



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

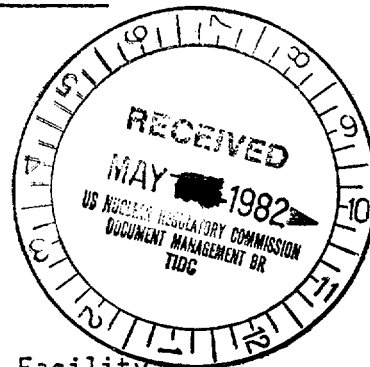
Docket File
FOR
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MAY 17 1982

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Docket No. 50-339

Mr. R. H. Leasburg
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261



Dear Mr. Leasburg:

The Commission has issued the enclosed Amendment No. 25 to Facility Operating License No. NPF-7 for the North Anna Power Station, Unit No. 2 (NA-2). The amendment revised the NA-2 Technical Specifications in response to your letter dated April 23, 1982 (Serial No. 257) and in our discussions with you regarding this matter. The amendment is effective as of its date of issuance.

The changes as requested in your April 23, 1982 request would upgrade and add twenty (20) containment isolation valves to Table 3.6.1 for NA-2 in order to meet the requirements of NUREG-0737, Action Items II.B.2 and II.B.3 for Post-Accident Shielding and Post Accident Sampling, respectively.

The additions to Table 3.6.1 include two (2) Phase A direct-acting solenoid valves, two (2) Phase A air-operated trip valves, and sixteen (16) remote manual operated valves.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Leon B. Engle
Leon B. Engle, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:

1. Amendment No. 25 to NPF-7
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

Please note:

Backup TS pages #
3/4 6-20, 3/4 6-24, 3/4 6-26,
3/4 6-27a, 3/4 6-29, + 3/4 6-31
have been resubmitted because
they were omitted in original copy.

Virginia Electric and Power Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 25
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:

- A. The application for amendment by Virginia Electric and Power Company (the licensee) dated April 23, 1982 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
- C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
- D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

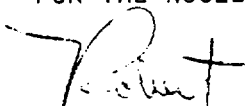
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. 25 NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 25, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 17, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Add the following pages to the Appendix "A" Technical Specifications as indicated. The new pages are identified by Amendment number and contain vertical lines indicating the area of change.

Pages

3/4 6-19
3/4 6-20
3/4 6-23
3/4 6-26
3/4 6-27
3/4 6-27a
3/4 6-28
3/4 6-29
3/4 6-30
3/4 6-31
3/4 6-32

TABLE 3.6-1 (Cont.)

| | <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-----|-------------------------|---|--|
| 47. | TV-CV250A | Containment Vacuum Pump Suction | 60 |
| 48. | TV-CV250B | Containment Vacuum Pump Suction | 60 |
| 49. | TV-CV250C | Containment Vacuum Pump Suction | 60 |
| 50. | TV-CV250D | Containment Vacuum Pump Suction | 60 |
| 51. | TV-SS203A | Residual Heat Removal System Sample Lines | 60 |
| 52. | TV-SS203B | Residual Heat Removal System Sample Lines | 60 |
| 53. | TV-LM201A | Reactor Containment Leakage Monitoring Lines to Reference System | 60 |
| 54. | TV-LM201B | Reactor Containment Leakage Monitoring Lines to Reference System | 60 |
| 55. | TV-LM201C | Reactor Containment Leakage Monitoring Lines to Reference System | 60 |
| 56. | TV-LM201D | Reactor Containment Leakage Monitoring Lines to Reference System | 60 |
| 57. | TV-2859 | Safety Injection Test Line | 10 |
| 58. | TV-2842 | Safety Injection Test Line | 10 |
| 59. | TV-SS212A | Steam Generator Surface Sample | 60 |
| 60. | TV-SS212B | Steam Generator Surface Sample | 60 |
| 61. | TV-MS209# | Main Steam Drains to Condenser | 60 |
| 62. | TV-MS210# | Main Steam to Blowdown | 60 |
| 63. | TV-SV202-2# | Condenser Air Ejector Vent | 60 |
| 64. | FCV-AS200A# | Condenser Air Ejector Steam Supply | 60 |
| 65. | FCV-AS200B# | Condenser Air Ejector Steam Supply | 60 |

TABLE 3.6-1 (Cont.)

| | <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC)</u> |
|------------------------|-------------------------|--|---|
| 66. | TV-IA201A | Containment Instrument Air Supply | 60 |
| 67. | TV-IA201B | Containment Instrument Air Supply | 60 |
| 68. | TV-IA202A | Containment Instrument Air Return | 60 |
| 69. | TV-IA202B# | Containment Instrument Air Return | 60 |
| 70. | TV-DA203A | Post Accident Sample System Containment Return Line | 60 |
| 71. | TV-DA203B | Post Accident Sample System Containment Return Line | 60 |
| B. PHASE "B" ISOLATION | | | |
| 1. | TV-CC203A | Component Cooling Water from RHR System and Excess Letdown Heat Exchanger | 60 |
| 2. | TV-CC203B | Component Cooling Water From RHR System and Excess Letdown Heat Exchanger | 60 |
| 3. | TV-CC201A | Reactor Coolant Pump Thermal Barrier Cooling Water Return | 60 |
| 4. | TV-CC201B | Reactor Coolant Pump Thermal Barrier Cooling Water Return | 60 |

TABLE 3.6-1 (cont.)

| | <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-----|-------------------------|---|--|
| D. | MANUAL | | |
| 1. | 2-SI-47* | Safety Injection Accumulator Make Up | NA |
| 2. | 2-RH-38* | Residual Heat Removal System to Refueling Water Storage Tank | NA |
| 3. | 2-RH-37* | Residual Heat Removal System to Refueling Water Storage Tank | NA |
| 4. | | (Deleted) | |
| 5. | | (Deleted) | |
| 6. | | (Deleted) | |
| 7. | | (Deleted) | |
| 8. | 2-DA-7* | Primary Vent Pot Vent | NA |
| 9. | 2-DA-9* | Primary Vent Pot Vent | NA |
| 10. | 2-CH-233#* | Reactor Coolant Pump Seal Water Supply | NA |
| 11. | 2-CH-237#* | Reactor Coolant Pump Seal Water Supply | NA |

TABLE 3.6-1 (Cont.)

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-------------------------|--|--|
| 12. 2-CH-241#* | Reactor Coolant Pump Seal Water Supply | NA |
| 13. 2-SA-65* | Service Air | NA |
| 14. 2-SA-123* | Service Air | NA |
| 15. (Deleted) | | |
| 16. NA*# | Fuel Transfer Tube (Penetration #65) | NA |
| 17. 2-CV-4* | Air Ejector Suction | NA |
| 18. 2-RC-143* | Dead Weight Pressure Calibrator | NA |
| 19. 2-RC-145* | Dead Weight Pressure Calibrator | NA |
| 20. 1-RP-84* | Refueling Purification Outlet | NA |
| 21. 2-RP-7* | Refueling Purification Outlet | NA |
| 22. 2-RP-6* | Refueling Purification Inlet | NA |
| 23. 1-RP-50* | Refueling Purification Inlet | NA |
| 24. (Deleted) | | |
| 25. (Deleted) | | |
| 26. (Deleted) | | |
| 27. 2-WT-437* | Steam Generator Wet Layup | NA |
| 28. 2-WT-438* | Steam Generator Wet Layup | NA |
| 29. 2-WT-439* | Steam Generator Wet Layup | NA |
| 30. 2-WT-446* | Steam Generator Wet Layup | NA |

TABLE 3.6-1 (Cont.)

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-------------------------|--|--|
| 31. 2-WT-447* | Steam Generator Wet Layup | NA |
| 32. 2-WT-448* | Steam Generator Wet Layup | NA |
| 33. 2-SI-83* | High Head Safety Injection, (Boron Injection Tank Bypass) | NA |
| 34. NA* | Fire Protection Supply (Penetration 34) | NA |
| E. REMOTE MANUAL | | |
| 1. MOV-QS201A* | Quench Spray Pump Discharge | NA |
| 2. MOV-QS201B* | Quench Spray Pump Discharge | NA |
| 3. MOV-RS255A#* | Recirc. Spray Pump Suction | NA |
| 4. MOV-RS255B#* | Recirc. Spray Pump Suction | NA |
| 5. MOV-2860A#* | LHSI Pump Suction From Containment Sump | NA |
| 6. MOV-2860B#* | LHSI Pump Suction From Containment Sump | NA |
| 7. MOV-RS256A* | Recirculation Spray Pump Discharge | NA |
| 8. MOV-RS256B* | Recirculation Spray Pump Discharge | NA |
| 9. MOV-SW203A* | Service Water to Recirculation Spray Coolers | NA |
| 10. MOV-SW203B* | Service Water to Recirculation Spray Coolers | NA |
| 11. MOV-SW203C* | Service Water to Recirculation Spray Coolers | NA |
| 12. MOV-SW203D* | Service Water to Recirculation Spray Coolers | NA |
| 13. MOV-SW204A* | Service Water from Recirculation Spray Coolers | NA |

TABLE 3.6-1 (Cont.)

| | <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-----|-------------------------|---|--|
| 14. | MOV-SW204B* | Service Water from Recirculation Spray Coolers | NA |
| 15. | MOV-SW204C* | Service Water from Recirculation Spray Coolers | NA |
| 16. | MOV-SW204D* | Service Water from Recirculation Spray Coolers | NA |
| 17. | TV-CV200* | Containment Air Ejector Suction | NA |
| 18. | MOV-2869A* | High Head Safety Injection to RCS Except Boron Injection Line | NA |
| 19. | MOV-2836* | High Head Safety Injection to RCS Except Boron Injection Line | NA |
| 20. | MOV-2869B* | High Head Safety Injection to RCS Except Boron Injection Line | NA |
| 21. | HCV-2142* | Reactor Coolant Letdown Line From RHR System | NA |
| 22. | | (Deleted) | |
| 23. | | (Deleted) | |
| 24. | MOV-2890A* | LHSI Pump Discharge to Reactor Coolant System Hot Legs | NA |
| 25. | MOV-2890B* | LHSI Pump Discharge to Reactor Coolant System Hot Legs | NA |
| 26. | MOV-2890C* | LHSI Pump Discharge to Reactor Coolant System Cold Legs | NA |

TABLE 3.6-1 (Cont.)

| | <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-----|-------------------------|--|--|
| 27. | MOV-2890D* | LHSI Pump Discharge to Reactor Coolant System Cold Legs | NA |
| 28. | FCV-2160* | Loop Fill Header | NA |
| 29. | MOV-2289A* | Charging Line | NA |
| 30. | MOV-2867C* | High Head Safety Injection, Boron Injection Tank | NA |
| 31. | MOV-2867D* | High Head Safety Injection, Boron Injection Tank | NA |
| 32. | MOV-RS-200A* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 33. | MOV-RS-200B* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 34. | MOV-RS-201A* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 35. | MOV-RS-201B* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 36. | TV-HC-208A | Containment Atmosphere Sample Line | NA |
| 37. | TV-HC-208B | Containment Atmosphere Sample Line | NA |
| 38. | TV-HC-200A | Suction Hydrogen Analyzer | NA |
| 39. | TV-HC-200B | Suction Hydrogen Analyzer | NA |
| 40. | TV-HC-201A | Discharge Hydrogen Analyzer | NA |
| 41. | TV-HC-201B | Discharge Hydrogen Analyzer | NA |
| 42. | TV-HC-202A | Suction Hydrogen Analyzer | NA |
| 43. | TV-HC-202B | Suction Hydrogen Analyzer | NA |
| 44. | TV-HC-203A | Discharge Hydrogen Analyzer | NA |

TABLE 3.6-1 (Cont.)

| | <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|----------|-------------------------|--|--|
| 45. | TV-HC-203B | Discharge Hydrogen Analyzer | NA |
| 46. | TV-HC-204A | Suction Hydrogen Recombiner | NA |
| 47. | TV-HC-204B | Suction Hydrogen Recombiner | NA |
| 48. | TV-HC-205A | Discharge Hydrogen Recombiner | NA |
| 49. | TV-HC-205B | Discharge Hydrogen Recombiner | NA |
| 50. | TV-HC-206A | Suction Hydrogen Recombiner | NA |
| 51. | TV-HC-206B | Suction Hydrogen Recombiner | NA |
| 52. | TV-HC-207A | Discharge Hydrogen Recombiner | NA |
| 53. | TV-HC-207B | Discharge Hydrogen Recombiner | NA |
| F. CHECK | | | |
| 1. | 2-CC-194 | Component Cooling Water to RHR System and Excess Letdown Heat Exchanger | NA |
| 2. | 2-CC-199 | Component Cooling Water to RHR System and Excess Letdown Heat Exchanger | NA |
| 3. | 2-SI-93 | High Head Safety Injection, Boron Injection to RCS | NA |
| 4. | 2-CC-302 | Component Cooling Water to Containment Air Recirculation Coils | NA |
| 5. | 2-CC-289 | Component Cooling Water to Containment Air Recirculation Coils | NA |

TABLE 3.6-1 (Cont.)

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-------------------------|--|--|
| 6. 2-CC-276 | Component Cooling Water to Containment Air Recirculation Coils | NA |
| 7. 2-CH-335 | Charging Line | NA |
| 8. 2-CC-152 | Component Cooling Water to Reactor Coolant Pumps | NA |
| 9. 2-CC-115 | Component Cooling Water to Reactor Coolant Pumps | NA |
| 10. 2-CC-78 | Component Cooling Water to Reactor Coolant Pumps | NA |
| 11. 2-CH-331 | Reactor Coolant Pumps, Seal Water Return | NA |
| 12. 2-SI-136 | Safety Injection Accumulator Make Up | NA |
| 13. 2-SI-85 | High Head Safety Injection to RCS except Boron Injection Line | NA |
| 14. 2-HC-20 | Discharge From Containment Atmosphere Clean-up System | NA |
| 15. 2-HC-15 | Discharge From Containment Atmosphere Clean-up System | NA |
| 16. 2-CH-308# | Reactor Coolant Pump Seal Water Supply | NA |
| 17. 2-CH-260# | Reactor Coolant Pump Seal Water Supply | NA |
| 18. 2-CH-284# | Reactor Coolant Pump Seal Water Supply | NA |
| 19. 2-IA-428 | Air Radiation Monitor Return | NA |
| 20. 2-RC-162 | Primary Grade Water | NA |
| 21. 2-CH-332 | Loop Fill Header | NA |
| 22. 2-IA-250 | Containment Instrument Air Return | NA |

TABLE 3.6-1 (Cont.)

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-------------------------|---|--|
| 23. 2-SI-132 | Nitrogen to Pressurizer Relief Tank and SI Accumulators | NA |
| 24. 2-SI-126 | LHSI Pump Discharge to Reactor Coolant System Hot Legs | NA |
| 25. 2-SI-128 | LHSI Pump Discharge to Reactor Coolant System Hot Legs | NA |
| 26. 2-SI-91 | LHSI Pump Discharge to Reactor Coolant System Cold Legs | NA |
| 27. 2-SI-99 | LHSI Pump Discharge to Reactor Coolant System Cold Legs | NA |
| 28. 2-SI-105 | LHSI Pump Discharge to Reactor Coolant System Cold Legs | NA |
| 29. 2-QS-22** | Quench Spray Pump Discharge | NA |
| 30. 2-QS-11** | Quench Spray Pump Discharge | NA |
| 31. 2-RS-30** | Recirculation Spray Pump Discharge | NA |
| 32. 2-RS-20** | Recirculation Spray Pump Discharge | NA |
| 33. 2-VP-24 | Air Ejector Vent | NA |
| 34. 2-SI-119 | High Head Safety Injection to RCS Except Boron Injection Line | NA |
| 35. 2-SI-107 | High Head Safety Injection to RCS Except Boron Injection Line | NA |
| 36. 2-FW-62# | Feedwater to Steam Generators | NA |

TABLE 3.6-1 (Cont.)

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-------------------------|--|--|
| 37. 2-FW-94# | Feedwater to Steam Generators | NA |
| 38. 2-FW-126# | Feedwater to Steam Generators | NA |
| 39. 2-WT-41# | Chemical Feed Lines | NA |
| 40. 2-WT-53# | Chemical Feed Lines | NA |
| 41. 2-WT-69# | Chemical Feed Lines | NA |
| 42. 2-FW-70# | Auxiliary Feedwater to Steam Generator | NA |
| 43. 2-FW-102# | Auxiliary Feedwater to Steam Generator | NA |
| 44. 2-FW-134# | Auxiliary Feedwater to Steam Generator | NA |
| 45. 2-RS-103# | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 46. 2-RS-118# | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 47. NA | Fire Protection Supply (Penetration 34) | NA |

TABLE 3.6-1 (Cont.)

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-------------------------|----------------------------|--|
| G. STEAM LINE ISOLATION | | |
| 1. TV-MS201A# | Main Steam Line Trip Valve | 5 |
| 2. TV-MS201B# | Main Steam Line Trip Valve | 5 |
| 3. TV-MS201C# | Main Steam Line Trip Valve | 5 |
| II. RELIEF | | |
| 1. RV-2203 | Letdown Line Relief Valve | NA |

Valve not subject to Type "C" leakage test.

* Valve position maintained by administrative control

NA - Not applicable

** Weight loaded check valve

CONTAINMENT SYSTEMS

3/4.6.4 COMBUSTIBLE GAS CONTROL

HYDROGEN ANALYZERS

LIMITING CONDITION FOR OPERATION

3.6.4.1 Two independent containment hydrogen analyzers (shared with Unit 1) shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTION:

With one hydrogen analyzer inoperable, restore the inoperable analyzer to operable status within 30 days or be in at least HOT STANDBY within the next 6 hours.

SURVEILLANCE REQUIREMENTS

4.6.4.1 Each hydrogen analyzer shall be demonstrated OPERABLE at least once per 92 days on a STAGGERED TEST BASIS by performing a CHANNEL CALIBRATION using sample gases containing:

- a. One volume percent ($\pm .25\%$) hydrogen, balance nitrogen, and
- b. Four volume percent ($\pm .25\%$) hydrogen, balance nitrogen.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 25 TO FACILITY OPERATING LICENSE NO. NPF-7
VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION, UNIT NO. 2
DOCKET NO. 50-339

Introduction:

By letter dated April 23, 1982 (Serial No. 275), the Virginia Electric and Power Company (the licensee) requested a change to the Technical Specifications (TS) for the North Anna Power Station, Unit No. 2 (NA-2).

The requested change would add and delete certain containment isolation valves to improve post-accident sampling activities. Also, presently installed manual isolation valves would be replaced with remote operated isolation valves for post-accident containment hydrogen analysis and control equipment.

A discussion and our evaluation and conclusion regarding the licensee's requested change is provided below.

Discussion:

The proposed change would revise the NA-2 TS 3/4.6.3.1, Table 3.6-1 to reflect the addition and deletion of containment isolation valves.

The addition of two (2) direct-acting solenoid valves (TV-SS203A and TV-SS203 B) will be used to replace two (2) air-operated trip valves (TV-SS207A and TV-SS207B) in the Residual Heat Removal System Sample Lines. The double isolation direct-acting solenoid valves are being installed to provide increased assurance of reliable operation during accident conditions. The valves will be normally closed and receive a Phase A signal to assure they are tripped closed on a safety injection signal. These modifications are required to meet the provisions of NUREG-0737, II.B.3, Post-Accident Sampling.

The addition of two air-operated Phase A trip valves (TV-DA203A and TV-DA203B) are being installed on the Post-Accident Sampling System return lines. These valves will reduce radiation levels outside containment should post-accident samples be required to be withdrawn from the reactor coolant system and containment sump. These modifications are required to meet the provisions of NUREG-0737, II.B.3, Post-Accident Sampling.

Four (4) manual isolation valves (2-HC-13, 2-HC-29, 2-HC-18, and 2-HC-33) in the hydrogen-recombiner and analyzer system are being replaced with sixteen (16) remote-manual valves (HC-series valves) to upgrade the hydrogen recombiner and analyzer system. The addition of these remote-manual isolation valves will reduce radiation levels

outside of containment should the hydrogen analyzer and recombiner be required to be in service for post-accident conditions.

Eight of these valves are in the suction and return lines for the hydrogen analyzers. These remote-manual valves are TV-HC-200A&B (in series), TV-HC-201 A&B (in series), TV-HC-202 A&B (in series) and TV-HC-203 A&B (in series).

Eight of these valves are in the suction and discharge lines for the hydrogen recombiners. These remote-manual valves are TV-HC-204 A&B (in series) TV-HC-205 A&B (in series), TV-HC-206 A&B (in series), and TC-HC-207 A&B (in series).

The above modifications are required to meet the provisions of NUREG-0737, Item II.B.2, Post Accident Shielding.

Evaluation:

The upgrading and installation of the above containment isolation valves meets the requirements for Category I Containment Isolation Valves specified in the NA-2 Final Safety Analysis Report. Double barrier protection is provided by two (2) valves to assure that no single failure will result in the loss of containment integrity. Containment penetration piping including the isolation valves are designed to Seismic Category I Requirements.

As stated above, isolation valves TV-SS203A, TV-SS203B, TV-DA203A, and TV-DA203B are normally closed and receive a Phase A signal to assure they are tripped closed on a safety injection signal. Maximum isolation time for these valves is specified to be 60 seconds. We have already reviewed Phase A isolation as specified in the NA-2 FSAR and found it to be acceptable as well as a maximum closure time of 60 seconds for containment isolation.

The remote-manual valves (HC series valves) being added for the post-accident containment hydrogen and control equipment and numbered 200 A&B through 208A&B (sixteen valves in all) will be closed at all times and can be only opened upon remote-manual activation from the control room. Opening of these valves will only take place under specific administrative control as specified in post-accident procedures.

Based on the above, we find the licensee's request to add the above specified isolation valves to the NA-2 TS 3/4.6.3.1, Table 3.6-1, to be acceptable. Also, these isolation valves are required to meet the provisions of NUREG-0737, Item II.B.2, Post Accident Shielding; and Item II.B.3, Post Accident Sampling.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: ~~(1) because the amendment does not involve a significant increase~~ in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public ~~will not be endangered by operation in the proposed manner, and (3)~~ such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 17, 1982

Principal Contributors:

Leon B. Engle

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-339VIRGINIA ELECTRIC AND POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 25 to Facility Operating License No. NPF-7 issued to the Virginia Electric and Power Company (the licensee) for operation of the North Anna Power Station, Unit No. 2 (the facility) located in Louisa County, Virginia. The amendment is effective as of its date of issuance.

The amendment revises the NA-2 Technical Specifications by upgrading and adding twenty (20) containment isolation valves to Table 3.6.1 to meet the requirements of NUREG-0737, Action Item II.B.2, Post-Accident Shielding; and Action Item II.B.3, Post-Accident Sampling.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

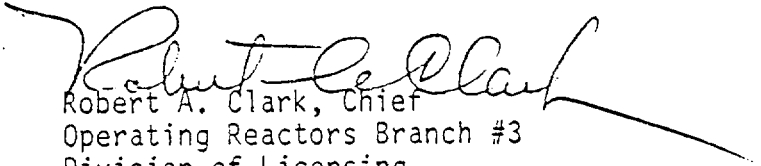
- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated April 23, 1982; (2) Amendment No. 25 to Facility Operating License No. NPF-7; and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. 20555 and at the Board of Supervisors Office, Louisa County Courthouse, Louisa, Virginia 23093 and at the Alderman Library, Manuscripts Department, University of Virginia, Charlottesville, Virginia 22901. A copy of items (2) and (3) may be obtained upon request to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland the 17th day May, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing