

July 7, 1989

Docket No. 50-339

DISTRIBUTION
See attached sheet

Mr. W. R. Cartwright
Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. Cartwright:

SUBJECT: NORTH ANNA UNIT 2 - ISSUANCE OF AMENDMENT RE: CONTAINMENT PENETRATION
CONDUCTOR OVERCURRENT PROTECTIVE DEVICES (TAC NO. 71135)

The Commission has issued the enclosed Amendment No. 101 to Facility Operating License No. NPF-7 for the North Anna Power Station, Unit No. 2 (NA-2). The amendment revises the Technical Specifications (TS) in response to your letter dated February 23, 1989.

This amendment modifies TS Table 3.8-1 by deleting components from the table which had previously been removed and by correcting one typographical error.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Leon B. Engle, Project Manager
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 101 to NPF-7
2. Safety Evaluation

cc w/enclosures:
See next page

LA: PDM 2
DME: 11er
6/27/89

PM: PDM 2
LEng 1e/jd
6/27/89

D: PDM 2
HBerkow
6/28/89

OGC
Bm Boudrick
6/30/89

Mr. W. R. Cartwright
Virginia Electric & Power Company

North Anna Power Station
Units 1 and 2

cc:

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Old Dominion Electric Cooperative
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Resident Inspector/North Anna
c/o U.S. NRC
Senior Resident Inspector
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Mineral, Virginia 23117

DATED: July 7, 1989

AMENDMENT NO. 101 TO FACILITY OPERATING LICENSE NO. NPF-7-NORTH ANNA UNIT 2

Docket File

NRC & Local PDRs

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ARM/LFMB

M. Sinkule, R-II

Others as required

cc: Plant Service list

Ofo
1/1



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 101
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated February 23, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

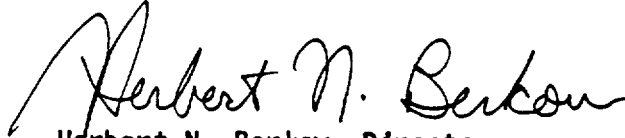
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 101 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 7, 1989

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

2. By verifying the OPERABILITY of molded case circuit breakers, by selecting and functionally testing a representative sample of at least 10% of all the circuit breakers of that type. Circuit breakers selected for functional testing shall be selected on a rotating basis. The functional test shall consist of injecting a current input at the specified setpoint to each selected circuit breaker and verifying that each circuit breaker functions as designed. Circuit breakers found inoperable during functional testing shall be restored to OPERABLE status prior to resuming operation. For each circuit breaker found inoperable during these functional tests, an additional representative sample of at least 10% of all the circuit breakers of the inoperable type shall also be functionally tested until no more failures are found or all circuit breakers of that type have been functionally tested.
 3. By verifying the OPERABILITY of fuses, by selecting and functionally testing a representative sample of each type of fuse on a rotating basis. Each representative sample of fuses shall include at least 10% of all fuses of that type. The functional test shall consist of a non-destructive resistance measurement test which demonstrates that the fuse meets its manufacturer's design criteria. Fuses found inoperable during these functional tests shall be replaced with OPERABLE fuses prior to resuming operation. For each fuse found inoperable during these functional tests, an additional representative sample of at least 10% of all fuses of that type shall be functionally tested until no more failures are found or all fuses of that type have been functionally tested.
- b. At least once per 60 months by subjecting each circuit breaker to an inspection and preventive maintenance in accordance with procedures prepared in conjunction with its manufacturer's recommendations.

TABLE 3.8-1 (Continued)

CONTAINMENT PENETRATION CONDUCTOR

OVERCURRENT PROTECTIVE DEVICES

<u>Device Number and Location (BK NO)</u>	<u>Nominal Trip Setpoint (Amperes)</u>	<u>Nominal Response Time (Sec)</u>	<u>System Powered (Mark No)</u>
2B1-2A1L	800	0.060	RHR Cub. Pwr. Recepts.
2B1-2A2R	800	0.060	Loop 2 Pwr. Recepts.
2B1-2A2L	500	0.060	Incore Inst. Drive B
2B1-2A3	2000	0.060	React. Cont. Crane
2B1-2C2	250	0.060	MOV-CC-200B
2B1-2E3	250	0.060	2-HV-F-03B
2B1-2E4	250	0.060	2-RC-P-01B1
2B1-2E5	250	0.060	2-DA-P-04B
2B1-2F2	500	0.060	2-RC-P-01B2
2B1-2F4	250	0.060	2-DG-P-01A
2C1-1A3L	800	0.060	Cont. Elevator
2C1-1B3R	500	0.060	Incore Inst. Drive C
2C1-1B3L	500	0.060	Monorail Fdr.
2C1-1C2	500	0.060	2-RC-P-01C2
2C1-1D3	250	0.060	2-RC-P-01C1
2C1-1E3	250	0.060	2-DA-P-05
2C1-1E4	250	0.060	2-DG-P-01B
2C1-1A2L	800	0.060	Loop 3 Pwr. Recepts.
2C1-4A1	250	0.060	2-NS-P-01A
2C2-1A2R	800	0.060	Port Crane Recepts.
2C2-1C1	250	0.060	2-NS-P-01B
2H1-2NA1	2000	0.060	2-HV-F-37A
2H1-2NA3	500	0.060	2-IA-C-02A
2H1-2NB1	2000	0.060	2-HV-F-37B
2H1-2NC4	2000	0.060	2-HV-F-37C
2H1-2NH4	500	0.060	MOV-2865B
2H1-2NK1	500	0.060	2-IA-C-02A
2H1-2NL3	500	0.060	MOV-2865A
2H1-2SB4	250	0.060	MOV-2536
2H1-2SC1	500	0.060	MOV-2720A
2H1-2SD1	250	0.060	MOV-2700

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 8-18

3/4 8-19



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 101 TO
FACILITY OPERATING LICENSE NO. NPF-7
VIRGINIA ELECTRIC AND POWER COMPANY
OLD DOMINION ELECTRIC COOPERATIVE
NORTH ANNA POWER STATION, UNIT NO. 2

DOCKET NO. 50-339

INTRODUCTION

By letter dated February 23, 1989, the Virginia Electric and Power Company (the licensee) requested changes to the Technical Specifications (TS) for the North Anna Power Station, Unit 2 (NA-2). The proposed changes would modify TS Table 3.8-1 by deleting components from the table which had previously been removed, and by correcting one typographical error.

DISCUSSION AND EVALUATION

TS Section 3.8.2.5, Table 3.8-1, specifies the trip point for the containment penetration conductor overcurrent protective devices. Included in this list are the steam generator support heaters, which were removed from the NA-2 TS by Amendment No. 40 dated June 7, 1984. The licensee proposes to delete the support heaters from Table 3.8-1. The proposed change is to achieve consistency and is therefore acceptable.

In addition, the licensee also proposes to correct a typographical error in Table 3.8-1. Currently, the "System Powered (Mark No.)" for the "Device Number and Location (Breaker No.)," 2C1-1A2L, is shown as "2-DA-P-01B." However, the correct reference should be "2-DG-P-01B." The proposed correction is acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental assessment need be prepared in connection with the issuance of this amendment.

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CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: July 7, 1989

Principal Contributor:
L. Engle