the correct NA-2 TS should change the reference from the third test to the second test and delete the reference to the 10-year ISI outage in the footnote.

Enclosed is the corrected NA-2 TS page 3/4 6-2 as well as the corresponding over-leaf page to be inserted into the NA-2 TS.

The staff has determined that the correction of this error does not change the staff's evaluation and conclusion which supported the changes to the NA-1&2 Technical Specifications.

Sincerely,

Original Signed by

Leon B. Engle, Project Manager
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

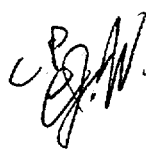
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Mr. W. R. Cartwright
Virginia Electric & Power Company

North Anna Power Station
Units 1 and 2

cc:

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3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

- a. At least once per 31 days by verifying that all penetrations* not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.3.1., and
- b. By verifying that each containment air lock is OPERABLE per Specification 3.6.1.3.
- c. After each closing of the equipment hatch, by leak rate testing the equipment hatch seals with gas at Pa, greater than or equal to 40.6 psig, and verifying that when the measured leakage rate for these seals is added to the leakage rates determined pursuant to Specification 4.6.1.2.d for all other Type B and C penetrations, the combined leakage rate is less than or equal to 0.60 La.

* Except valves, blind flanges and deactivated automatic valves which are located inside the containment and are locked sealed or otherwise sealed in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days.

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CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

- a. An overall integrated leakage rate of:
 1. Less than or equal to L_a , 0.1 percent by weight of the containment air per 24 hours at P_a , greater than or equal to 40.6 psig, or
- b. A combined leakage rate of less than or equal to $0.60 L_a$ for all penetrations and valves subject to Type B and C tests, when pressurized to P_a , greater than or equal to 40.6 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75 L_a$ or (b) with the measured combined leakage rate for all penetrations and valves subject to Type B and C tests exceeding $0.60 L_a$, restore the overall integrated leakage rate to less than $0.75 L_a$ and the combined leakage rate for all penetrations subject to Type B and C tests to less than or equal to $0.60 L_a$ prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972:

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at P_a greater than or equal to 40.6 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.*

*The second test of the first 10-year service period shall be conducted during the 1989 Refueling Outage.

DATED: December 6, 1988

CORRECTION TO AMENDMENT NO. 92 TO FACILITY OPERATING LICENSE NO. NPF-7
ISSUED NOVEMBER 10, 1988

Docket File

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