Appendix D	Scenario Outline NRC Se	t I Form FS-D-1
Facility: Catawba	Scenario No.: NRC 1	Op-Test No:
Examiners:		Applicants:

Objectives: To evaluate the applicants' ability to reduce power using NOPs while maintaining Tave matched to Tref, and to use AOPs to respond to a pressurizer level channel instrument failure, complicated by the loss of letdown with a turbine runback The candidates will then be evaluated using the FOPs to respond to a small S/U tube leak that increases in size until a reactor trip is required. At this point the leak increases to a 330 GPM rupture. Mitigation will be complicated by the ruptured generator 5/0 PORV failing to reseat after opening and the loss of the A ND and NV pumps.

Initial Conditions: 100% Power 200 EFPD 807 [B] 100% CPL Equilibrium Xe (SNAP 153) Prevent A NV pump from starting (auto and manual) DO IA inoperable ND pump IA inoperable 1NV123B binding closed

Turnover: 100% Power 200 EFPD 807 [B] 100% CPL Equilibrium Xe IA ND pump tagged out for an oil change. Back in 3 hours.

IA DIG in maintenance mode for gasket change. Back early next shift.

Reduce load to 65% due to Chemistry concerns for a CFPT condenser tube leak. FflventM Event Event IITvpe* Description 1 N/A N-BOP Boration for load decrease N(A R-RO Decrease turbine load 2 3 XMT-NCO13 I~BOP Pressurizer Level Channel II fails LOW with loss of nonmal letdown N/A C-BOP Failure to restore any letdown to service 4 5MAL-1Rx004 C-RO Turbine fails to runback in auto on loss of Generator Breaker OVR-EPOOSB OVR-EPOO6C MAL-SCOOI B C-BOP I B Steam Generator Tube Leak. IA NV Pump fails to start. 6 SV=190 MAL-NVOO6A 7MAL-SCOOIS M IB Steam Generator Rupture coincident with manual reactor trip SV=330 8MAL-SMOO2B C-RO S/G PORV on B SIG leaks by seat just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure) Sv=100 NUREO-1021 Page 1 Rev. 8 Scenario Outline NRC Set 3 Appendix D Form ES-D-I Simulation Facility: Catawba Scenario No.: NRC-3 Op-Test No: Examiners: Operators:

Objectives: To evaluate the applicants' ability to reduce power using NOPs while maintaining Tave matched to Tret; and to use AOPs to respond to instrument failures in the Rod Control program, a failure of the transmitter controlling reactor coolant pump seal injection flow and the ability to diagnose and use the AP for a small Reactor Coolant System leak. Tile applicants will be evaluated using EOPs to mifig ate an ejected control rod that results in a small break LOCA. Mitigation efforts will be complicated by the failure of the reactor to automatically trip and the failure of SI cold leg injection valves INI-9A and iNI-IOB to open automatically. when the emergency diesel generators are secured in F-i, normal power will be lost to 1 ETA. The applicants will have to restart the equipment that was running prior to the loss of 1 ETA.

Initial Conditions: 100% power 400 EFPD 30 [B] Equilibrium Xenon conditions (SNAP 151)

Block auto and manual reactor trip

Block auto safety injection

Turnover: 100% power 400 EFPD 30 [B] Equilibrium Xenon conditions

IB diesel in maintenance mode for injector maintenance

18 CA pump tagged out for an oil change. Back in 4 hours.

Channel 4 FWST level transmitter is inoperable (Failed)

Thunderstorms in the area

Reduce power to 0% in preparation for the next reftieling outage Event

Event ThEvent No. ThMalf No.

- I Type* Description T
- Ι N-BOP Boration for load decrease
- 2 R-RO Decrease turbine load
- XMT+iCo61 1-RO NC Loop B Tcold fails to 6300 F 3 XMT44VO52 1-BOP NCP seal injection transmitter failure 4

5 M Small reactor coolant leak (~90 gpm) 6 MAL ~Coos M Rod ejection
BIO
6aMALIPXOOIA C-RO ATWS MAL~PXOO1B
MALI PXoU~
MAL4PXOO2S 6b VLV*ZIOOIA C-flOP Failure of INI-9A and INI-IOB to open automatically
7 OvR~Po290C-HOP Loss of normal power to I ETA
NUREG-1021 Page 1 Revision 8
Appendix D Operator Actions Form ES-D-2
Op-Test No.: NRC Scenario No.: 1 Event No.: 1
Event Description: Boration for load decrease Time Position Applicant's Actions or Behavior
DOD Defer to ODIIA/C1501000 Dever Concentration Control
BOP Refer to OPIIIA/61501009, Boron Concentration Control
BOP Ensure the following valve control switches in "AUTO":
INV-238A (B/A Xfer Pmp To Blender Ctrl)
• 1 NV-i 864 (B/A Blender OtIt To VCT OtIt)
BOP Adjust the boric acid batch counter to the desired volume of boric acid to be added.
BOP Place the "NC MAKEUP MODE SELECT' switch in "BORATE".
BOP Adjust the controller for 1 NV-238A (B/A Xfer Pmp to Blender Ctrl)
BOP Ensure I NV-236A (B/A Xfer Pmp to Blender Ctrl) controller in
BOP Ensure at least one boric acid transfer pump in "AUTO" or "ON".
BOP Place the "NC MAKEUP MODE SELECT' switch in "START'
position. BOP Verify the following valves open:
• 1 NV-238A (B/A Xfer Pmp To Blender Ctrl Vlv)
• 1 NV-i 86A (B/A Blender OtIt To VCT Otit)
BOP If in "AUTO", verify the boric acid transfer pump starts.
NUREG-1021 1 of 38 Rev. S
Appendix D Operator Actions 10111 L3-D-2
Op-Test No.: NRC Scenario No.: 1 Event No.: 1
Event Description: Boration for load decrease
F;;ne I~oswon Applicant's Actions or Behavior
BOP Verify proper flow by observing the boric acid flow totalizer. {PIP 96-0137}
BOP When the desired volume of boric acid is reached on the boric acid batch counter, ensure the following valves close:
• 1 NV-238A (B/A Xfer Pmp To Blender Ctrl Vlv)

• 1 NV-i 86A (B/A Blender Ott To VCT OtIt)

- BOP IF desired, flush the makeup line as follows:
 - Open the following valves:
 - 1 NV-242A (RMWST To B/A Blender Ctrl)
 - 1 NV-i 86A (B/A Blender OtIt To VCT OtIt)
 - Ensure one reactor makeup water pump is in "ON".
 - WHEN -20 gallons of makeup water have been flushed through the makeup line, close the following valves:
 i NV-242A (RMWST To B/A Blender Ctrl)
 - •1 NV-i 86A (B/A Blender OtIt To VCT OtIt)
 - Placethe following valve control switches in "AUTO":
 - •i NV-242A (RMWST To B/A Blender Ctrl)
 - •I NV-i 86A (B/A Blender OtIt To VCT OtIt)
 - IF liQI required for current plant operation, place the reactor makeup water pump started in earlier step in "AUTO".

IF automatic makeup is desired, refer to Enclosure 4.1 (Automatic Makeup).

NUREG~1021	2 of 38	Rev. S
Appendix D	Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 2 Event Description: Decrease turbine load

[Time ffl~osition] Applicant's Actions or Behavior

- RO Refer to OPI1/B/6300/OO1, Turbine Generator, Enclosure 4.2, Section 2.4.
 - RO Depress the "Load Rate" pushbutton and verify it illuminates.
 - RO Input the desired load rate on the numeric keypad and verify the load rate appears on the Variable Display.
 - RO Depress the "Enter" pushbutton.
 - RO Depress the "Target" pushbutton and verify it illuminates.
 - RO Input the desired load target on the numeric keypad and verify the load target appears on the Target Display.
 - RO Depress the "Enter" pushbutton.

RO	To start load decrease,	, depress the "Go"	pushbutton and verify it
NUREG-102 1	3 of 38	Rev.	8
Appendi	ix D Operato	or Actions I	Form ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 3

Event Description: Pressurizer Level Channel II fails LOW Time_Wmtion Applicant's Actions or Behavior BOP Recognizes Pressurizer Level Channel II has failed LOW and informs SRO.

- 1AD-6, D19 (PZR LO LEVEL HTR OFF & LETDOWN SECURED annunciator
- 1AD-6, AlIO (PZR HTR CONTROLLER TROUBLE) annunciator
- Pressurizer level channel II 1NCP5153offscale~LOW

EXAMINER NOTE: Annunciator response is anached.

BOP Responds to event using the annunciator response.

BOP Recognizes that letdown has been lost due to this failure and informs

510.		
NUREG-1021	4 of 38	Rev. S
Appendix D	Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 4

CDU

Event Description: Loss of letdown - establish excess letdown Time Position Applicant's Actions or Behavior

- SRO Implements APIIINSSOOII2 (Loss of Charging or Letdown) Case II (Loss of Letdown) and directs operators.
- BOP Verify all Pzr level channels - INDICATING THE SAME.

Determines that channel II has failed low and informs SRO.

- SRO Transitions to Step 1 RNO and directs the operators.
- IF the controlling channel is failed low, THEN place "PZR LEVEL BOP CTRL SELECT" switch in any alternate operable position.
- SRO Transitions back to Step 2 NER column and directs operators.
- RO Stop any power changes.

BOPVerify the following letdown isolation valves - CLOSED.

- 1NV-1OA (Letdn Orif IB OtIt Cont Isol)
- INV-11A (Letdn Orif ic OtIt Cont Isol)
- INV-13A (Letdn Orif IA Ott Cont Iso)
- BOP Verify PZR level GREATER THAN 17%
- BOP Control charging to stabilize Pzr level at program level while maintaining seal injection flow.

NUREG-1021	5 of 38	Rev. S
Appendix D	Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 4

Event Description: Loss of letdown - establish excess letdown

- Time Position Applicant's Actions or Behavior
 - BOP Ensure "PZR HEATER GROUP iC" ON.
 - BOP Control VCT level as follows:
 - a. Verify VCT makeup SET FOR DESIRED BORON CONCENTRATION
 - b. Verify VCT makeup IN AUTOMATIC
 - SRO Determine and oorrect cause of loss of letdown.
 - BOP Ensure "PZR LEVEL TO REC SEL" is selected to an operable

channel.

- SROEnsure compliance with appropriate Tech Specs:
 - 3.3.1 (Reactor Trip system (RTS) Instrumentation)
 - 3.3.3 (Post Accident Monitoring (PAM) Instrumentation)
 - 3.3.4 (Remote Shutdown System)
 - 3.4.1 (RCS Pressure, Temperature, and Flow Departure From

Rev. S

Nucleate Boiling (DNB) Limits)

REG~1021	6 OF 38

Op-Test No.: NRC Scenario No.: 1 Event No.: 4

Event Description: Loss of letdown - establish excess letdown TimePosition Applicant's Actions or Behavior BOPEvaluate normal letdown restoration as follows:

- a. Verity at least one of the following valves CLOSED:
 - 1NV-1A (NC Letdn To Regen Hx Isol)

OR

- 1 NV-2A (NC Letdn To Regen Hx Isol)
- b. Perform the following."
 - 1) Establish excess letdown. REFER TO OP/1/k62001001 (Chemical and Volume Control System)
 - 2) Dispatch operator(s) to pressurize the normal letdown line. REFER TO Enclosure 1 (Pressurization Of Normal Letdown Line).
- C. Do not continue in this procedure until one of the following is met:
 - Notified by dispatched operator that the letdown line is pressurized.

OR

- Station management authorizes normal letdown restoration.
- SRO Determine that normal letdown is currently not available due to 1 NV-2A closing and directs excess letdown to be placed in service.
- SRO REFERS TO OP/1/N6200/OO1 (Chemical and Volume Control System) Enclosure 4.12 (Establishing/Securing Excess Letdown) and directs operators.
- Time Position Notify Primary Chemistry of the following: (PIP 96-3230)
 - Excess Letdown will be placed in service

VCT pressure will be reduced to - 20 psig
NUREG-1021
 7 of 38
 Rev. S
 Appendix D
 Operator Actions
 Form ES-ID-2

Op-Test No.: NRC Scenario No.: I Event No.: 4

- Event Description: Loss of letdown establish excess letdownTimePositionApplicant's Actions or Behavior
 - BOP CAUTION: At least 15 psig backpressure shall be maintained on the NCP #1 seals

Reduce VCT pressure to - 20 psig per Enclosure 4.20 (Adjusting the Volume Control Tank (VCT) Hydrogen Pressure)

- BOPOpen the following valves to establish KC flow to the Excess
Letdown Heat Exchanger:
1 KC-305B (Exs Letdn Hx Supply Cont Isol)
 - I KC-3 1 5B (Excess Letch Hx Ret Cont Isol)
- BOP Verify 1NV-125B (Excess Letdn Hx Otit Ctrl) is in the "VCT" position.
- BOP IF either of the following conditions exist, place 1 NV-i 25B (Excess

Letdn Hx OtIt Ctrl) in the "NCDT" position:

- VCT pressure greater than or equal to 45 psig as indicated on 1 NVP5SOO (VCT Vent Press)
- VCT level greater than or equal to 50% as indicated on 1 NVP5761 (VCT Level)
- BOP Open the following valves:
 - 1NV-122B (Loop C To Exs Letdn Hx Isol)
 - 1NV-123B (Loop C To Exs Letdn Hx Isol)

Determines that 1NV-123B will not open and informs SRO. SRO determines that no letdown is available and stops in API12. EXAMINER NOTE: Letdown will not be restored. 1 S of 38 Rev. S

NUREG-1021S of 38Rev. SAppendix DOperator ActionsForm ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 5 - 6

Event Description: Turbine runbackTimePositionApplicant's Actions or Behavior

ALL Recognize indication of a generator breaker trip:

iAD-1 1, C/il (Gen Bkr A Overcurrent) annunciator Determines that a runback should be occurring but turbine is not automatically running back. Recognize conditions for AP/1/N5500103 (Load Rejection), inform SRO, and perform immediate actions from memory.

SRO Implements AP/11AI5500/03 (Load Rejection) and directs operators.

RO Verify turbine load - DECREASING.

Determines turbine load is not decreasing and informs SRO. SRO Transitions to Step I RNO and directs operators. ROPerform the following: a. Select "MANUAL" on the turbine control panel.

b. Depress "CONTROL VALVES LOWER" pushbutton and reduce turbine load as required.

SRO Transitions to Step 2 AIER column and directs operators.

- RO Verify proper reactor response:
 - Control rods IN "AUTO" AND STEPPING IN

Op-Test No.: NRC Scenario No.:1 Event No.: 5-6

Event Description: Turbine runback

TimePosition Applicant's Actions or Behavior ROVerify proper steam dump operation as follows:

- a. "C-9 COND AVAILABLE FOR STM DUMP" status light (1SI-18) - LIT
- b. "C-7A LOSS OF LOAD INTLK COND DUMP" status light (iSI-18)-LIT
- C. Steam dump valves MODULATING

d. T-Avg - DECREASING TO T-REF

BOP Verify Pzr PORV and Pzr spray valve status as follows:

a. All Pzr PORVs - CLOSED

- b. Normal Pzr spray valves CLOSED
- SOP Verify proper CM System operation as follows:
 - a. WHEN reactor power is less than 75%, THEN secure both C-Htr drain pumps. REFER TO OP/1/B16250/004 (Feedwater Heater, Vents, Drains And Bleed Systems)
 - b. Verify reactor power GREATER THAN 56% PRIOR TO THE EVENT
 - c. Verify standby hotwell pump(s) ON

 d. Verify standby condensate booster pump(s) - ON
 Determines that standby hotwell and booster pumps did not start and manually starts them if necessary per Step S.c and S.d RNO.
 BOP Verify the following generator alarms - DARK:

• 1AD-11,CI1 "GEN BKRAOVER CURRENT'

•	IAD-il, F/i "GEN BKR B OVER	CURRENTS
NUREG~1021	11 of 38	Rev, S
Appendix D	Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 5 - 6

Event Description: Turbine runback [Time]IIWfltion Applicant's Actions or Behavior

> EXAMINER NOTE: If alarm in at this point, SRO will go to RNO. SRO EXAMINER NOTE: This is Step 6 RNO.

Ensure turbine generator load - REDUCED TO APPROXIMATELY 50% AND THE ALARM CLEARS

RO Verify SIG levels are adequate as follows:

- All SIG low level alert alarms (IADA) DARK
- All S/C low CF flow alarms (1 ADA) DARK

EXAMINER NOTE: If alarm in at this point, SRO will go to RNO. SRO EXAMINER NOTE: This is Step 7 RNO.

- Perform the following: a. Ensure feedwater regulating valves - MODULATING TO CONTROL SIG LEVELS AT PROGRAM SETPOINT.
 - b. IF any S/C(s) NR level is decreasing in and uncontrolled manner, THEN:
 Determines this step is N/A and continues
 - Determines this step is N/A and continues.
- SRO Transitions to Step 8 NER column and directs operators.
- BOP Verify AS header pressure GREATER THAN OR EQUAL TO 140 PSIG.
- BOP Monitor Enclosure 3 (Rod Insertion Limit Boration).
- EXAMINER NOTE: AP111AI5500103, Enclosure 3 is aflached.NUREG-102112 of 3SRev. SAppendix DOperator ActionsForm ES-D-2

Op-Test No.: NRC Scenario No.: I Event No.: 5 - 6 Event Description: Turbine runback Time Position Applicant's Actions or Behavior RO Verify reactor power - LESS THAN 30%

Determines that power is greater than 30% and informs SRO

- SRO Transitions to Step 10 RNO and directs operators.
- SRO a. IF the runback target load is less than 30%, THEN:

Determines this step is N/A and continues

- b. WHEN the appropriate runback target load is reached, THEN:
- 1) Stabilize unit at current power level
- 2) Maintain control rods above insertion limits
- 3) Adjust the following as required to maintain T-Avg within 10F
- of T-Ref:
- Turbine load
- Control rods
- Boron Concentration
- c. GOIQStepl2.

SRO Transitions to Step 12 A/ER and directs operators. NUREG-1021 13 of 38 Rev. S

Appendix D Operator Actions Form ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 5 - 6 Event Description: Turbine runback Time Position Applicant's Actions or Behavior

RO/BOP Verify the following PCBs - CLOSED:

- Generator breaker IA
- Generator breaker 1 B
- PCB 14
- PCB14
- PCB13
 PCB17
- PCB18

Determines that Generator breaker 1 \mbox{B} is not closed and informs SRO.

SRO Transitions to Step 12 RNO and directs operators.

SRO Perform the following:

- a. IF the turbine generator is separated from the grid, THEN Determines step is N/A and continues.
- b. IF load rejection caused by loss of main busline IA or IB, THEN:

Determines step is N/A and continues.

c. IF a full load rejection has occurred, IHitL: Determines step is N/A and continues.

SRO Transitions to Step 13 NER column and directs operators.

RO Adjust power factor as necessary. REFER TO Unit 1 Revised Data Book Figure 43.

NUREG-1021	1401 30	Rev. 5
Appendix D	Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 5 - 6

Event Description: Turbine runback

Time_'mtion Applicantas Actions or Behavior

- ALLWHEN the appropriate runback target load is reached, THEN:
 - Stabilize unit at appropriate power level
 - Maintain control rods above insertion limits
 - Adjust the following as required to maintain T-Avg within 10F of T-Ref
 - Turbine load
 - Control rods
 - Boron concentration

ROISRO Notify System Operating Center (SOC) using the red dispatcher telephone of current unit status.

- CF pump. REFER TO OP/1/Al62501001 (Condensate and Feedwater)
- RC pump(s) and cooling tower fans. REFER TO OP/i IB/6400/OO1 A (Condenser Circulating Water)
- Hotwell pump(s)

Condensate booster pump(s).
 NUREG-1021 15 of 38 Rev. S
 Appendix D Operator Actions Form ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 5 - 6

Event Description: Turbine runback Time Position Applicant's Actions or Behavior ROReset steam dump valves as follows:

- a. Verify reactor power STABLE.
- b. Verify steam dumpvalves- IN "T AVG" MODE.
- C. Verify steam dump valves CLOSED.
- d. Reset steam dump valves.
- e. Verify the following status lights (1SI-18) DARK.
 - "C-7A LOSS OF LOAD INTLK COND DUMP"
 - "C-7B LOSS OF LOAD INTLK ATMOS DUMP"
- f. IF "T AVG" mode of operation is available, THEN ensure steam dump valves in "T AVG" mode.
- g. Verify "STM DUMP CTRL" IN AUTOMATIC.
- RO Verify reactor power GREATER THAN 15%.

ROIBOP Verify CA Pumps - OFF.

RO Verify reactor power change - GREATER THAN OR EQUAL TO 15% IN AI HOUR PERIOD.

SRO/BOP/RO Verify the following sections to take appropriate samples:

- Radiation Protection to sample and analyze gaseous effluents. REFER TO Selected Licensee Commitments Manual, Section 16.11-6.
- Primary' Chemistry to sample for isotopic analysis of iodine. REFER TO Tech Specs 3.4.16 (Sample must be taken between 2 hours and 6 hours following last power change greater than or equal to 15% rated thermal power within a 1 hour period).

NUREG-102116 of 38Rev. SAppendix DOperator ActionsForm ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 5 - 6

SRO Ensure compliance with appropriate Tech Sp	pecs:
--	-------

- 3.1.1 (Shutdown Margin (SDM))
- 3.1.6 (Control Bank Insertion Limits)
- 3.8.1 (AC Sources Operating)

SROIBOPIRO Notify Reactor Group Engineer of occurrence.

ALL Determine.long term plant status. RETURN TO 0P111N61001003 (Controlling Procedure for Unit Operation). NUREG-1021 17 of 38 Rev. S

-			
	Appendix D	Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 1 Event No.: 7 - 8

Event Description: 1 B Steam Generator Tube Leak, IA NV pump fails to start. TimePosition Applicant's Actions or Behavior ALLRecognize symptoms of a Steam Generator Tube Leak:

- Charging flow increasing
- EMF 33, 72,73, 27, 31 alarm

SRO Implements AP111N5500110 (Reactor Coolant Leak) Case I, Steam Generator Tube Leak, and directs operators ALL Monitor Enclosure 1 (Foldout Page) ROIBOP Verify Pzr level - STABLE OR INCREASING

Determines Pzr level is decreasing and informs SRO. SRO Transitions to Step 2 RNO and directs operators. NUREG-1021 18 of 38 Rev. S Form ES-D-2 Appendix D **Operator Actions**

Op-Test No.: NRC Scenario No.: 1 Event No.: 7 - 8

Event Description: 1 B Steam Generator Tube Leak, IA NV pump fails to start. Time Position ApplicanUs Actions or Behavior

- BOP Perform the following:
 - a. Maintain charging flow less than 180 GPM
 - b. Manually throttle 1 NV-294 (NV Pmps A&B Disch Flow Ctrl) to stabilize Pzr level
 - C. IF Pzr level is stable OR increasing, THEN GO TO Step 3.

Determines Pzr level is still decreasing and continues in RNO. d. IF Pzr level continues to decrease, THEN:

1) Reduce letdown flow to 45 GPM as follows: EXAMINER NOTE: The Crew recognizes all letdown is isolated and intent of this step is met.

2) IF Pzr level continues to decrease, THEN ensure the following valves closed: f IA Ott Cont Isol) Determine step is N/A intent met and continues in RNO column. 3) IF Pzr level is stable OR increasing, THEN GO TO Step 3. Determines Pzr level is continuing to decrease and continues in RNO.

NUREG-102 1 19 of 38 Rev. S Form ES-D-2 Appendix D Operator Actions Op-Test No.: NRC Scenario No.: 1 Event No.: 9 - 10

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, S/C PORV on B S/G leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure). SRO

- 4) IFPzr level continues to decrease, THEN:
 - a)Start an additional NV pump as follows:
 - (1) Open 1NV-252A (NV Pumps Suct From FWST)
 - (2) Open 1NV-253B (NV Pumps Suct From FWST)
 - (3) Close 1 NV-i 88A (VCT OtIt Isol)

(4)	Close	1NV-189B	(VCT	OtIt Isol)
-----	-------	----------	------	------------

(5) Start the desired NV Pump

Recognizes than IA NV Pump fails to start and informs SRO.

Determines that no additional charging is available and determines than Pzr level is continuing to decrease.

b) Adjust Control Rods and turbine load as required to maintain T-Avg within 10F of T-Ref.

SRO REFER TO the following:

- AP/11Al5500109 (Rapid Downpower (OR)
- OP1IA/61001003 (Controlling Procedure for Unit Operation
- SRO c) LFPZr level is stable OR increasing THEN GO TO Step 3.
 - Determines that Pzr level is continuing to decrease and continues in RNO.
 - d) IE Pzr level is stable OR increasing THEN GO TO Step 3.

Determines that Pzr level is continuing to decrease and continues in

NUREG-1021	20 of 38	Rev. S
Appendix D	Operator Actions	Form ES-D-2
Op-Test No.: NRC Scenario No.: 1		Event No.: 9 - 10

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure).

ROIBOP e) if Pzr level continues to decrease OR Pzr level cannot be maintained greater than 11%,'tiiN:

- (1) Manually trip reactor
- (2) Manually initiate S/I
 - 3) GO TO EPI1IA/5000/E-0 (Reactor Trip Or Safety Injection).

NUREG-1021 21 of 38 Rev. S

Event Description: Steam Generator Tube Rupture coincident with manual reactor tnp, SIG PORV on B SIG leaks by seat -just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure).

Position Applicant's Actions or Behavior

- SRO Enters EPI1IA/50001E-O (Reactor Trip Or Safety Injection) and directs operators
- ALL Monitor Enclosure (1) (Foldout Page)

ROVerify Reactor Trip:

- All rod bottom lights LIT
- All reactor trip and bypass breakers OPEN
- hR amps DECREASING
- RO Verify Turbine Trip:

• All turbine stop valves - CLOSED

OR

- All turbine control valves CLOSED
- BOP Verify 1 ETA and 1 ETB ENERGIZED.
- BOP Verify S/I is actuated:
 - a."SAFETY INJECTION ACTUATED" status light(1SI-13) - LIT
 - b.EIS load sequencer actuated status lights (1SI-14) LIT
- RO Announce "Unit 1 Safety Injection"

RO	Verify all Feedwater Isolation status lights (151-5) - LIT		
NUREG-1021	22 of 38	Rev. S	
Appendix D Operator Actions		Form ES-D-2	
Op-Test No.: NRC Scenario No.: 1		Event No.: 9 - 10	

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure).

Time Position Applicant's Actions or Behavior

BOP Verify Phase A Containment Isolation status as follows:

a. Phase A RESET lights - DARK

b. Monitor Light Panel Group 5 St lights - LIT EXAMINER NOTE: IKC-305 may be in wrong position due to affempt to put excess letdown in service earlier in the scenario. If so, SOP will close it in response to this step's RNO.

- ALL Verify proper Phase B actuation as follows:
 - a. Containment pressure HAS REMAINED LESS THAN 3 PSIG
 - b. IF AN ANY TIME containment pressure exceeds 3 PSIG while in this procedure, Itjjt' perform Step 10.a.

RO Verify proper CA pump status as follows:

a. Motor driven CA pumps - ON

b. 3 SIG N/R levels - GREATER THAN 11%

- BOP Verify all of the following S/I pumps ON:
 - NVpumps
 - Nflpumps
 - NI pumps

Determines that IA NV pump and IAND pump are off and informs

	SRO.	
SRO	Transitions to Step 12 RN	O and directs the operators
NUREG~1021	23 of 38	Rev. S
Appendix	CD Operator Actions	Form ES-D-2
Op-Test No.: NRC Scenario No.: 1 Event No.: 9 - 10		

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure).

[Time Position Applicant's Actions or Behavior

BOPPerform the following for affected trains:

- a. Reset ECCS.
- b. Reset DIG load sequencer.
- c. Manually start affected pump (1/A NV Pump does not start).
- d. IF AT ANY TIME a BIO occurs, THEN restart S/I equipment previously on.
- BOP Verify proper ventilation systems operations.
 - REFER TO Enclosure 2 (Ventilation System Verification)
 - Notify Unit 2 operator to perform Enclosure 3 (Opposite unit Ventilation Verification)

EXAMINER NOTE: Enclosure 2 of this procedure is affached.

- RO Verify all S/G pressures GREATER THAN 775 PSIG
- BOP Verify proper S/I flow as follows:
 - a. "NV S/I FLOW' INDICATING FLOW
 - b. NC Pressure LESS THAN 1620 PSIG

Determines NI pressure is greater than 1620 psig and informs SRO.

SRO	Transitions to Step 17.b RNO and directs operators		
NUREG-1021	L021 24 of 33 Rev. 9		
Appendix D Operator Actions		Form ES-D-2	
Op-Test No.: NRC Scenario No.: 1		Event No.: 9 - 10	

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat -just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure). Time Position Applicant's Actions or Behavior BOP Perform the following: Ensure ND pump miniflow valve on operating ND pump(s) -1) OPEN 2) 'Ethe ND Pump minWow valve(s) cannot be opened, THEN perform the following for the affected train(s): Determines that this step is N/A and continues in RNO column. GOTOStepi8. 3) SRO Transition to step 18 A/ER and directs operators. RO Control SIG levels as follows: a. Verify total CA flow - GREATER THAN 450 GPM b. WHEN at least one SIG NIR level is greater than 11% (29% ACC), THEN throttle feed flow to maintain all SIG NIR levels between 11% (29% ACC) and 50%. Verify all CA isolation valves - OPEN RO BOP Verify S/I equipment status based on monitor light panel - IN PROPER ALIGNMENT Determines that IA NV pump, IAND pump, and possibly 1KC-305 are not in proper alignment and informs SRO. SRO Transition to Step 20 RNO and directs operators NUREG-1021 25 of 38 Rev. S Operator Actions Appendix D Form ES-D-2 Op-Test No.: NRC Scenario No.: 1 Event No.: 9 - 10 Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure). Time Position Applicantts Actions or Behavior BOP Manually align equipment. SRO Transitions to Step 21 AIER column and directs operators. RO NOTE: Enclosure 4 (NC Temperature Control) shall remain in effect until subsequent procedures provide alternative NC temperature Control NC temperature. REFER TO Enclosure 4 (NC Temperature EXAMINER NOTE: Enclosure 4 of this procedure is affached. BOP/RO Verify Pzr PORV and Pzr spray valve status as follows: a. All Pzr PORVS - CLOSED b. Normal Pzr spray valves - CLOSED c. At least one Pzr PORV isolation valve - OPEN RO Verify NC subcooling based on core exit TICs - GREATER THAN 00F BOP Verify main steamlines are intact as follows: All SIG pressures - STABLE OR INCREASING ALL SIGS - PRESSURIZED NUREG-1021 26 of 38 Rev. 8 Appendix D **Operator Actions** Form ES-D-2 Op-Test No.: NRC Scenario No.: I Event No.: 9 - 10 Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, S/G PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure).

Lmelfljlosition Applicant's Actions or Behavior

ROIBOP Verify SIG tubes are intact as follows:

Verify the following EMF trip 1 lights - DARK:

- 1 EMF-33 (Condenser Air Ejector Exhaust)
- IEMF-34 (SIG Sample)
- 1EMF-26 (Steamline IA)
- 1EMF-27 (Steamline IB)
- 1EMF-28 (Steamline iC)
- 1EMF-29 (Steamline iD)

All SIG levels - STABLE OR INCREASING IN A CONTROLLED MANNER

Determines that 1EMF-33 and 1EMF-27 are in alarm, and IB SIG level is increasing in an uncontrolled manner and informs SRO

- SRO Transitions to Step 25 RNO and directs operators.
- SRO IE any EMF trip 1 light is lit QB any SIG level is increasing in an uncontrolled manner, fljEN concurently:
 - Implements EPI1/N50001F-0 (Critical Safety Function Status Trees)
 - GO TO EPI1/AI50001E-3 (Steam Generator Tube Rupture).

SRO Transitions to EPI1IAI5OOOIE-3 and directs operators.

ROIBOP Monitor Enclosure 1 (Foldout Page)			
NUREG-102 1	27 of 38	Rev. S	
Appendix	0 Operator Actions	Form ES-D-2	
Op-Test No.: NRC Scenario No.: 1 Event No.: 9-10			

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure).

Time Position Applicant's Actions or Behavior ROIBOP Identify ruptured SIGs as follows:

- SIG level INCREASING IN AN UNCONTROLLED MANNER
 OR
- Chemistry or RP determines ruptured SIG by frisking the cation columns in the CT lab.
 OR
- The following EMF trip 1 lights LIT:
- •1EMF-26 (Steamline IA)
- •1EMF-27 (Steamline IB)
- •1EMF-28 (Steamline IC)
- •1EMF-29 (Steamline 10)
- Chemistry determines ruptured SIG using 1 EMF-34 (SIG Sample).
 - OR
- IF SIG Sampling is required to identify ruptured SIG(s), THEN:
- a. Ensure the following signals reset:
 - 1) Phase A Containment Isolations
- 2) CA System valve control
- 3) KC NC NI NM St signals

- b. Align all SIGs for Chemistry sampling.
- c. Notify Chemistry to sample all SIGs for activity.

ROVerify at least one intact SIG - AVAILABLE FOR NC SYSTEM COOLDOWN.NUREG-102128 of 38Rev. S[A~~endix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 1Event No.: 9 - 10
Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat -just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure). Time Jifiositioni Applicant's Actions or Behavior
RO Isolate steam flow from ruptured SIGs as follows:
 a. Verify all ruptured SIGs PORV - CLOSED b. Verify SIG(s) I B and 1 C - INTACT Determines 1 B SIG is ruptured and informs SRO SRO Perform the following: IE CA Pump #1 is the only source of feedwater, ItQi mainta~n steam flow to the CAPT from at least one SIG Determines this step is N/A and continues in RNO column. IE SIG IB is ruptured, THEN:
 a) Dispatch twooperators to unlock and close 1 SA-1 (Main Steam 18 To CAPT Maintenance Isol) (DH-624, FF-53, Rm 572) (Breakaway lock installed).
 b) IF 1 SA-1 cannot be closed, THEN dispatch two operators to unlock and close 1 SA-3 (S/G 1 B SM To CAPT Stop Check) (AM-Sal, DD-53, Rm 217) (Key #589 and #599).
3) IE SIG iC is ruptured, THEN:
Determines that step is N/A. NURE~~102 1 29of 38 Rev. 8 Appendix 0 Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 1 Event No.: 9 - 10
Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B S/G leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure). Time Position Applicants Actions or Behavior SRO Transitions to Step 4.c A/ER column and directs operators.
RO C. Isolate blowdown and steam drain on all ruptured SIG(s) as follows:
Determines this step is N/A for SIGs IA, iC, and 10.
SIGiB
1) Close 1 SM-76B (SIG 1 B OtIt Hdr Bldwn CN)

- 2) Verify the following blowdown isolation valves CLOSED:
 - a) 1BB-19A (SIG IB Bldwn Cont Isol Insd)
 - b) 1BB-1 SOB (SIG IB Bldwn Cont Isol Byp)
- c) 1BB-21B (SIG IB Bldwn Cont Isol Otsd)
- RO Close the following valves on all ruptured SIG(s):

(CRITICAL TASK)

MSIV

 MSIV Bypass valve EXAMINER NOTE: At some point after SIG I B MSIVs are isolated, SIG pressure will increase to the PORV lift setpoint. When that occurs the PORV will stick open. Isolation of the steam release path is a CRITICAL TASK. NUREG-1021 30of 38 Rev. S Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 1 Event No.: 9 - 10 Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG teaks by seat -just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure). Time Position Applicant's Actions or Behavior RO/BOP Control ruptured SIG(s) level as follows: a. Verify ruptured SIG(s) NIR level - GREATER THAN 16% (29% ACC). b. Isolate feed flow to all ruptured SIG(s) as follows: Determines this step is N/A for SIGs IA, 1 C, and 1 D. S/GIIB: 1) Close 1 CA-58A (CA Pmp A Disch To SIG I B Isol) 2) Close ICA-54B (CA Pmp 1 Disch To SIG IB Iso) c. IF AT ANY TIME ruptured SIG(s) NIR level is less than 16% (29% ACC), IijjN perform step 6. BOPIRO Verify Pzr PORV and isolation valve status as follows: a. Power to all Pzr PORV isolation valves - AVAILABLE b. All Pzr PORVs - CLOSED c. At least one Pzr PORV isolation valve - OPEN d. IF AT ANY TIME Pzr PORV opens due to high pressure while in this procedure, 'tiN, after Pzr pressure decreases to less than 2315 PSIG, perform Step 7.b (All Pzr PORVs - CLOSED) RO Verify main steamlines are intact as follows: All SIG pressures - STABLE OR INCREASING • All SIGs - PRESSURIZED NUREG~1021 31 of 38 Rev. 8 Appendix D **Operator Actions** Form ES-D-2 Op-Test No.: NRC Scenario No.: 1 Event No.: 9 - 10 Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure). Position Applicant's Actions or Behavior ROIBOP Control intact SIG levels as follows: a. Verify NIR level in all intact SIGs - GREATER THAN 11% (29% ACC).

- b. Throttle feed flow to maintain all intact SIG NIR levels between 11% (29% ACC) and 50%.
- C. Ensure CA suction source switchover criterion is monitored. REFER TO Enclosure 1 (Foldout Page)
- BOP Ensure S/I RESET:

a. ECCS

b. DIG load sequencers

C. IF AT ANY TIME a BIO occurs, THEN restart S/I equipment previously on
BOP Ensure the following containment isolation signals - RESET:
Phase A
Phase B BOP Establish VI to containment as follows:
a. Ensures 1VI-77B (VI Cont Isol) - OPEN
b. Verify VI pressure - GREATER THAN 85 PSIG NUREG-1021 32 of 38 Rev. S Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 1 Event No.: 9 - 10
Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat -just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure). ITime~tionI Applicant's Actions or Behavior BOP Verify all AC busses are energized by offsite power as follows:
• ATrain:
• FTA BIO NORM FDR FRM ATC" - CLOSED
• DIG IA BKR TO ETA" - OPEN
IETA-ENERGIZED
• FTB BIO NORM FDR FRM ATD" - CLOSED
• DIG I B BKR TO ETB" - OPEN
1ETB-ENERGIZED. BOP/RO Verify criteria to stop operating ND pumps as follows:
a. At least one ND pump - ON
b. NC pressure - GREATER THAN 285 PSIG
 c. Ensure all ND pump(s) not supporting Cold Leg Recirc - STOPPED d. IF AT ANY TIME NC pressure decreases to less than 285 PSIG in an uncontrolled manner, THEN restart ND pumps
SRO Verify ruptured SIG(s) - IDENTIFIED.
Determines IB SIG is the ruptured SIG.
RO Verify the following valves on all ruptured SIGs - CLOSED:
• MSIV
MSIV bypass valves
NUREG-1021 33 of 38 Rev. S Appendix D Operator Actions Form FS-D-2
Op-Test No.: NRC Scenario No.: 1 Event No.: 9 - 10
Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure). Time Position Applicants Actions or Behavior RO Verify all ruptured SIGs pressure - GREATER THAN 325 PSIG BOP WHEN "P-li PZR S/I BLOCK PERMISSIVE" status light (ISI-18) is lit, juiti:

a.	Depress	ECCS	steam	pressure	"BLOCK"	pushbuttons
----	---------	------	-------	----------	---------	-------------

- b. Verify main steam isolation blocked status lights (iS-i 3) LIT
- c. Maintain NC pressure less than 1955 PSIG using one of the following:
 - Pzr spray

OR

PzrPORV

ALL NOTE: NC Pump trip criteria based on NC subcooling does not apply after starting a controlled cooldown

After the low steamline pressure main steam isolation signal is blocked, Main Steam Isolation will occur if the high steam pressure rate setpoint is exceeded. 34 of 38 Rev. S

NUREG-102134 of 38Rev. SAppendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 1Event No.: 9 - 10

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure).

Lmme~tionI Applicants Actions or Behavior

RO EXAMINER NOTE: Cooldown and depressurization to minimize break flow is a CRITICAL TASK.

Initiate NC System cooldown as follows:

a. Determine required core exit temperature from the table below:

EXAMINER NOTE: Table is attached.

b. Verify the condenser is available as follows:

- "C-9 COND AVAILABLE FOR STM DUMP" status light (1SI-18) - LIT
- MSIV on intact SIG(s) OPEN
- c. WHEN "P-12 LO-LO TAVG" status light (1SI-18) is lit, THEN place the steam dump interlock bypass switches in "BYP INTK."

d. Verify steam dumps - IN PRESSURE MODE Determines steam dumps are in "T AVG" Mode and informs SRO.

- SRO Transition to Step 19.d RNO and directs operators. ROPlace steam dumps in pressure mode as follows: 1)Place "STM DUMP CTRL" in manual.
 - 2) Manually adjust the "STM DUMP CTRL" to match "STM DUMP CTRL" demand and "% STM DUMP DEMAND".
 - 3) WHEN demand on the "STM DUMP CTRL" is equal to the "% STM DUMP DEMAND", Ii:mN place the steam dumps in pressure mode.
- SRO Transition to Step 19.e NER column and direct operators.

RO EXAMINER NOTE: If Main Steam Isolation occurs during theNUREG-102135 of 38Rev. SAppendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 1Event No.: 9 - 10

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure).

Time Position Applicant's Actions or Behavior cooldown, the SRO will direct steam to be dumped from intact SIG PORVs per step 19.e RNO.

- RO e. Dump steam to condenser from intact S/G(s) at maximum rate while attempting to avoid a Main Steam Isolation.
 - L Verify main steam isolation blocked status lights (1 SI-i 3) LIT.
 - 9. Verify core exit TICs LESS THAN REQUIRED TEMPERATURE.

EXAMINER NOTE: Procedure will loop in Step 19 until the required temperature is reached per Step 19.g RNO.

- h. Stabilize core exit TICs LESS THAN REQUIRED TEMPERATURE.
- SRO Verify NC System cooldown in Step 19 COMPLETED.
- RO Verify ruptured SIG(s) pressure is under operator control as follows: a. All ruptured SIG(s) pressure - STABLE OR INCREASING.
 - b. IF AT ANY TIME ruptured SIG(s) pressure is decreasing while in this procedure, THEN perform Step 21.
- RO Verify NC subcooling based on core exit TICs GREATER THAN 200F.

NUREG-1021	3601 38	Rev. S
Appendix D	Operator Actions	Form ES-D-2
Op-Test No.: NRC Scenario No.: 1		Event No.: 9 - 10

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat -just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure).

TimePosition Applicant's Actions or Behavior

- BOP Depressurize NC System using PZR Spray as follows:
 - a. Verify normal Pzr spray flow AVAILABLE.
 - b. Verify Pzr level LESS THAN 76% (73% ACC).
 - c. Depressurize NC System with maximum available spray.
 - d. Verify subcooling based on core exit TICs GREATER THAN 0~ F.
 - e. Verify Pzr level LESS THAN 76% (73% ACC).
 - f. Verify NC pressure LESS THAN RUPTURED SIG(s) PRESSURE.

g. Verify Pzr level - GREATER THAN 11% (20% ACC). EXAMINER NOTE: SRO will loop back through this step until one of the conditions is met. At that time, the depressurization will be stopped. The SRO may determine that sprays are not effectively depressurizing the NCS and transition to Step 23.f RNO or Step 23.g RNO which transitions to Step 24 AIER and use Pzr PORVs instead.

Close the following valve(s):

1) Pzr spray valves

2) 1 NV-37A (NV Supply To Pzr Aux Spray) Observe Caution prior to step 26 and GO TO Step 26. EXAMINER NOTE: This is Step 24.

BOP EXAMINER NOTE: This is Step 2

Depressurize NC System using PZR PORV.

- a. Verify at least one Pzr PORV AVAILABLE
- b. Verify Pzr level LESS THAN 76% (73% ACC)
- c. Open one Pzr PORV.
- d. Verify subcooling based on core exit TICs GREATER THAN 00F
- e. Verify Pzr level LESS THAN 76% (73% ACC)

NUREG-102137 of 38Rev. SAppendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 1Event No.: 9 - 10

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat - just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure).

Time Position Applicant's Actions or Behavior

- f. Verify NC pressure LESS THAN RUPTURED SIG(s) PRESSURE
- 9. Verify Pzr level GREATER THAN 11% (20% ACC)
- h. Close Pzr PORV.
- i. Close Pzr spray valve(s).

EXAMINER NOTE: SRO will loop back through this step until one of the conditions is met. At that time, the depressurization will be stopped.

- RO Verify NC Pressure INCREASING.
- ALL CAUTION: S/I must be terminated when termination criteria are satisfied to prevent overfilling the ruptured SIG(S).

RO/BOP Verify S/I termination criteria as follows:

- a. NC subcooling based on core exit TICs GREATER THAN 00F.
- b. Verify secondary heat sink as follows:
- NIR level in at least one intact SIG GREATER THAN 11% (29% ACC).

OR

- Total feed flow available to SIG(s) GREATER THAN 450 GPM.
- C. NC pressure STABLE OR INCREASING.
- d. Pzr level GREATER THAN 11% (20% ACC).

BOP Stop S/I pumps as follows:

a. Stop NI pumps. NUREG-1021 38 of 38 Rev. 8 { Appendix D Operator Actions Form ES-D-2 1 Op-Test No.: NRC Scenario No.: 1 Event No.: 9 - 10

Event Description: Steam Generator Tube Rupture coincident with manual reactor trip, SIG PORV on B SIG leaks by seat-just prior to commencing the NC system cooldown in E-3 (upon reaching peak pressure). Time Position Applicant's Actions or Behavior

b. Ensure only one NV pump - ON BOP Verify VI pressure - GREATER THAN 50 PSIG

BOP Isolate NV S/I flowpath as follows:

- a. Verify the following valve OPEN:
 - 1 NV-252A (NV Pumps Suct From FWST)
 - 1 NV-253B (NV Pumps Suct From FWST)

b. Verify the following valves - OPEN

- 1NV-203A (NV Pumps A&B Recirc Isol)
- 1NV-202B (NV Pumps A&IB Reciro Isol)
- c. Close the following valves:
 - 1NI-9A (NV Pmp CIL Inj Isol)

• iNI-IOB (NV Pmp CIL Inj Isol) TERMINATE SCENARIO WHEN S,' FLOWED VERIFIED NOT REQUIRED

Classification:Site Area Emergency per 4.1.S.3 - Potential Loss of Either Nuclear CoolantSystem or Fuel Clad and Loss of Any Other Barrier. This EAL poses no threat to the safetyof plant personnel or the general public.NUREG-102139 of 38Rev. SAppendix DScenario Outline NRC Set 2Simulation Facility: CatawbaScenario No.: NRC-2Op-Test No:

Examiners: _____ Operators:

Objectives: To evaluate the applicants' ability to increase power using NOPs while maintaining Tave matched to Tref, and to use AOPs to respond to a failure of a feed water flow transmifter, a VCT level transmifler, loss of the running RN Pump and a spurious opening of a S/a PORV. The applicants will be evaluated using EOPs to respond to a loss of all AC power caused by a loss ofoffsite power and a failure of the DOs to power the safety busses.

Initial Conditions: Snap 160)

IA DIG is in Maintenance mode. IA ND Pump is tagged out. \$10 ID feed water flow, Channel I is 00\$ Unit 2 is in Mode 5.

Turnover: 50% power 8 EFPD 99lppm [B] 100% CPL Equilibrium Xe
IAF technicians are calibrating the Channel I S/U ID feed water flow transmitter.
D/G IA has been tagged out since the last shift for a routine Diesel Day.
ND Pump IA is tagged for motor oil change. All work is to be completed by the end of this shift.
Unit 2 is in a forced outage and has just entered Mode 5.
Enclosure I of OP/I/AI6100/03 has been completed through 2.34.

Thunderstorms are in the area and the load dispatcher has directed that Unit I increase power to 100% as ujekl as ossible due to rid supply problems.

Event Not Malt; No. Event Event
II Type* Description
I N/A N-BOP Dilute for power increase
2 N/A R-RO Increase power to 100%
3 xMT~Fo08 1-RO S/C ID feed water flow transmitter channel II fails high
Sv=120
4 XMT41VOI4 1-BOP VCT level transmifter fails high
sv=100
50VR-RNOIIC C-BOP Loss of operating RN pump
OFF
OVR~NO110
ON
6 MAL4MOO2O C-RO \$10 ID PORV fails open
SV=40
/ MAL~POC2 M Loss of all AC power, station blackout
ALL BKRS
MAL-EGBCUI
SV=0 NUDEC 1021 1 of 20 Dov 9
NUREG-1021 I 01 29 REV. 0 8 MAI - D00056 C-ROP IB DIG field fails to flash
NI IDEG. 1021 2 of 20 Pay B
Appendix D Operator Actions Form FS-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 1

Event Description: Increase power to 100%

- BOP Refer to OP/i /N61 50/009 (Boron Concentration Control) Enclosure 4.3 (Dilution).
- BOP Ensure the following valve control switches in "AUTO":
 - INV-242A (RMWST To B/A Blender Ctrl)
 - 1NV-181A (B/A Blender OtIt To VCT)
- BOP Adjust the total makeup batch counter to the desired volume of reactor makeup water to be added.
- SOP Place the "NC MAKEUP MODE SELECT' switch to the "DILUTE" position.
- SOP Adjust the controller for 1 NV-242A (RMWST To B/A Blender Ctrl) to the desired flow.
- BOP Ensure 1NV-242A (RMWST To B/A Blender Ctrl) controller in "AUTO".
- SOP Ensure at least one reactor makeup water pump is in "AUTO" or "ON".

NUREG- 1021	3 of 29	Rev. B
Appendix D	Operator Actions	Form ES-D-2
if Position	Applicant's Actions or Be	havior

- BOP NOTE: If necessary, dilution can be manually secured at any time by placing the "NC MAKEUP CONTROL" switch to the "STOP" position.
 - BOP Place the "NC MAKEUP CONTROL" switch in the "STARr' position.
 - BOP Verify the following valves open:
 - 1NV-242A (RMWST To B/A Blender Ctrl)
 - 1NV-1B1A (B/A Blender OtIt To VCT)
 - BOP If in "AUTO", verify the reactor makeup water pump starts.
 - BOP When the desired volume of reactor makeup water is reached on the total makeup batch counter, ensure the following valves close:
 - 1 NV-242A (RMWST To B/A Blender Ctrl)
 - 1NV-181A (B/A Blender Ott To VCT)

If automatic makeup is desired, refer to Enclosure 4.1 (Automatic Makeup).

NUREG-1021	4 of 29	Rev B
Appendix D	Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 2

Event Description: Dilute for power increase Position Applicant's Actions or Behavior

RO	Refer to OP/i /B16300/OOI,	Turbine Generator, Enclosure 4.2,
	Section 2.2	

- RO Depress the "Load Rate" pushbutton and verify it illuminates.
- RO Input the desired load rate on the numeric keypad and verify the load rate appears on the Variable Display.
- RO Depress the "Enter" pushbutton.
- RO Depress the "Target" pushbutton and verify it illuminates.
- RO Depress the "Enter" pushbutton.
- RO To start load increase, depress the "Go" pushbutton and verify it illuminates.
- ALL SIG blowdown changes should be coordinated with Secondary Chemistry.

NUREG-1021	S of 29	Rev. 8
Appendix D	Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 3

Event Description: S/G 1 D feed water flow transmitter channel II fails highTimePositionApplicant's Actions or Behavior

- RO Recognize indications that SIG 1 D feed water flow transmitter channel II failed high based on the following indications and informs the SRO:
 - 1AD-4, DIS (DFCS NOT IN AUTO) annunciator- LIT
 - 1CFP 5070 failed to off-scale HIGH
 - SIG 1 D feed water controllers swapped to manual
- RO Refer to OP/1IBI61OOIO1OE (Annunciator Response For Panel 1AD-4) Section D15 and perform actions.

EXAMINER NOTE: Annunciator response is attached.

- RO Control S/G level and CF flow by manual operation of controllers.
- BOP Notify IAE to determine and correct cause of DFCS malfunction.

EXAMINER NOTE: This will not be repaired for the remainder
of the scenario and manual control will be required.NUREG-10216 of 29Rev. SAppendix DOperator ActionsForm ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 4

Event Description: VCT level transmifter fails high TimePosition Applicant's Actions or Behavior BOPRecognize indications that VCT level transmitter 1NVP5760 failed high based on the following indications and informs SRO:

- 1AD-7, H/i (VCT HI LVL) annunciator LIT
- Computer alarm screen indicates 1NVP5760 at 100%
- Computer alarm screen for 1 NV-i 72A (3 Way Divert To

BOP Refer to OP/i IB/6100/O1OH (Annunciator Response For Panel 1AD-7) Section H/i (VCT HI LVL) and performs actions.

EXAMINER NOTE: Annunciator response is attached.

EXAMINER NOTE: This will not be repaired for the remainder of the scenario.

NUREG-1021	7 of 29	Rev. S
Appendix D	Operator Actions	Form ES-D-2
Time Position	Applicant's Actions or	Behavior

- BOP Recognize indication for loss of operating RN Pump and informs SRO.
 - 1AD-12, A/2 (RN ESSENTIAL HDR A PRESSURE -LO) annunciator LIT
 - IAD-i 2. AI5 (RN ESSENTIAL HDR B PRESSURE -LO) annunciator LIT

EXAMINER NOTE: Annunciator Response is attached.

EXAMINER NOTE: BOP may start an idle RN pump per the

annunciator response procedure.

EXAMINER NOTE: Starting an idle RN pump is a CRITICAL

STEP.

- SRO Implements APIOIA'5500120 (Loss of Nuclear Service Water) and directs operators.
- BOP Start idle RN Pumps as required.
- RO Ensure Unit 1 and Unit 2 OATC monitors Enclosure 1 (Foldout Page).
- BOP Verify each operating RN pump discharge flow GREATER THAN 8,600 GPM.
- BOP Verify each operating RN pump discharge flow LESS THAN 23,000 GPM.

BOP	Ensure RN pumps - IN OPEIRA	TION AS NEEDED.
NUREG-1021	S of 29	Rev. 8
Append	ix D Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 5

Event Description: Loss of operating RN Pumpifi PositionApplicant's Actions or BehaviorBOPEnsure proper alignment of RN to KC Hxs as follows:

- a. Verify RN ALIGNED TO IN SERVICE KC HX(S).
 b. Ensure KC Hx OtIt Mode switches PROPERLY ALIGNED.
- BOP Verify each operating RN pump discharge flow GREATER THAN

BOP Verify RN available to all Unit 1 and Unit 2 DIG(s).

BOP Determine VCIYC status as follows:

• Verify VCIYC - ALIGNED TO OPERATING RN TRAIN.

 Verify YC Chiller - RUNNING. 					
ALL	Determine	and correct	loss of RN	train.	
iruasa- 1021		9 of 29		Rev.	S
Append	dix D	Operator	Actions		Form ES-D-2

LOp-Test No.: NRC ScenarioNo.:2 EventNo.:5

Event Description: Loss of operating RN Pump Position Applicant's Actions or Behavior

- SRO Ensure compliance with appropriate Tech Specs:
 - 3.6.5 (Containment Air Temperature)
 - 3.6.6 (Containment Spray System)
 - 3.6.17 (Containment Valve Injection Water System)
 - 3.7.5 (Auxiliary Feedwater System)
 - 3.7.7 (Component Cooling Water System)
 - 3.7.8 (Nuclear Service Water System)
 - 3.7.10 (Control Room Area Ventilation System)
 - 3.7.11 (Control Room Area Chilled Water System)
 - 3.8.1 (A.C. Sources Operating)
 - 3.8.2 (A.C. Sources Shutdown)
- SRO Determine required notifications:
 - REFER TO RP/0IA/50001001 (Classification of Emergency)
 - REFER TO RPIOIBI5000I01 3 (NRC Notification Requirements)
- BOP Notify Environmental Chemistry of any RN pump shifts that have occurred.

ALL	Determine long term plant state	tus. RETURN TO procedure in	effect.
NUREG-1021	10 of 29	Rev. S	
Append	lix 0 Operator Actions	s Form ES-D-2	

Op-Test No.: NRC Scenario No.: 2 Event No.: 6

Event Description: SIG 10 PORV fails to intermediate positionTimePositionApplicant's Actions or Behavior

RO Recognize indication that SIG 1 D PORV failed open and informs SRO.

isv-i (SIG iD PORV) indicates OPEN.

EXAMINER NOTE: OMP 1-7 allows RO to isolate a known leak without SRO permission and inform SRO of isolation actions performed.

- SRO Directs RO to close isv-i (SIG 10 PORV)
- RO Attempts to close PORV by:
 - Places "SG 1 D PORV CTRL MODE" switch in "MANUAL" position.
 - Positions isv-i (SIG 10 PORV) controller to zero output.

Informs SRO) SIG 10 PORV will not go CLOSED.
SRO Directs RO) to close 1 SV-25B (SIG 1 D PORV ISOL).
RO Places 1 S	V-25B (SIG 1 D PORV ISOL) in the "CLOSE" position.
Informs SRO) that SIG iD RORV isolation valve is CLOSED.
SRO Refers to	Tech Specs:
• TS 3.6	5.3 (Containment Isolation Valves)
TS 3.7.4	(Steam Generator Power Operated Relief Valves
(SG POR	Vs))
EXAMINER I	NOTE: SRO may implement AP111X5500128
(Secondary	Steam Leak) at this point but it is not required.
Required ac	tions are on the subsequent pages of this section.
NUREG-1021	11 of 29 Rev. S
Appendix D	Operator Actions Form ES-D-2
Op-Test No.: NRC Scena	rio No.: 2 Event No.: 6
Event Description: SIG 1	D PORV fails to intermediate position
Time Position	Applicant's Actions or Behavior
EXAMINER I the SRO ent	NOTE: The following actions are provided in case ers APIIIA'5500128 (Secondary Steam Leak).
SRO Implemen operators. OATC Monitor E RO Verify turb RO Verify the	ts AP/11AI5500128 (Secondary Steam Leak) and directs inclosure 1 (Foldout Page). ine - ONLINE. following:
• Reactor	or power - LESS THAN OR EQUAL TO 100%
POWE	R.
• I-avg-	WITHIN 1.50F OFT-Ref
RO Verify prop	per reactor response as follows:
• Contro	ol rods - IN "AUTO" AND STEPPING IN
• PIR ne	eutron flux - DECREASING.
ALL IF AT ANY	TIME reactor power is greater than 100%, THEN
perform Ste	p 3 RNO.
RO Verify Pzr	level - STABLE OR INCREASING.
ALL IF AT ANY	TIME Pzr level is decreasing in an uncontrolled manner,
NUREG-1021	12 of 29 Rev. S
Appendix D	Operator Actions Form ES-D-2
Op-Test No.: NRC Scena	rio No.: 2 Event No.: 6
Event Description: SIG 1	D PORV fails to intermediate position
Time Position	Applicantts Actions or Behavior
THEN RETU	RN TO Step 6.
BOP IF AT ANY	TIME VCT level goes below 23%, THEN align NV
pump suctio	n to FWST as follows:
a. Op	en the following valves:
• 1NV-	252A (NV Pumps Suct From FWST)

• 1NV-253B (NV Pumps Suct From FWST).

- b. Close the following valves:
- INV-188A (VCT OtIt Iso)
- 1 NV-i 89B (VCT OtIt Isol).
- ALL Attempt to identify and isolate leak as follows:
 - a. Verify the following conditions NORMAL
 - Containment temperature
 - Containment pressure
 - Containment humidity
 - Containment floor & equipment sump level.
 - b. Dispatch operators to locate and identify source of steam leak.
- RO c. Verify SIG PORVs CLOSED.

	Informs SRO that 1 D SIG PO	ORV is NOT CLOSED.
SRO	Transitions to Step 9.c RNO.	
NUREG-1021	13 of 29	Rev. S
Append	ix D Operator Actions	Form ES-ID-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 6

Event Description: SIG 1 D PORV fails to intermediate position ifi Position Applicant's Actions or Behavior

RO

- C. IF SIG pressure is less than 1090 PSIG, THEN perform the following: 1)Close affected SIG PORV.
 - 2)IF SIG PORV is still open, THEN:
 - a) Close affected SIG PORV isolation valve.
- RO Informs SRO that iD SIG PORV isolation valve is closed.
- SRO Transitions to Step 9.d A/ER column.

RO d. Verify condenser dump valves - CLOSED.

- e. Verify atmospheric dump valves CLOSED.
- f. Verify CAPT #1 OFF.
- g. IF leak is suspected to be in the doghouse, THEN close the following valves: Outside OH:
 - 1SM-74B (SIG IB Ott Hdr Bldwn CN)
 - 1SM-75A (SIG iC Ott Hdr Bldwn CN).
- SRO Determine required notifications:
 - REFER TO RPIO/A'50001001 (Classification of Emergency)
- REFER TO RPIOIBI5000I0I 3 (NRC Notification Requirements). BOP Notify RP of leak. NUREG-1021 14 of 29 Rev. B Appendix D Operator Actions Form ES-D-2

Op-Test No..: NRC Scenario No.: 2 Event No.: 6

Event Description: SIG 1 D PORV fails to intermediate position

_meIffl~osnion		
NUREG-1021	15 of 29	Rev. S

Appendix D Operator Actions Form ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 7 and 8

Event Description: Loss of all AC power, station blackout; 1 B D/G field fails to flash. Ljlkfltion Applicant's Actions or Behavior EXAMINER NOTE: Main Generator will trip approximately 5 seconds after loss of switchyard resulting in a loss of both onsite and offsite power.

- ALL Recognize indications of a loss of the switchyard followed by a loss of in-house power:
 - Control rods stepping in, (then reactor trip)
 - Speed I megawatt meter shifts to speed indication, (then turbine trip)
 - Steam dumps open
 - NCPORVsopen

EXAMINER NOTE: SRO may enter E-O initially, but will transition to ECA-O.O at Step 4 of E-O.

- SRO Implements EPI1/A/5000/ECA-O.O (Loss of All AC Power) and directs the operators.
- SRO CSF Status trees should be monitored for information only. Do not exit this procedure to implement any of the CSF procedures.
- RO Verify Reactor Trip:
 - All rod bottom lights LIT
 - All reactor trip and bypass breakers OPEN

hR amps - DECREASING
NUREG-1021
 16 of 29
 Rev. B
 Appendix D
 Operator Actions
 Form ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 7 and B

Event Description: Loss of all AC power, station blackout; 1 B DIG field fails to flash.TimePositionApplicant's Actions or Behavior

- RO Verify Turbine Trip:
 - All Turbine Stop valves CLOSED
 - All Turbine Control valves CLOSED.
- BOP Establish NC pump seal injection from the SSF: (CRITICAL TASK)
 - Notify Security Officer at SSF to perform Enclosure 2 (Establishing NC Pump Seal Injection From the SSF).
 - Dispatch operator to 1 ETA switchgear room to swap power supply for 1 EMXS. REFER TO Enclosure 1 (Swapping Power Supply for 1 EMXS).
 - Dispatch operator to SSF to establish NC pump seal injection. REFER TO Enclosure 2 (Establishing NC Pump Seal Injection From the SSF).

BOP Verify NC System is isolated as follows:

		 a.All Pzr PORVs - CLOSED. b.All the following letdown isolation valves - CLOSED 1NV-10A (Letdn Orif IB Cont Isol) 1NV-11A (Letdn Orif iC Cont Isol) 1NV-13A (Letdn Orif IA Cont Isol) 1NV-1A (NC Letdn To Regen Hx Isol) 1NV-2A (NC Letdn To Regen Hx Iso) 1NV-135 (ND Flow To Letdn Hx).
NURE Time	BOP G- 102 Appen Positic SROTi SOP	Determines that 1NV-1A and 1NV-2A are not closed and informs SRO. 1 17 of 29 Rev. S Idix D Operator Actions Form ES-D-2 on Applicant's Actions or Behavior ransitions to Step 5.b RNO column and continues. Manually close valves.
	SOP	Manually closes 1NV-1A and 1NV-2A and informs SRO.
	SRO BOP	Transitions to Step S.c NER column and continues. c. All the following excess letdown isolation valves - CLOSED:
		• 1NV-1228 (Loop C To Exs Letdn Hx Iso)
		• 1NV-123B (Loop C To Exs Letdn Hx 1501)
		• 1NV-124B (Excess letdn Press Ctrl).
	BOP SOP	Verify total CA flow - GREATER THAN 450 GPM. Attempt to restore power to 1 ETA or 1 ETS as follows:
		a. Manually start DIGs from control room.
		b. Verify DIG load sequencer-AUTOMATICALLY LOADING BUS.
	BOP	Determines that DIG IB is running but DIG IB breaker is open and sequencer is not loading the bus and informs SRO.
NURE	SRO G-1021 Appen	Transitions to Step 7.b RNO and directs operators. I 16 of 29 Rev. S Idix D Operator Actions Form ES-D-2
Op-Te	est No.:	: NRC Scenario No.: 2 Event No.: 7 and 8
Event Time	Descri e Pos	ption: Loss of all AC power, station blackout; 1 B DIG field fails to flash. Applicant's Actions or Behavior
	BOP	Perform the following for the affected train(s):
		1 ETA (Not available)
		EXAMINER NOTE: IETA will be the bus that will be returned
		later in the scenario, but currently without power.

1ETB:

- 1) IE 1 ETB is de-energized, THEN:
 - a) Ensure the following breakers OPEN:
 - "ETB NORM FDR FRM ATD"

- "ETB ALT FDR FRM SATB".
- b) IF 1 ETB is still de-energized, THEN:
 - (1) Depress and hold the DIG "OFF" pushbutton.
 - (2) Dispatch operator to open 1 EDF-FO1 F (Diesel Generator Load Sequencer Panel 1 DGLSB) (AB-560, BB-46, Rm 372).
 - (3) WHEN 1EDF-FO1F is open, THEN release the DIG "OFF" pushbutton.
- SRO Transitions to step 7.c NER column and continues.
- BOP C. Verify 1ETAor 1ETB- ENERGIZED.
 - Determines that neither bus is energized and informs SRO.
- SRO Transitions to Step 7.c RNO.
- SRO GO TO Step 8.

NUREG-1021	19 of 29	Rev. S
Appendix D	Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 7 and 8

Event Description: Loss of all AC power, station blackout; 1 B DIG field fails to flash. TimePosition Applicant's Actions or Behavior SROTransitions to Step 8 AIER column and continues. BOPVerify CA supply is adequate as follows: a. VI pressure - GREATER THAN 85 PSIG. b. Ensure proper CA pump suction supply, REFER TO AP/1kl5500106 (Loss Of SIG Feedwater).

- SRO Implement RP/01N5000101 (Classification Of Emergency).
- ALL WHEN power is restored to one essential bus, THEN GO TO Step 31.
- BOP Verify S/I status as follows:

a. "SAFETY INJECTION ACTUATED" status light (1SI-13) - LIT. Determines that light is NOT lit and informs SRO.

- SRO Transitions to Step 11 .a RNO and directs operators.
- SRO Perform the following:
 - i) IF AT ANY TIME an S/I signal is generated while in this procedure, THEN, after the 1 minute time delay, reset ECCS.

2) GOTOStepl2.

BOP/SRO	Dispatch operator with a scree	ewdriver to load shed essential b	usses
and	verify lockout relay status. RI	EFER TO Enclosure 4 (Manual	
Loa	d Shed of 1 ETA and 1 ETB.		
NURHG-1021	20 of 29	Rev. S	
Appendix	D Operator Actior	ns Form ES-D-2	

Op-Test No.: NRC ScenarioNo.:2 EventNo.:7and8

Event Description: Loss of all AC power, station blackout; IB DIG field fails to flash.

LOp-Test No.: NRC ScenarioNo.:2 EventNo.:7and8

Event Description: Loss of all AC power, station blackout; 1 B DIG field fails to flash. TimePosition Applicant's Actions or Behavior BOP/SRORestore power to at least one essential bus as follows:

- a. Dispatch operator and maintenance IAE personnel to DIG rooms to prepare DIGs for start. REFER TO the following:
 - Enclosure 20 (Local Start of Diesel Generator IA)
 - Enclosure 2i (Local Start of Diesel Generator IB).

• EM/1IAI52001003 (Troubleshooting Cause For a Diesel Generator Failing to Start)

b. Verify switchyard - ENERGIZED.

C. WHEN time and manpower permit, THEN perform Enclosure 22 (Switchyard Battery Conservation).

NOTE: Oftsite power may be unavailable for reasons other than switchyard de-energized.

d. Verify at least one of the following power sources available:

- Offsite Power
 - OR
- DIGIA
- OR
- DIGiB
- e. Verify offsite power AVAILABLE.

EXAMINER NOTE: Crew may determine that offsite power is available however, the intent is to restore power from IA DIG. Whether the crew responds that offsite power is available or is

NOT available, the final result will be the same.

In the former case, they will be directed to restore power from offsite per Enclosure S (Restoration of Offsite Power) and in step i of that enclosure, the dispatcher will inform them NOT to reconnect to the switchyard at this time. For the latter case, Step 13.e RNO directs the crew to restore power to one essential bus per Enclosure 5 (Power Restoration From DIGs). NUREG-1021 22 of 29 Rev. 6

Appendix ID Operator Actions Form ES-D-2

Op-Test No.: NRC ScenarioNo.:2 EventNo.:7and8

Event Description: Loss of all AC power, station blackout; 1 B D~G field fails to flash.TimePositionNUREG-102123 of 29Appendix 0Operator ActionsForm ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 7 and 8

Event Description: Loss of all AC power, station blackout; 1 B D/G field fails to flash. Time Position Applicant's Actions or Behavior

SRO Transitions to Step 14 A/ER column and directs operators.

SOP Verify one train of VCIYC - OPERATING.

Determines neither train of VC'YC is in service and informs SRO.

SRO Transitions to Step 14 RNO and directs the operators. BOP/SRO Perform the following:

- a. Request IAE to open the doors on control room cabinets within 30 minutes of the loss of power. REFER TO Enclosure 9 (Control Room Cabinet Doors to Open).
- b. IF Unit 2 power is available, THEN notify the Unit 2 Supervisor to dispatch operator to align Unit 2 power to 1 EMXG and 2EMXH as follows:

NOTE: The following breakers are kirk-key interlocked.

EXAMINER NOTE: The following would be performed by Unit

2 personnel. Steps will not be completed for this scenario.

- 1EMXG:
- 1) Open 1 EMXG-FO3A (Incoming Breaker Fed from Unit 1 Load enter 1 ELXA).
- 2) Close 1EMXG-FO6A (Incoming Breaker Fed From Unit 2 Load Center 2ELXA).
- 3) Close 2ELXA-5B (MCC I EMXG Unfl 2 Feeder).

• 2EMXH:

1) Open 2EMXH-FOSA (Incoming Breaker Fed From Unit 1 Load Center 1ELXB).

2) Close 2EMXH-FO3A (Incoming Breaker Fed From Unit 2 Load Center 2ELXB). 3) Close 2ELXB-5B (MCC 2EMXH Unit 2 Feeder) NUREG-1021 24 of 29 Rev. S Appendix D Operator Actions Form ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 7 and 8

Event Description: Loss of all AC power, station blackout; 1 B DIG field fails to flash.TimePositionApplicant's Actions or Behavior

- BOP C. Place one train of VCIYC in service with power from Unit 2. REFER TO 0P101N645011 1 (Control Room Area VentIchilled Water System).
 - d. WHEN one train of VCIYC is returned to service, THEN close the cabinet doors previously opened.

ROIsolate NC pump seals as follows:

a. Notify operator at SSF to ensure 1 NV-89A (NC Pmps Seal Ret Cont Isol) - CLOSED.

b. Notify operator at SSF to verify standby makeup pump - ON. ROVerify SIGs are isolated as follows:

a. Verify the following SM valves - CLOSED:

• AIIMSIVs

- All MSIV bypass valves.
- SRO Transitions to Step 16.a RNO and directs the operators. RO Perform the following:
 - 1) Manually close valve(s).

Manually closes all MSIVs and MSIV bypasses and informs SRO.

- RO b. Verify the following CF valves CLOSED:
 - All CF control valves
 - All CF bypass control valves.

c. Verify all blowdown flow control valves - CLOSED.

NUREG-1021	25 of 29	Rev. S
Appendix D	Operator Actions	Form ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event NQ.: 7 and 8

Event Description: Loss of all AC power, station blackout; 1 B DIG field fails to flash.

- MI Position Applicant's Actions or Behavior
- RO d.Dispatch operator to close the following valves:
 - •1SM-77A (SIG IA OtIt Hdr Bldwn CN)
 - •1SM-768 (SIG IB Otit Hdr Bldwn CN)
 - •1SM-75A (SIG iC OtIt Hdr Bldwn CN)
 - •1SM-74B (SG iD OtIt Hdr Bldwn CN).
 - RO Verity main steamlines are intact as follows:
 - All SIG pressures STABLE OR INCREASING
 - All S/Gs PRESSURIZED.

BOP Verify SIG tubes are intact as follows:

The following EMF trip 1 lights - DARK:

- 1EMF-26 (Steamline IA)
- 1EMF-27 (Steamline IB)
- 1EMF-28 (Steamline iC)
- 1EMF-29 (Steamline iD)
- 1 EMF-33 (Condenser Air Ejector Exhaust)
- IEMF-34 (SIG Sample).

EXAMINER NOTE: At this point, the operator at the IA DIG will call the Control Room to inform them that the DIG is ready to start per Enclosure 20. The crew will return to step 13 and restore power per Enclosure 5. Once power is restored to I ETA, the crew will immediately transition to Step 31 and the scenario will be terminated. (CRITICAL TASK) NUREG-1021 26 of 29 Rev. S **Operator Actions** Appendix D Form ES-D-2 Applicant's Actions or Behavior Time Position RO Control intact S/G levels as follows: a. Maintain total CA flow from CA Pump #1 less than 1000 GPM while feeding SIG(s) in the following steps.

- b. Verify CA flow to all intact SIGs INDICATING FLOW.
- c. Verify NIR level in all intact SIGs GREATER THAN 11% (29% ACC).
- d. Throttle CA flow from the control room to maintain SIG NIR level in all intact SIGs between il% (29% ACC) and 50%.
- e. IF AT ANY TIME NR level in any SIG increases in an uncontrolled manner, THEN:
- EXAMINER NOTE: The remainder of this step is NIA. BOP Reduce nonvital DC loads as follows:

a. Dispatch operator to open breakers for Group A, large nonvital DC loads. REFER TO Enclosure 12 (DC Loads to be Shed During Loss of All AC Power), Step 1.
 b. WHEN the large nonvital DC loads are removed, THEN notify operator to open additional breakers to maintain the required voltage on the following "DIST CTR VOLTS" meters (1 MC-8). REFER TO Enclosure 12 (DC Loads to be Shed During Loss Of All AC Power), Step 2:
1CDA greater than 105 volts
• 1 CDB greater then 1 Os volts
1 DPD greater than 210 volts. NUREG-1021 27 of 29 Rev. 8 Appendix D Operator Actions Form ES-D-2
Op-Test No.: NRC Scenario No.: 2 Event No.: 7 and 8
Event Description: Loss of all AC power, station blackout; 1 B D/G field fails to flash [Time Th~ositionI Applicant's Actions or Behavior BOP Verify criteria requiring NC System cooldown and depressurization as follows:
 Standby makeup pump - OFF OR
Pzr level - DECREASING IN AN UNCONTROLLED MANNER. NUREG- 1021 26 of 29 Rev. 8 Appendix D Operator Actions Form ES-D-2
Op-Test No.: NRC Scenario No.: 2 Event No.: 7 and 8
Event Description: Loss of all AC power, station blackout; 1 B D/G field fails to flashTimePositionApplicant's Actions or Behavior
BOP Verity the following Monitor Light Panel Group 5 St lights - LIT:
a. VP, VQ, and VY:

- G-lorG-11
- G-2orG-l0
- G-3 or G-9
- G-4 or G-S
- G-5 or G-7
- H-2orH-10
- H-3 or H-9
- H-4 or H-8
- H-S or H-7
- H-il or 1-2
- 1-3 or 1-10
- 1-6 or 1-7
 - c. VUCDT:
- L-12 or M-1

SRO Transitions to Step 22.b RNO and directs the operators.

BOPISRO Dispatch operator to locally close outside containment isolation valve for affected penetration. REFER TO Enclosure 16 (Local Isolation of Phase B Isolation Valves). NUREG-1021 29 of 29 Rev. B Appendix D **Operator Actions** Form ES-D-2

Op-Test No.: NRC Scenario No.: 2 Event No.: 7 and 8

Event Description: Loss of all AC power, station blackout; IB DIG field fails to flash. Time Position Applicant's Actions or Behavior

- ALL CAUTION: Lowering SIG pressures less than 65 PSIG will cause injection of nitrogen from the CLAs into the NC System.
 - NOTE: Pzr level may be lost and reactor vessel head voiding may occur due to depressurization of SIGs.

RO Depressurize intact SGs to 165 PSIG as follows:

- a. Verify SIG NIR level in at least one intact SIG GREATER THAN 11% (29% ACC).
- b. Ensure operator monitors Enclosure 13, (SIG Depressurization Limits) throughout the SIG depressurization.

c. Dump steam from all intact SIG PORVs at maximum rate. EXAMINER NOTE: A SIG is isolated because of leaking PORV and should not be used during the cooldown.

- d. Verify all NC T-Colds GREATER THAN 2800F.
- e. Maintain at least one intact SIG NIR level greater than 11% (29% ACC).
- f. Verify all intact SIG pressures LESS THAN 165 PSIG.
- RO Determines all intact SIG Pressures are not less than 165 psig and informs SRO.
- SRO Transitions to 23.f RNO and directs operators
 - 1) WHEN all intact SIG pressures are less than 165 PSIG, THEN perform Step 23.q. 2) ~GOflOStep24.

Transitions to Step 24 AIER column and directs operators. SRO NUREG-1021 30 of 29 Rev. S

Appendix D **Operator Actions** Form ES-D-2

Op-Test No.: NRC ScenarioNo.:2 EventNo.:7and8

Event Description: Loss of all AC power, station blackout; IB DIG field fails to flash.

- Applicant's Actions or Behavior Time Position
 - RO Verify the reactor is subcritical as follows:
 - IR SUR ZERO OR NEGATIVE

• S/R SUR - ZERO OR NEGATIVE.

- BOPVerify S/I signal status as follows:
 - a. Verify SI! HAS BEEN ACTUATED
 - b. After 1 minute time delay, reset ECCS.

BOP Verify Phase A Containment Isolation as follows:

a. Phase A "RESET' lights - DARK

b. Monitor Light Panel Group 5 St lights - LIT. Determines that Monitor Light Panel is not in proper alignment and informs SRO.

- SRO Transitions to Step 26.b RNO and directs the operators. BOP Perform the following:
 - 1) Manually close valves
 - 2) ILvalves cannot be closed, THEN dispatch operator to close affected outside containment isolation valves. REFER TO Enclosure 15 (Local Isolation of Phase A Isolation Valves).
- SROTransitions to Step 27 A/ER column and directs the operators.NUREG-102131 of 29Rev. BAppendix DOperator ActionsForm ES-D-2TimePositionApplicant's Actions or Behavior
 - BOP Verify containment pressure HAS REMAINED LESS THAN 3 PSIG.

Ensure H2 recombiners remain de-energized on power restoration as follows:

a. Verify H2 recombiners - HAVE BEEN PLACED IN SERVICE.

Determines that H2 recombiners have not been placed in service and informs SRO.

- SRO Transitions to Step 28.a RNO and then goes to Step 29.
- BOP Verify containment radiation levels are normal as follows:
 - + 1 EMF-53A (Containment Train A) LESS THAN 10 R'HR
 - 1 EMF-53B (Containment Train B) LESS THAN 10 R'HR.
- RO Verify Core exit TICs LESS THAN 1 2000F.
- BOP Verify either 1 ETA or 1 ETB undervoltage status lights (1SI-14) DARK.

EXAMINER NOTE: DIG IA should be running and supplying I ETA prior to reaching this step due to earlier directions to restore power per Enclosure 20 (Local Start of Diesel Generator IA).

TERMINATE SCENARIO WHEN POWER RESTORED TO IETANUREG-102132 of 29Rev. 8Classification: SITE AREA EMERGENCY - 4.5.S.1 - All AC electrical power to the
vital busses (Offsite and Onsite) has been lost for more than 15
minutes. This condition, by itselt posed no immediate threat to
public safety.NUREG-102133 of 29Rev. SAppendix DOperator ActionsForm ES-D-2

Appendix D Operator Actions Form E Op-Test No.: NRCScenario No.: 3 Event No.: 1

Event Description: Boration for load decrease Time [ffl~osition Applicant's Actions or Behavior

- INV-238A (B/A Xfer Pmp To Blender Ctrl)
- 1NV-186A (B/A Blender OtIt To VCT Ott)
- BOP Adjust the boric acid batch counter to the desired volume of boric acid to be added.
- BOP Place the "NC MAKEUP MODE SELECT' switch in "BORATE".
- BOP Adjust the oontroller for 1 NV-238A (B/A Xfer Pmp to Blender Ctrl) controller to the desired flow.
- BOP Ensure 1 NV-238A (B/A Xfer Pmp to Blender Ctrl) controller in "AUTO".
- BOP NOTE: If necessary, Boration can be manually secured at any time by placing the "NC MAKEUP CONTROL" switch to "STOP" position.

Ensure at least one boric acid transfer pump in "AUTO" or "ON".

BOP Place the "NC MAKEUP MODE SELECT' switch in "START' position.

BOP Verify the following valves open:

• 1 NV-238A (B/A Xfer Pmp To Blender Ctrl VIv)

1 NV-i 86A (B/A Blender Ott To VCT Ott)
NUREG-1021 1 of 32 Rev. 8
Appendix D Operator Actions Form ES-D-2
Op-Test No.: NRCScenario No.: 3 Event No.: 1

Event Description: Boration for load decrease[ifflime PositionApplicant's Actions or Behavior

BOP When the desired volume of boric acid is reached on the boric acid batch counter, ensure the following valves close:

- 1 NV-238A (B/A Xfer Pmp To Blender Ctrl Vlv)
- 1NV-186A (B/A Blender OtIt To VCT OtIt)
- BOP IF desired, flush the makeup line as follows:
 - Open the following valves:
 - 1 NV-242A (RMWST To B/A Blender Ctrl)
 - I NV-i 86A (B/A Blender OtIt To VCT OtIt)
 - Ensure one reactor makeup water pump is in "ON".
 - WHEN 20 gallons of makeup water have been flushed through the makeup line, close the following valves:
 - I NV-242A (RMWST To B/A Blender Ctrl)
 - 1 NV-i 86A (B/A Blender OtIt To VCT OtIt)
 - Place the following valve control switches in "AUTO":
 - 1 NV-242A (RMWST To B/A Blender Ctrl)
 - 1 NV-i 86A (B/A Blender OtIt To VCT OtIt)
 - IF NOT required for current plant operation, place the reactor makeup water pump started in earlier step in "AUTO".

BOP	IF automa	tic makeup is desir	ed, refer	to Enclosure 4.1 (Auto	matic	
NUREG-102 Apper	ndix D	2 of32 Operator Actio	Re	ev. 8 Form ES-D-2		
Op-Test No.	: NRC Scena	ario No.: 3 Event N	o.: 2			
Event Descr Posit	iption: Decre ion /	ase turbine load Applicant's Actions o	or Behavi	ior		
RO	Refer to O Section 2.4	PI1/B163001001, T	urbine G	enerator, Enclosure 4.2) - 1	
RO	Depress th	e Load Rate" push	button ar	nd verify it illuminates.		
RO	Input the o	desired load rate or	n the nun	neric keypad and verify	the load	
RO	rate appears Depress th	s on the Variable Di e "Enter" pushbutt	isplay. on.			
RO	Depress th	e "Target" pushbut	ttoh and	verify it illuminates.		
RO RO	Input the o load target a Depress th	desired load target appears on the Tar e "Enter" pushbutt	on the n get Displa on.	umeric keypad and veri ay.	fy the	
RO	To start lo	ad decrease, depre	ss the "G	Go" pushbutton and veri	ify it	
RO	SIG Blowd	own changes shoul	d be coo	rdinated with Secondar	M	
NUREG-102 Apper Op-Test No.	Chemistry. 1 ndix D : NRCScenar	3 of32 Operator Actions io No.: 3 Event No.	Re : 3	ev. 8 Form ES-D-2	7	
Event Descrip to Time Po	otion: Tc Fa step in at ma sition	ils to 630 0F. (This aximum rate). Applicant's Action	instrume s or Beha	ent failure will cause the avior	e Control Rods	
RO/BC)P Recogniz Refers to Ar	zes unwarranted ro inunciator Respons	d motion e and Re	and places rods to ma equired Actions per:	nual.	
	1AD-2 Al4	and 1AD-6 N6,B/6	,C16			
	EVALUATOR	NOTE: The Annun	ciator Re	esponse is attached.		
SRO	May deter	mine transition to <i>I</i>	API1/N55	500/i5 is appropriate, Ca	ase II.	
SROIR RO	O Ensure ' Verify all r	CRD BANK SELECT od motion - STOPS	' switch ·	- IN MANUAL.		
RO	Manually a	djust control rods a	as necess	sary to maintain T-Avg	within	
	10F of T-Re	f				
SRO/R SRO	C Determi Ensure comp • 3.1.1 (Shu	ne and correct caus liance with appropri utdown Margin (SD	se of con riate Tecl M)	tinuous rod movement h Specs:		

3.1.4 (Rod Group Alignment Limits)3.1.5 (Shutdown Bank Insertion Limits)

3.1.6 (Control Bank Insertion Limits)
Appendix D Operator Actions Form ES-D-2
Op-Test No.: NRCScenario No.: 3 Event No.: 3
Event Description:Tc Fails to 630 0F. (This instrument failure will cause the Control Rods to step in at maximum rate).TimeFositionApplicant's Actions or Behavior
SRO Determine required notifications:
REFER TO RP/0/AI50001001 (Classification Of Emergency)
REFER TO RP/0/BIEOOOIO1 3 (NRC Notification Requirements)
Determine long term plant status.
NUREG-1021 5 of32 Rev. 8
Appendix D Operator Actions Form ES-D-2
Op-Test No.: NRC Scenario No.: 3 Event No.: 4
Event Description: NC Pump Seal Injection Flow Meter Fails LOW (1NVP5650) Time Iffl~osition Applicant's Actions or Behavior
BOP Recognizes 1 NV-309 failing closed and takes valve to manual based
SRO Directs activities of BOP.
BOP Refers to annunciator response and required actions per
• 1AD-7. E/1, E/2. and F/3.
SRO Calls SPOC and reports failure of NC Pump Seal Injection flow
NUREG-1021 6of32 Rev. 8
Appendix D Operator Actions Form ES-D-2
Op-Test No.: NRC Scenario No.: 3 Event No.: 5
Event Description: NCS Leak (=- 90 GPM) _ Position Applicant's Actions or Behavior
ALL Recognize increase in Containment Activity/Humidity/Pressurel OAC
alarm for unidentified leakage in containment.
RO Verify Pzr level - STABLE OR INCREASING
Determines Pzr pressure is decreasing and informs SRO.
SRO Determines Transition to step 2 RNO required
NUREG-1021 7 of 32 Rev. 8 Appendix D Operator Actions Form ES-D-2
Op-Test No.: NRC Scenario No.: 3 Event No.: 5

Event Description: NCS Leak (E 90 GPM) Time Position Applicant's Actions or Behavior Perform the following:

- a. Maintain charging flow less than 180 GPM
- b. Manually throttle 1 NV-294 (NV Pmps A&B Disc Flow Ctrl) to stabilize Pzr level

c. IF Pzr level is stable OR increasing, THEN GO TO Step 3. SRO determines pressure is still decreasing and contineus in RNO.

- d. If Pzr level continues to decrease. THEN:
- 1) Reduce letdown flow to 45 GPM as follows:
 - a) Manually control 1NV-148 (Letdn Press Control) to maintain letdown pressure at 350 PSIG
 - b) IF 1NV-13A (Letdn Orif IA OtIt Cont Isol) is open, THEN perform the following:
 - (1) Open 1NV-11A (Letdn Orif iC OtIt Cont Isol)
 - (2) Close 1 NV-i 3A (Letdn Orif 1 A OtIt Cont Isol)
 - C) IF 1NV-1OA is open, THEN throttle 1NV-849 until letdown flow is 45 GPM.

 d) WHEN letdown pressure is stable at 350 PSIG, THEN place 1NV-148 (Letdn Press Control) in "AUTO"

SRO IF Pzr level continues to decrease, THEN ensure the following valves closed:

SRO determines this step is N/A and continues.

SRO IF Pzr level is stable OR increasing, THEN GO TO step 3

 ALL
 IF AT ANY TIME the NC System leak increases, THEN perform step 2

 NUREG-1021
 8 of32
 Rev. S

 Appendix D
 Operator Actions
 Form ES-D-2

 II]
 Position
 Applicant's Actions or Behavior

- RO Verity Pzr pressure TRENDING TO OR STABLE AT 2235 PSIG
 - SROIBOP Dispatch operator(s) to locate and isolate NC System leak

EXAMINER NOTE: The crew will not send an operator to look for this since, based on indications, it is in containment.

- BOP IF AT ANY TIME 1AD-7, Ii 'VCT LO LVL'1 alarm is lit, THEN align NV pump suction to the FWST as follows:
 - SRO evaluates step as currently not applicable and continues. Determine NC leak rate by any of the following methods:
 - Compare charging flow and letdown flow

OR

• Monitor OAC NV graphic

OR

RO

Initiate OAC Program "NSNCLEAK"	
OR	
Monitor OAC point EROPLEAK	
OR	
 Monitor OAC point Cl P0976 (Gross N Mm Run Avg) 	C System Leak Rate, Ten
RO Verity auxiliary building radiation is no	rmal as follows:
 EMF-41 (Aux Bldg Ventilation) - TRIP COUNTRATE STABLE 	1 LIGHT DARK AND
All area monitor EMF trip 1 lights - D. NUREG-1021 9 of 32 R Appendix D Operator Actions	ARK ev. 8 Form ES-D-2
Op-Test No.: NRC Scenario No.: 3 Event No.: 5	
Event Description: NCS Leak (=- 90 GPM) Fm Position Applicant's Actions or Beh	avior
ALL NOTE: Letdown flow and pressure are Letdown HX and 1 NV-i 48 (Letdn Press SROIBOP Verify letdown lines - INTACT	e monitored between the s Control)
BOP Verify the following NC pump therma	barrier alarms - DARK:
 iAD-6, Eli, "NCPATHERMAL BARRIER FLOW" 	KC OUTLET HI/LO
 1AD-6, E/2 "NCP B THERMAL BARRIE FLOW" 	ER KC OUTLET HIILO
 1AD-6, E13, "NCP C THERMAL BARR: FLOW 	ier KC outlet HI/Lo
 1AD-6, E14, "NCP D THERMAL BARR FLOW 	IER KC OUTLET HI/LOW
SOP Verify 1 EMF-46A and 1 EMF-46B (Co BOPVerify containment conditions are norma • Containment EMF - Trip 1 LIGHTS D/ STABLE	mponent Cooling) - TRIP 1 Il as follows: ARK AND COUNTRATE
 iEMF38 (Containment Paniculate) 1EME30 (Containment Gas) 	
1 EMF40 (Containment Iodine) Containment floor and equipment su	mn level(s) - STABLE
NUREU-1021 10 of 32 F	lev. 8
if Position Applicant's Actions or Behav	vior
SRO Determines transition to step 12 RNC	appropriate
BOP Stop any VO releases in progress. RE (Containment Air Release and Addition.	FER TO 0P111N64501017
 SRO Transition to Step 13 A/ER column ar BOPVerify NCDT conditions are normal as fol NCDT pressure - STABLE NCDT level - STABLE 	d continues. ows:
 NCD1 discharge flow - EQUAL TO RE 	

- NCDT temperature STABLE
- BOP Verify PRT conditions are normal as follows:
 - PRT pressure LESS THAN 8 PSIG
 - PRT level LESS THAN 89%
 - PRT temperature LESS THAN 1 300F
- SRO Ensure compliance with appropriate Tech Specs and Selected Licensee Commitments Manual:
 - 3.4.13 (RCS Operational Leakage)
 - 3.4.14 (RCS Pressure Isolation Valve (PIV) Leakage)
 - 3.5.5 (Seal Injection Flow)
 - 3.7.17 (Secondary Specific Activity)
 - SLC 16.7-9 (Standby Shutdown System) SRO Determine required notification:
 - - REFER TO RPIOIA/5000I001 (Classification Of Emergency)

	 REFER TO) RP/O/B15000101 3 (NF	RC Notification Requirements)
NURE	G-1021	11 of32	Rev. 8
	Appendix D	Operator Actions	Form ES-D-2
LVI	Position	Applicant's Actions or B	ehavior

ALL Verify NC System leak - ISOLATED

Determines transition to step 17 RNO appropriate: SRO/BOP IF shutdown is required, THEN:

- a. Decrease the unit load at a rate based on the ability to remove the unit from line in a controlled manner
- b. Notify Reactor Group Engineer of occurrence.
- c. IF reactor power is greater than 15%, THEN:
 - 1) Initiate unit shutdown. REFER TO:
 - OPI1IN6100/003 (Controlling Procedure For Unit Operation)

OR

• API1/AI5500/09 (Rapid Downpower)

- 2) Ensure adequate shutdown margin is maintained. REFER TO ROD Book, Section 5.113) GOTO Stepl8.
- SRO Consult with station management for further actions

Determine long term plant status. RETURN TO procedure in effect NUREO-1021 12 of 32 Rev. 8 Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 3 Event No.: 6

 Event Description:
 Rod Ejection with ATWS (NCS leak of 1500 GPM from ejected Rod) NI-SA & NI-IOB fail to automatically open on the SI signal.

 I K[e
 Position
 Applicant's Actions or Behavior ALL

 Recognizes increased RCS Leakage by Pressurizer Pressure and

Level decreasing and Containment Conditions (temp, pressure

- SRO Directs ROIBOP to manually initiate a Reactor Trip and Safety Injection. Evaluate and perform Immediate action's of EPII /A/50001E-O.
- ROIBOP Monitor Enclosure 1 (Foldout Page)

ROVerify Reactor Trip:

- All rod bottom lights LIT (NO)
- All reactor trip and bypass breakers OPEN (NO)
- I/Ramps DECREASING (NO)

Determines reactor is not tripped and informs SRO and performs immediate actions of FR-S.I

- SRO Transitions to step I RNO
- RO Perform the following:
 - · Manually trip reactor
 - IF reactor will not trip, THEN concurrently:
 - Implement EP/I/A/5000/F-0 (Critical Safety Function Status Trees)
 - GO TO EP/1IN5OOOIFR-S.I (Response To Nuclear Power Generation/ATWS)

SRODetermines transition to EP/1/A/5OOOIFR-S.1 (Response To NuclearNUREG-102113 of32Rev. 8Appendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 3Event No.: 6

Event Description: Rod Ejection with AWIS (NCS leak of 1500 GPM from ejected Rod) NI-9A & NI-IOB fail to automatically open on the SI signal.

~eI Position Applicant's Actions or Behavior

Power Generation/AWIS) is required.

- ROVerify Reactor Trip:
 - All rod bottom lights LIT
 - All reactor trip and bypass breakers OPEN
 - hR amps DECREASING
- SRO Determines transition to RNO of step 1 is required
- RO Perform the following:
 - Manually trip the reactor
 - IF reactor will not trip, I!:!LN manually insert rods (CRITICAL STEP)
- SRO Transitions to Step 2 NER column and continues.
- RO Verify Turbine Trip:
 - All turbine stop valves CLOSED

OR

• All turbine control valves - CLOSED

EXAMINER NOTE: The SRO may determine that the iB CA pump is tagged and determine the RNO is Not Applicable NURFU-]021 14of32 Rev. 8 Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 3 Event No.: 6
Event Description:Rod Ejection with ATWS (NCS leak of 1500 GPM from ejected Rod) NI-9A & NI-IOB fail to automatically open on the SI signal.['nPositionApplicant's Actions or Behavior
RO b. S/G levels-GREATER THAN 11% Determines this condition does not exist and informs SRO
SRO Transitions to step 3.b RNO.
 BOP Ensure CA Pump #1 - RUNNING BOPInitiate emergency boration of NC System as follows: a. Verify at least one NV pump - ON b. Open 1 NV-238B (Boric Acid To NV Pumps Suct) c. Ensure both boric acid transfer pump switches - IN THE "ON" POSITION. d. Verify emergency boration flow - GREATER THAN OR EQUAL TO 30 GPM e. Verify the following charging line isolation valves - OPEN: 1NV-321A (Chrg Line Cont Isol)
• 1NV-314B (chrg Line Cont Isol) Determines that this does not exist and informs SRO.
SRO Transition to step 4.e RNO NUREG-1021 15 of 32 Rev. 8 Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 3 Event No.: 6
Event Description:Rod Ejection with ATWS (NCS leak of 1500 GPM from ejected Rod) NI-9A & NI-IOB fail to automatically open on the SI signal.TimePositionApplicant's Actions or Behavior
BOP Perform the following:
1) Align NV pump suction to the FWST as follows:
a) Open the following valves:
• 1NV-252A (NV Pumps Suct From FWST)
• 1NV-253B (NV Pumps Suct From FWST)
b) Close the following valves:
c) 1 NV-i 88A (VCT Otit Isol)
d) 1 NV-i 89B (VCT OtIt Isol)
2) Ensure the following valves - OPEN: : (CRITICAL TASK)
iNI-9A (NV Pmp CIL Inj Isol)

a. Motor driven CA pumps - ON

- iNI-IOB (NV Pmp CIL Inj Isol)
- SRO Transitions back to Step 4.f NER column and directs operators.

RO	Verify Pzr pressure - LESS THAN 2335 PSIG.
5	a. "SAFETY INJECTION ACTUATED" status light (ISI-13 - LIT
	b. WJILN manpower AND time permits, THEN verify proper system alignments. REFER TO Enclosure 1 (System Verification Following S/I Actuation)
	Verification Following 3/1 Actuation/
R	EXAMINER NOTE: Enclosure i affached. D Verify the following trips have occurred: Reactor Trip
NUREG-1 Api	Determines reactor not tripped and informs SRO D21 ~6 of 32 Rev. 8 Dendix D Operator Actions Form FS-D-2
Op-Test N	Io.: NRC Scenario No.: 3 Event No.: 6
Event Desc	ription: Rod Ejection with ATWS (NCS leak of 1500 GPM from ejected Rod) NI-9A & NI-IOB fail to automatically open on the SI signal.
LPI Po	sition Applicant's Actions or Behavior
SRO	Transitions to Step 6.a RNO.
RO	Dispatch Operator to open the following:
	 Reactor trip breakers Reactor trip bypass breakers
	The following breakers for CRD MIG sets: IMate CL Preakers
	"MotoC' Breaker. "Generator" Breaker.
SR	O Transitions to Step 6.b NER column and directs operators.
RO	Verify Turbine Trip
RO	Control SIG levels as follows:
	Verify NIR level in at least one SIG - GREATER THAN 11% (29% ACC)
	Determines this does not exist and informs SRO.
SRC	Transition to step 7 RNO.
RO	Perform the following:
	 IF total CA flow is less than 1000 GPM, THEN manually start pumps and align valves as required
	 WHEN N/R level is greater than 11% (29% ACC) in at least one SIG, THEN perform step 7.b
NUREG-1	D21 17 of 32 Rev. 8 Dendix D Operator Actions Form FS-D-2
Pos	ition Applicant's Actions or Behavior
	• GO TO step 7.c
BO	P WHEN either of the following annunciators are lit, THEN REFER TO
	AP/11Al5500106 (Loss Of SIG Feedwater)

• 1AD-5, H/4 "CACST LO LEVEL"

• 1 BOP E	LAD-8, B/i "UST LO LEVEL" insure all dilution paths are isola	ted as follows:
• F	Place NC makeup control switch	to "STOP"
• F	Place reactor makeup water pum	ips to "OFF"
	orifu main staamlings are intact	as follows:
KU V		as follows.
a.	Verify the following:	
•	All SIG pressures - STABLE O	R INCREASING
SRO G	• All SIGS - PRESSURIZED 60 TO step 12.	
RO Ve	erify all NC T-Colds - STABLE OF	R INCREASING.
RO Ve	erify Core Exit TCs - LESS THAN	12000F.
RO Ve	erify the reactor is subcritical as	follows:
	PIR channels - LESS THAN 5%	Day 9
Appendix	D Operator Actions	Form ES-D-2
Op-Test No.: NR	C Scenario No.: 3	Event No.: 6
Event Description NI-9A Time [Position	: Rod Ejection with AWYS (NC & NI-i OB fail to automatically o Applicant's Actions or IRSUR-NEGATIVE	S leak of 1500 GPM from ejected Rod) pen on the SI signal. Behavior
NUREG-1021	19 of 32	Rev. 8 Form FS-D-2
Аррених		
Op-Test No.: NR	C Scenario No.: 3 Event No.: 6	
Event Description NI-9A ['nPosition BOPEnsu a.E	: Rod Ejection with ATWS (NC & NI-IOB fail to automatically op Applicant's Actions or B ire adequate shutdown margin a insure the following signals - RES	S leak of 1500 GPM from ejected Rod) ben on the SI signal. ehavior s follows: GET:
•	Phase A Containment Isolatio	ns
•	KC NC NI NM St signals	
BOP/SRO (ALLb. WH perform OP/0/N	b. Obtain current NC boron co Chemistry. HEN current NC boron concentra n shutdown margin calculation. F 61 00/006 (Reactivity Balance C	oncentration from Primary tion is obtained, THEN REFER TO alculation)
c. 9	WHEN following conditions are s System boration:	atisfied, THEN stop NC
	Adequate shutdown margin is	s obtained
•	Uncontrolled cooldown has be	een stopped
SRO n SRO R	nplement RPIOIA/50001001 (Cla RETURN TO procedure and step	ssification Of Emergency). in effect.

SRO Determines transition to EP/I/A/5000IE-0, Step 1 is appropriate. Reminds RO/BOP to monitor Enclosure 1 of EP/1IAI5000/E-0.

RO	Verify Reactor Trip: • All rod bottom lights - LIT • All reactor trip and bypass breakers - OPEN • hR amps - DECREASING
NUREO-102 Apper	1 20of32 Rev. 8 ndix D Operator Actions Form ES-D-2
Op-Test No.	: NRC Scenario No.: 3 Event No.: 6
E	
event Descrip NI ~meI Pos	I-9A & NI-IOB fail to automatically open on the SI signal. sition Applicant's Actions or Behavior
RO	Verify Turbine Trip:
	 All turbine stop valves - CLOSED OR
BOP BOP	 All turbine control valves - CLOSED Verify 1 ETA and 1 ETB - ENERGIZED Verify S/I is actuated:
	 "SAFETY INJECTION ACTUATED" status light (1SI-13) - LIT
	• EIS load sequencer actuated status lights (1SI-14) - LIT
RO	Announce "Unit 1 Safety Injection".
SRO	Implement RPIOIA/50001001 (Classification Of Emergency).
RO	Verify all Feedwater Isolation status lights (151-5) - LIT.
BOP	Verify Phase A Containment Isolation status as follows:
	Phase A "RESET' lights - DARK
BOP	 Monitor Light Panel Group 5 St lights - LIT. Verify proper Phase B actuation as follows:
NUREO-102	a. Containment pressure - HAS REMAINED LESS THAN 3 PSIG Determines that containment ressure has exceeded 3 Si and 1 21 of 32 Rev. 8
Apper Op-Test No.	ndix DOperator ActionsForm ES-D-2: NRC Scenario No.: 3Event No.: 6
Event Descrip NI Fm Posi	tion: Rod Ejection with ATWS (NCS leak of 1500 GPM from ejected Rod) I-9A & NI-IOB fail to automatically open on the SI signal. tion Applicant's Actions or Behavior
informs SRC SRO). Transition to step 10.a RNO.
BOP	Perform the following:
	NOTE: This time may be used later to determine when to align ND Aux spray
	Record approximate time of reactor trip
	• Verify NS pumps - INDICATING FLOW.
	• IF flow is not indicated, 'utti manually initiate Phase B Isolation.

Determines step does not apply and continues.

- Verify Phase B Isolation has actuated as follows:
- Phase B Isolation "RESET" lights DARK
- IF Phase B Isolation "RESET' lights are lit, THEN manually initiate Phase B Isolation.
- Verify following monitor light panel lights LIT:
- Group 1 Sp lights
- Group 5 Sp lights
- GroupS St lights LIII and L112

IF monitor light panel is not in correct alignment, THEN ensure correct valve alignment and component operation.

NC pumps while main	taining seal injection flow.
22of32	Rev, 8
Operator Action	s Form ES-D-2
nario No.: 3	Event No.: 6
	NC pumps while main 22of32 Operator Action nario No.: 3

Event Description: Rod Ejection with ATWS (NCS leak of 1500 GPM from ejected Rod) NI-9A & NI-IOB fail to automatically open on the SI signal. Applicant's Actions or Behavior TimePosition SRO GO TO step 11 BOP Verify proper CA pump status as follows: Motor driven CA pumps - ON EXAMINER NOTE: The SRO may determine that the I B CA pump is tagged and determine that the RNO is Not Applicable. RO

- 3 SIG NIR levels GREATER THAN 11%
- Transitions to step 3.b RNO. SRO
- Ensure CA Pump #1 RUNNING BOP
- BOP Verify all of the following S/I pumps - ON:
 - Nvpumps
 - NDpumps
 - NI pumps
- BOP Verify all KC pumps - ON

Verify all Unit 1 and Unit 2 RN pumps - ON BOP Rev. 8

NUREG-1021 23 of 32

Appendix D Form ES-D-2 **Operator Actions**

Op-Test No.: NRC Scenario No.: 3 Event No.: 6

Event Description: Rod Ejection with ATWS (NCS leak of 1500 GPM from ejected Rod) NI-9A & NI-IOB fail to automatically open on the SI signal.

- Π Position Applicant's Actions or Behavior
- BOP . Verify proper ventilation systems operation as follows:
 - REFER TO Enclosure 2 (Ventilation System Verification)
 - Notify Unit 2 operator to perform Enclosure 3 (Opposite Unit Ventilation Verification)
 - Verify all SIG pressures GREATER THAN 775 PSIG. RO
 - BOP/RO Verify proper S/I flow as follows:
 - "NV S/I FLOW" INDICATING FLOW
 - NC pressure LESS THAN 1620 PSIG
 - NI pumps INDICATING FLOW
 - NC pressure LESS THAN 285 PSIG
 - Determines NC pressure is greater than 285 psig and informs SRO.
 - Transition to step 17.d RNO. SRO

BOP Perform the following:

Ensure ND pump miniflow valve on operating ND pump(s) - OPEN SRO GO TO step 18. NUREG-1021 24 of32 Rev. 8 Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 3 Event No.: 6

Event Description: Rod Ejection with AWIS (NCS leak of 1500 GPM from ejected Rod) NI-9A & NI-IOB fail to automatically open on the SI signal.

- I K[e Position Applicant's Actions or Behavior
 - BOP Control SIG levels as follows:
 - Verify total CA flow GREATER THAN 450 GPM

WHEN at least one SIG NIR level is greater than 11% (29% ACC), THEN throttle feed flow to maintain all S/C N/R levels between 11% (29% ACC) and 50%

- Verify all CA isolation valves OPEN
- · Verify S/I equipment status based on monitor light panel IN

PROPER ALIGNMENT

- SRO NOTE: Enclosure 4 (NC Temperature Control) shall remain in effect until subsequent procedures provide alternative NC temperature control guidance.
- RO Control NC temperature. REFER TO Enclosure 4 (NC Temperature Control).

Verify Pzr PORV and Pzr spray valve status as follows:

- BOP All Pzr PORVs CLOSED
 - Normal Pzr spray valves CLOSED
 - At least one Pzr PORV isolation valve OPEN
- RO Verify NC subcooling based on core exit TICs GREATER THAN 00F

Determines that subcooling has been lost and informs SRO.

SRO	Transitions to step 23 RNO.	
NUREO-1021	25 of32	Rev. 8
Append	ix D Operator Actions	Form ES-D-2
Op-Test No.: NRC Scenario No.: 3		Event No.: 6

Event Description: Rod Ejection with AW'S (NCS leak of 1500 GPM from ejected Rod) NI-9A & NI-IOB fail to automatically open on the St signal.

- Time Position Applicant's Actions or Behavior BOP IF any NV OR NI pump is on, THEN:
 - Ensure all NC pumps OFF
 - Maintain seal injection flow
 - RO Verify main steamlines are intact as follows:
 - All SIG pressures STABLE OR INCREASING
 - All SIGs PRESSURIZED
 - BOP Verify SIG tubes are intact as follows:
 - Verify the following EMF trip 1 lights DARK
 - 1 EMF-33 (Condenser Air Ejector Exhaust)
 - 1EMF-34 (SIG Sample)
 - 1EMF-26 (Steamline IA)
 - 1EMF-27 (Steamline IB)
 - 1EMF-28 (Steamline iC)
 - 1EMF-29 (Steamline ID)

BOP	 All SIG levels - STABLE OR INCREASING IN A CONTROLLED MANNER Verify NC System is intact as follows:
	 Containment pressure - LESS THAN 1 PSIG. Determines pressure has exceeded 1 psig and informs SRO.
SRO	Performs step 26 RNO and transitions to EPI1IN5OOOIE-1 (Loss of Reactor Or Secondary Coolant.
NUREG-102 Appe Op-Test No	1 26 of 32 Rev. S ndix D Operator Actions Form ES-D-2 .: NRC Scenario No.: 3 Event No.: 6
Event Descrip N Time Pos	otion: Rod Ejection with AWIS (NCS leak of 1500 GPM from ejected Rod) I-9A & NI-IOB fail to automatically open on the SI signal. ition Applicant's Actions or Behavior
ALL	Monitor Enclosure 1 (Foldout Page).
RO	Verify main steamlines are intact as follows:
	All S/G pressures - STABLE OR INCREASING
BOF	 All SIGs - PRESSURIZED Control intact SIG levels as follows: Verify N/R level in all intact SIGs - GREATER THAN 11% (29% ACC)
	Determines condition does not exist and informs SRO.
SRO	Performs Step 3.a RNO.
BOP	 Maintain Total Feed Flow > 450 GPM until at least 1 intact SIG NIR >11% (29%ACC)
	 Throttle feed flow to maintain all intact SIG NIR levels between 11% (29% ACC) and 50%
ALL	Ensure CA suction source switchover criterion is monitored. REFER TO Enclosure 1 (Foldout Page).
Appe	ndix D Operator Actions Form ES-D-2
Op-Test No.	: NRC Scenario No.: 3 Event No.: 6
Event Descrip	otion: Rod Ejection with ATWS (NCS leak of 1500 GPM from ejected Rod) I-9A & NI-IOB fail to automatically open on the SI signal.
BOP	Verify secondary radiation is normal as follows: Ensure the following signals - RESET: • Phase A Containment Isolations
	KC NC NI NM St signals
	Align all SIGS for Chemistry sampling. Perform at least one of the following: • Notify Chemistry to sample all S/Gs for activity
	OR Notify Chemistry or DD to frick all estion columns for activity
	Verify the following EMF trip I lights - DARK:
	 1 EMF-33 (Condenser Air Ejector Exhaust) 1 EMF-34 (S/G Sample)
	 1EMF-26 (Steamline IA) IEMF-27 (Steamline IB)
	 1EMF-28 (Steamline iC) IEME-29 (Steamline iD)

ALLWHEN activity results are reported, THEN verify all S/Gs indicate activity.NUREG-102128 of32Rev. 8Appendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 3Event No.: 6	no
Event Description: Rod Ejection with AWIS (NCS leak of 1500 GPM from ejected NI-gA & NI-i OB fail to automatically open on the SI signal. II Position Applicant's Actions or Behavior BOP Verify Pzr PORV and isolation valve status as follows: • Power to all Pzr PORV isolation valves - AVAILABLE • All Pzr PORVs - CLOSED • At least one Pzr PORV isolation valve - OPEN	d Rod)
ALL IF AT ANY TIME a Pzr PORV opens due to high pressure, THEN, after Pzr pressure decreases to less than 2315 PSIG, ensure the valve closes or is isolated.	
RO/BOP Verify S/I termination criteria as follows:	
NC subcooling based on core exit TICs - GREATER THAN 00F	
Verify secondary heat sink as follows:NIR level in at least one intact SIG - GREATER THAN 11% (29% ACC)	
OR	
• Total feed flow to all intact SIGs - GREATER THAN 450 GPM	
NC pressure - STABLE OR INCREASING	
Pzr level - GREATER THAN 11% (20% ACC) (NO)	
SROTransitions to step 6.d RNO.NUREG-102129 of32Rev. 8Appendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 3Event No.: 6	
Event Description:Rod Ejection with AWIS (NCS leak of 1500 GPM from ejected NI-9A & NI-IOB fail to automatically open on the SI signal.[¶]PositionApplicant's Actions or Behavior	d Rod)
BOP IE NC pressure is increasing AND normal Pzr spray is available, THEN attempt to stabilize NC pressure using normal Pzr spray	
EXAMINER NOTE: Normal spray is not available due to NC pumps being secured. SRO GO TO step 6.f.	
ALL Monitor S/I termination criteria. REFER TO Enclosure 2 (SII Termination Criteria)	
IF AT ANY TIME S/I termination criteria is met while in this procedure, THEN RETURN TO step 6.	

Verify proper NS pump operation as follows: BOP At least one NS pump - ON

Verify the foI~owing valves - OPEN: 1FW-27A (ND Pump IA Suct From FWST) • 1 FW-55B (ND Pump 1 B Suct From FWST) Containment pressure - LESS THAN 2.4 PSIG Ensure S/I - RESET: • ECCS • DIG load sequencers NUREO-1021 30 of 32 Rev. 8 Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 3 Event No.: 6 Event Description: Rod Ejection with ATWS (NCS leak of 1500 GPM from ejected Rod) NI-9A & NI-IOB fail to automatically open on the SI signal. Time Position Applicant's Actions or Behavior ALL IF AT ANY TIME a BIO occurs, THEN restart S/I equipment previously on. BOP Reset NS Stop NS pumps Close the following valves: 1NS-29A(NS Spray Hdr IACont Iso) 1NS-32A (NS Spray Hdr IA Cont Iso) • INS-15B (NS Spray Hdr IB Cont Iso) INS-12B (NS Spray Hdr IB Cont Isol) BOP Verify criteria to stop operating ND pumps as follows: NC pressure - GREATER THAN 285 PSIG NC pressure - STABLE OR INCREASING At least one ND pump - ON • Ensure S/I - RESET: ECCS D/G load sequencers

ALL IF AT ANY TIME a BIO occurs, THEN restart S/I equipment previously on.

BOP Ensure all ND pump(s) not supporting Cold Leg Recirc - STOPPED. 31 of 32 Rev. 8 NUREG-1021 Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: S Event No.: 6

Event Description: Rod Ejection with ATWS (NCS teak of 1500 GRIll from ejected Rod) NI-9A & NI-i OB fail to automatically open on the SI signal. if Position Applicant's Actions or Behavior

- IF AT ANY TIME NC pressure decreases to less than 285 PSIG in an uncontrolled manner, THEN restart ND pumps.
- ALL

RO Verify LOCA has occurred as follows:

- All SIG pressures -STABLE or increasing
- NC pressure STABLE OR DECREASING
- BOP Verify conditions to stop operating D/Gs as follows:

At least one DIG - ON

Verify 1 ETA is energized by offsite power as follows: • DIG iABKRTO ETA-OPEN

• 1 ETA undervoltage status lights (1SI-14) - DARK

WHEN S/I is reset, £tjLN dispatch operator to stop IA DIG and place in standby readiness. REFER TO OP/i 1N63501002 (Diesel DIG Generator Operation)

Verify 1 ETB is energized by offsite power as follows: • DIGiBBKRTOETr-OPEN

• iETB undervoltage status lights (1SI-14)-DARK WHEN S/I is reset, THEN dispatch operator to stop IB DIG and place in standby readiness. REFER TO OPI1/N6350/002 (Diesel Generator Operation)

Ensure S/I RESET:

• ECCS

 DIG lo 	ad sequences	
NUREO-1021	32 of32	Rev. 8
Appendix D	Operator Actions	Form ES-D-2
Op-Test No.: NRC Scenario No.: 3		Event No.: 6

Event Description:Rod Ejection with ATWS (NCS leak of 1500 GPM from ejected Rod)NI 9A & NI-IOB fail to automatically open on the SI signal.Lpime LfflJ~ositionApplicant's Actions or Behavior

ALL IF AT ANY TIME a BIO occurs, THEN restart S/I equipment previously on.

SOP Obtain containment H2 concentration as follows:

- Ensure operator has been dispatched to secure all ice condenser air handling units. REFER TO Enclosure 3 (Securing All Ice Condenser Air Handling Units).
- Verify containment H2 analyzers- IN SERVICE
- Verify containment H2 concentration- LESS THAN 6%.
- Verify containment H2 concentration-LESS THAN 0.5%.
- When ice condenser air handling units are off AND h2 concentration is less than 6%, THEN energize the H2 igniters (1 MC-7)
- ALL EXAMINER NOTE: At this time, the normal breakers for I ETA will open and IA Diesel Generator will load blackout loads on the bus. Per procedure, the crew must load LOCA loads on the bus. (IA NI will be the only equipment required to be started).

required SI loads are on the bus

TERMINATE SCENARIO WHEN REQUIRED PUMP(S) STARTED. (NI)

Classification: RP/O/A/5000/OO1 (Classification Of Emergency) Enclosure 4.1 - Site Area Emergency (Based on Reactor failing to Trip from a valid Reactor Trip signal received or required and automatic reactor trip was not successful, and Manual reactor trip from the control room was not successful in reducing reactor power to less than 50/o and decreasing

(4A.S.1-1) NUREG-1021 33 of32 Rev. 8 Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 4 Event No.: 1

Event Description: Boration for load decrease[Time ~oswonI Applicant's Actions or Behavior

BOP Refer to OP/11N6150/OO9 (Boron Concentration Control), Enclosure 4.2 (Boration)

Ensure the following valve control switches in "AUTO":

- 1 NV-238A (B/A Xfer Pmp To Blender Ctrl)
- 1 NV-i 86A (B/A Blender OtIt To VCT OtIt)

Adjust the boric acid batch counter to the desired volume of boric acid to be added.

Place the "NC MAKEUP MODE SELECT' switch in "BORATE".

Adjust the controller for 1 NV-238A (B/A Xfer Pmp to Blender Ctrl) to the desired flow.

Ensure 1 NV-238A (B/A Xfer Pmp to Blender Ctrl) controller in "AUTO".

Ensure at least one boric acid transfer pump in "AUTO" or "ON".

NOTE: If necessary, boration can be manually secured at any time by placing the "NC MAKEUP CONTROL" switch to the "STOP" position.

Place the "NC MAKEUP MODE SELECT' switch to the "START' position.

Verify the following valves open:

- 1 NV-238A (B/A Xfer Pmp To Blender Ctrl Vlv)
- 1 NV-I 86A (B/A Blender Ott To VCT OtIt)

IF in "AUTO", verify the boric acid transfer pump starts.

Verify proper flow by observing the boric acid flow totalizer {PIP 96-

0137) NUREG-1021 1 of2 Rev. 8 Appendix D Operator Actions Form ES-D-2 { Op- Test No.: NRC Scenario No.: 4 Event No.: 1

Eve

nt Description: Boration for load decrease

Efflime ffl~osition Applicant's Actions or Behavior BOP WHEN the desired volume of boric acid is reached on the boric acid

batch counter, ensure the following valves close:

- .1 NV-238A (B/A Xfer Pmp To Blender Ctrl Vlv)
- •1NV-186A (B/A Blender Ott To VCT OtIt)

IF desired, flush the makeup line as follows:

- Open the following valves:
 - 1 NV-242A (RMWST To B/A Blender Ctrl)
 - 1NV-186A (B/A Blender OtIt To VCT OtIt)
- WHEN -20 gallons of makeup water have been flushed through

the makeup line, close the following valves:

- 1 NV-242A (RMWST To B/A Blender Ctrl)
- 1 NV-i 86A (B/A Blender OtIt To VCT OtIt)
- Place the following valve control switches in "AUTO":
 - 1 NV-242A (RMWST To B/A Blender CtrI)
 - 1NV-186A (B/A Blender OtIt To VCT OtIt)

IF automatic makeup is desired, refer to Enclosure 4.1 (Automatic Makeup). NUREO-1021 2 of2 Rev. 8 Appendix D Operator Actions Form ES-D-2

Op-Test No.: NRC Scenario No.: 4 Event No.: 2

- Event Description: Decrease turbine load Position Applicant's Actions or Behavior
 - RO Refer to OPI1/B/6300/OO1 (Turbine Generator), Enclosure 4.2 (Load Changing), Step 2.4

Depress the "Load Rate" pushbutton and verify it illuminates.

Input the desired load rate on the numeric keypad and verify the load

rate appears on the Variable Display.

Depress the "Enter" pushbutton.

Depress the "Targer' pushbutton and verify it illuminates.

Input the desired load target on the numeric keypad and verify the load target appears on the Target Display.

Depress the "Enter" pushbutton.

To start load decrease, depress the "Go" pushbutton and verify it illuminates.

S/G blowdown changes should be coordinated with Secondary Chemistry.

NUREG-102~3 of2Rev. 8Appendix DOperator ActionsForm ES-D-2

Op-Test No.: NRC Scenario No.: 4 Event No.: 3

Event Description: Trip of running KC Pump

- mTime Position Applicant's Actions or Behavior
 - BOP Recognizes and reports Trip of operating KC Pump.
 - SRO Implements and directs actions of AP111N5500121 (Loss of Component Coo~ing)
 - CAUTION: Failure to restore NC pump seal cooling via thermal barrier cooling or NV seal injection within 10 minutes will cause damage to the NC pump seals resulting in NC inventory loss.

Monitor Enclosure 1 (Foldout Page).

NOTE: The following step prevents damage to the 1 B2 KC pump as a result of deadheading. (PIP #00-5862) BOP IF AT ANY TIME the following conditions are met:

- SOP IF AT ANT TIME the following conditions are the
 - Train B KC isolation valves CLOSED

AND

• 1 KC-81 B (KC to ND HX IB Sup Isol) - CLOSED.

THEN

a. Ensure less than 2 train B KC Pumps - IN SERVICE.

Verity at least one KC pump - ON

Determines no KC pumps are running and informs SRO.

Transitions to Step 3 RNO and directs the operators.NUREG-~O214 of2Rev. 8Appendix DOperator ActionsForm ES-D-2

Op-Test No.: NRC Scenario No.: 4 Event No.: 3

Event Description: Trip of running KC Pump Ffflme~~~'tion[Applicant's Actions or Behavior

- SRO CAUTION: Operation of more than one train B KC pump should be avoided when the train B KC isolation valves and 1 KC-81 B (KC To ND HX 1 B Sup Isol) are closed.
 BOP Perform the following:

a. Start at least one KC pump

Determines that pump alternate KC pump started and informs SRO.

- SRO SRO determines that Step 3.b RNO does not apply and transitions to Step 4 A/ER column and directs operators.
- SRO CAUTION: A loss of KC cooling to the NC pumps results in a gradual approach to an over heated condition in approximately 10 minutes which will result in shaft seizure.
 - BOP Verify KC flow to NC pumps as follows:
 - 1AD-20 Ni "KC SUPPLY HDR FLOW TO NCP BRGS LOW" -DARK
 - 1AD-21 Ni "KC SUPPLY HDR FLOW TO NCP BRGS LOW" -DARK
 - BOP Verify KC available as follows:

Verify the following Train A KC isolation valves - OPEN:

- 1 KC-230A (Rx Bldg Non-Ess Hdr Isol)
- 1KC-3A (Rx Bldg Non-Ess Ret Hdr Isol)
- 1 KC-SOA (Aux Bldg Non-Ess Hdr Isol)
- 1KC-1A (Aux Bldg Non-Ess Ret Hdr iso)

 NUREG-1021
 5 of2
 Rev. 8

 Appendix D
 Operator Actions
 Form ES-D-2

Op-Test No.: NRC Scenario No.: 4 Event No.: 3

Event Description: Trip of running KC Pump

[Timeflifition! Applicant's Actions or Behavior

BOPVerify the following Train B KC isolation valves - OPEN:

- 1 KC-228B (Rx Bldg Non-Ess Hdr Isol)
- 1KC-18B (Rx Bldg Non-Ess Ret Hdr Isol)
- 1KC-53B (Aux Bldg Non-Ess Hdr 1501)
- IKC-2B (Aux Bldg Non-Ess Ret Hdr Isol)
- SRO CAUTION: Operation of more than one train B KC pump should be avoided when the train B KC isolation valves and 1 KC-81 B (KC To ND Hx IB Sup Isol) are closed

BOP	Start a	dditiona	I KC p	ump(s)	as ne	cessary	to supply	any KC	loads
	presently	in serv	ice.						
						<i>.</i>			

- BOPVerify KC surge tank levels normal as follows: a. Verify both KC surge tank levels - 50% - 90% AND STABLE. b. GOLOStePIO.
- BOP Ensure KC heat exchanger outlet mode switches PROPERLY ALIGNED.
- SRO Determine and correct cause of loss of KC.

10ILC	1021	0.012	
	Appendix D	Operator Actions	Form ES-D-2

Op-Test No NRC Scenario No.: 4 Event No.: 3

Event Description: Trip of running KC Pump

- [11imelITh[oswon Applicant's Actions or Behavior SROEnsure compliance with appropriate Tech Specs and Selected Licensee Commitments Manual:
 - SLC 16.9-7 (Boration Systems Flow Path Shutdown)
 - SLC 16.9-8 (Boration Systems Flow Path Operating)
 - SLC 16.9-9 (Boration Systems Charging Pump Shutdown
 - SLC 16.9-10 (Boration Systems Charging Pumps-Operating)
 - 3.5.2 (ECCS Operating)
 - 3.5.3 (ECCS Shutdown)
 - 3.6.6 (Containment Spray System)

- 3.7.5 (Auxiliary Feedwater (AFW) System)
- 3.7.7 (Component Cooling Water (CCW) System)

(NO)

Determine required notification:

- REFER TO RP/01N50001001 (Classification Of Emergency)
- REFER TO RPIOINSOOO/013 (NRC Notification Requirements)

IF KC Hx leak to RN is suspected, THEN perform the following:

Determines that this step is not applicable and continues to Step 15. Verify KC surge tanks level as follows:

 Greate 	r than 50%	
NUREO-1021	7 of2	Rev. 8
Appendix D	Operator	r Actions Form ES-D-2

Op-Test No NRC Scenario No.: 4 Event No.: 3

Event Description: Trip of running KC Pump Lfflime[ffl~oswon Applicant's Actions or Behavior

- RO Verify IAD-7, F13 "LETDN HX OUTLET HI TEMP" DARK
- BOP IF desired to restore letdown flow through the NV demineralizers, THEN momentarily place 1 NV-i 53A (Ltdn Hx Ott 3-Way VIv) to the "DEM IN" position and return to "AUTO"

SRODetermine long term plant status.
RETURN TO procedure in affectNUREO-10218 of2Rev. 8Appendix DOperator ActionsForm ES-D-2

- Op-Test No.: NRC Scenario No.: 4 Event No.: 4
- Event Description: Pressurizer Pressure Channel I Fails HighTime[PositionApplicant's Actions or Behavior

LZ Determines that Pressurizer Pressure Channel I has failed high, informs SRO, and performs immediate action steps from memory.

I (Pressurizer Pressure Decreasing) and directs operators.

BOP Verify all Pzr level channels - INDICATING THE SAME

Determines that Channel 1 is reading high and informs SRO.

IF either selected channel is failed high, THEN place "PZR PRESS CTRL SELECT" switch in any alternate operable channel position. Verify all Pzr PORVS - CLOSED.

Verify all Pzr heaters - ENERGIZED.

Ensure 1 NV-37A (NV Supply To Pzr Aux Spray) - CLOSED.

Verify NC pressure - STABLE OR INCREASING.

IF a Pzr pressure channel is failed high, THEN notify IAE to fail the following bistables for the affected channel under Model WIO #91002943. Bistables shall be tripped within six (6) hours:

- Pzr low pressure S/I
- OT Delta T

RO

- Pzr high pressure Reactor Trip
- Pzr low pressure Reactor Trip

NUREG-10219of2Rev. 8Appendix DOperator ActionsForm ES-D-2

Op-Test No.: NRC Scenario No.: 4 Event No.: 4

Event Description: Pressurizer Pressure Channel I Fails High [Time Position Applicant's Actions or Behavior

- BOPEnsure compliance with appropriate Tech Specs:
 - 3.3.1 (Reactor Trip System Instrumentation)
 - 3.3.2 (Engineered Safety Features Actuation System)
 - 3.3.3 (Post Accident Monitoring Instrumentation)
 - 3.3.4 (Remote Shutdown System)
 - 3.4.1 (RCS Pressure, Temperature, and Flow Departure From Nucleate Boiling Limits)
 - 3.4.4 (RCS Loops MODES 1 and 2)
 - 3.4.5 (RCS Loops MODE 3)
 - 3.4.6 (RCS Loops MODE 4)
 - 3.4.9 (Pressurizer)
 - 3.4.10 (Pressurizer Safety Valves)
 - 3.4.11 (Pressurizer Power Operated Relief Valves)
- SRO Ensure "PZR PRESS REC SELECT' is selected to an operable channel.

BOPDetermine long term plant status. RETURN TO procedure in effect.NUREG-102110 of2Rev. 8Appendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 4Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at Power Time Position Applicant's Actions or Behavior

RO Determines Main Steam Isolation Valve has closed and informs SRO.

RO Trips Reactor.

С

ROIBOP Perform Immediate actions of E-O from memory

EXAMINER NOTE: Turbine will fail to trip automatically on reactor trip and RO will have to manually trip the turbine. Additionally, automatic S/I will not occur and the BOP will have

to manually initiate both trains of safety injection.

SRO Enter EPI1IAI5OOO/E-O (Reactor Trip or Safety Injection) and direct actions of operators.

EXAMINER NOTE: A large break LOCA occurs coincident with the reactor trip.

ROIBOP Monitor Enclosure 1 (Foldout Page)

ROVerify Reactor Trip:

- All rod bottom lights LIT
- All reactor trip and bypass breakers OPEN
- I/Ramps DECREASING

Verify Turbine Trip:

- All turbine stop valves CLOSED OR
- All turbine control valves CLOSED

EXAMINER NOTE: RO will manually trip the turbine per Step 3.a RNO if not previously tripped.

NUREG-1021	1 of2	Rev. 8
Appendix D	Operator Actions	Form ES-D-2
Op-Test No.: NRC Scenario	No.: 4	Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at PowerTimePositionApplicant's Actions or BehaviorBOPVerify IETA and 1ETB - ENERGIZED

Verify S/I is actuated:

a. "SAFETY INJECTION ACTUATED" status light (1SI-13 - LIT

EXAMINER NOTE: BOP will initiate SII here per Step 5 RNO if not previously completed. b. E/S load sequence actuated status lights (1SI-14) - LIT

RO Announce "Unit 1 Safety Injection"

SRO Implement RPIOIA/5000/001 (Classification Of Emergency)

- RO Verify all Feedwater Isolation status lights (151-5) LIT
- BOPVerify Phase A Containment Isolation status as follows: a. Phase A "RESET' lights - DARK
 - b. Monitor Light Panel Group 5 St lights LIT
- BOP Verify proper Phase B actuation as follows:

a. Containment pressure - HAS REMAINED LESS THAN 3 PSIG Determines that pressure has not remained less than 3 psig and informs SRO.

Transitions to Step 10.a RNO and directs operators.

EXAMINER NOTE: Greater than 3 psig in containment requires use of ACC numbers throughout the remainder of the scenario. ACC numbers are designated in parentheses behind normal values. NUREG-102~ 2 of2 Rev. 8 Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 4 Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at PowerTimePositionApplicant's Actions or Behavior

SRO Perform the following:

NOTE: This time may be used later to determine when to align ND Aux spray

- 1) Record approximate time of reactor trip
- 2) Verify NS pumps INDICATING FLOW.
- 3) IE flow is not indicated, THEN manually initiate Phase B Isolation.

Determines that this step 3) is not required based on plant conditions

- 4) Verify Phase B Isoaltion has actuated as follows:
 - a) Phase B Isolation "RESET' lights DARK.
 - b) if Phase B Isolation "RESET lights are lit, THEN manually initiate Phase B:

Determines that this step b) is not required based on plant conditions

- c) Verify following monitor light panel lights LIT:
 - Group I Sp lights
 - Group 5 Sp lights
 - GroupS Sp lights Lu 1 and L112
- d) JEmonitor light panel is not in correct alignment, THEN ensure correct valve alignment and component operation.
- 5) Stop all NC pumps while maintains seal injection flow.
- 6) GOIQStepll.
- RO Verify proper CA pump status as follows:

a. Motor driven CA pumps - ON

Determines that 1 B CA Pump is not running due to being tagged out.

NUREQ-1021	3 of2	Rev. 8
Appendix D	Operator Actions	Form ES-D-2
Op-Test No.: NRC Scenario	No.: 4	Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at Power TimePosition Applicant's Actions or Behavior SROTransition to step 11.8 RNO and directs operators.

- BOP a. Perform the fol~owing for affected train(s):
 - 1) Reset ECCS.
 - 2) Reset DIG load sequencer.
 - 3) Manually start affected motor drive CA pump.

4) IF AT ANY TIME a BIO occurs, THEN restart S/I equipment previously on.

ROIBOP Transitions to NER column Step 11 .b and directs operators.

- RO b. 3 SIG NIR levels GREATER THAN 11%
- BOP Verify all of the following S/I pumps ON:
 - Nvpumps
 - ND pumps
 - NI pumps

Verify all KC pumps - ON

Determines that 1 Bi KC pump is off and informs SRO.

SRO Transition to step 13 RNO.

Perform the following for affected train(s):

- a. Reset ECCS.
- b. Reset DIG load sequencer.
- c. Manually start affected pump.

______d. IF AT ANY TIME a BIO occurs, THEN restart S/I e ui mentNUREG-102~4of2Rev. 8Appendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 4Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at Power Time Position Applicant's_Actions or Behavior previously on.

Transitions back to Step 14 AIER column and directs operators.

BOP Verify all Unit 1 and Unit 2 RN pumps - ON

Verify proper ventilation systems operation as follows:

- REFER TO Enclosure 2 (Ventilation System Verification)
- Notify Unit 2 operator to perform Enclosure 3 (Opposite Unit Ventilation Verification).
- RO Verify all S/G pressures GREATER THAN 775 PSIG
- RO EXAMINER NOTE: SRO will transition to Step i6 RNO if SIG pressures are less than 775 psig and perform the RNO. This step may or may not occur depending on timing of the scenario.

Perform the following:

- a. Verify the following valves CLOSED:
- AIIMSIVs
- All MSIV bypass valves
- All SIG PORVs

b. IF any valve is still open THEN manually close valve. Transition back to Step 17 AIER column and direct operators.

Verify proper S/I flow as follows:

a. "NV S/I FLOW" - INI	DICATION FLOW
------------------------	---------------

b. NC pressure - LESS THAN 1620 PSIG

C. NI pumps - INDICATING FLOW

d. NC pressure - LESS THAN 285 PSIG NURFG-1021 5 of2 Rev. 8 Appendix D Operator Actions Form ES-D-2 Op-Test No.: NRC Scenario No.: 4 Event No.:7-10

Event Descdption: Main Steam Isolation Valve Closes at Power Time Position Applicant's Actions or Behavior

e. ND pumps - INDICATING FLOW TO C-LEGS

BOPControl S/G levels as follows:

- a. Verify total CA flow GREATER THAN 450 GPM
- b. WHEN at least one S/G NIR level is greater than 11% (20% ACC), THEN throttle feed flow to maintain all SIG NIR levels between 11% (29% ACC) and 50%
- RO Verify all CA isolation valves OPEN
- BOP Verify S/I equipment status based on monitor light panel IN

PROPER ALIGNMENT

- SRO NOTE: Enclosure 4 (NC Temperature Control) shall remain in effect until subsequent procedures provide alternative NC temperature control guidance
- RO Control NC temperature REFER TO Enclosure 4 (NC Temperature Control)

Verify Pzr PORV and Pzr spray valve status as follows:

a. All Pzr PORVs - CLOSED

b. Normal Pzr spray valves - CLOSED

c. At least one Pzr PORV isolation valve- OPEN

Verify NC subcooling based on core exit TICs - GREATER THAN $\rm 00F$

Determines that subcooling is less than 00F and notifies SRO. SRO Transition to step 23 RNO.

NUREO-1021	6of2	Rev. 8
Appendix 0	Operator Actions	Form ES-D-2
Op-Test No.: NRC S	Event No.:7-10	

Event Description: Main Steam Isolation Valve Closes at Power

Time Position Applicant's Actions or Behavior RO LEany NV OR NI pump is on, THEN: a.Ensure all NC pumps - OFF b.Maintain seal injection flow.

> Transition to Step 25 NER column and directs operators. RO Verify main steamlines are intact as follows:

- All SIG pressures STABLE OR INCREASING
- All SIGs PRESSURIZED

Verify S/G tubes are intact as follows:

- Verify the following EMF trip 1 lights DARK:
 - 1 EMF-33 (Condenser Air Ejector Exhaust)
 - 1 EMF-34 (SIG Sample)
 - 1EMF-26 (Steamline IA)
 - 1EMF-27 (Steamline IB)
 - 1EMF-28 (Steamline iC)
 - 1EMF-29 (Steamline iD)
- All SIG levels STABLE OR INCREASING IN A CONTROLLED MANNER.

Verify NC System is intact as follows:

• Containment pressure - LESS THAN 1 PSIG Determines that containment pressure is greater than 1 psig and informs SRO.

Transitions to Step 26 RNO.NUREG-10217 of2Rev. 8Appendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 4Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at Power TimePosition Applicant's Actions or Behavior

- SRO Concurrently:
 - Implement EPI1IA/5000/F-0 (Critical Safety Function Status Trees).
 - GO TO ER/i IA/5000/E-1 (Loss Of Reactor Or Secondary Coolant).

EXAMINER NOTE: When FWST Level reaches 37%, ECCS pumps will swap suction to containment sump. ND pump IA trip will occur at 45% FWST Level.

- SRO Transition to EP/1/A/5000/FR-P.1 (Response to Imminent PTS Condition) due to valid RED PATH and direct actions.
- RO Verify NC pressure GREATER THAN 285 PSIG. Determines that pressure is less than 285 psig and notifies SRO.

Transitions to Step 1 RNO and directs operators.

IF ND flow to C;Legs is greater than 675 GPM, THEN RETURN TO procedure and step in effect.

Transitions back to Step 1 of E-1 (Loss of Reactor or Secondary Coolant).

RO/BOP Monitor Enclosure 1 (Foldout Page)

RO Verify main steamlines are intact as follows:

• All SIG pressures - STABLE OR INCREASING

All SIGs - PRESSURIZED
 NUREG-1021 8 of2 Rev. 8
 Appendix 0 Operator Actions Form ES-D-2
 Op-Test No.: NRC Scenario No.: 4 Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at PowerTimePositionApplicant's Actions or Behavior

Control intact S/G levels as follows:

- Verify N/R level in all intact S/Gs GREATER THAN 11% (29% ACC)
- Throttle feed flow to maintain all intact SIG NIR levels between 11% (29% ACC) and 50%
- Ensure CA suction source switchover criterion is monitored. REFER TO Enclosure 1 (Foldout Page)

Verify secondary radiation is normal as follows:

a. Ensure the following signals - RESET:

- 1) Phase A Containment Isolations
- 2) CA System valve control
- 3) KC NC NI NM St signals
- b. Align all SIGs for Chemistry sampling
- c. Perform at least one of the following:
 - Notify Chemistry to sample al SIGs for activity

OR

• NotifyChemistry or RR to frisk all cation columns for activity

d. Verify the following EMF trip 1 lights - DARK

- 1 EMF-33 (Condenser Air Ejector Exhaust)
- 1 EMF-34 (S/G Sample)
- 1EMF-26 (Steamline IA)
- 1 EMF-27 (Steamline 1 B)
- 1EMF-28 (Steamline iC)
- 1EMF-29 (Steamline iD)
- e. WHEN activity results are reported, THEN verify all SIGs indicate no activity.

NUREO-1 021	9 of 2	Rev. 8
Appendix D	Operator Actions	Form ES-D-2
Op-Test No.: NRC Scenaric	o No.: 4	Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at PowerTimePositionApplicant's Actions or Behavior

EXAMINER NOTE: When FWST level reaches 37%, go to page Xx of this guide. When swap to cold leg recirculation is complete, then return to the step that was transitioned from. ROVerify Pzr PORV and isolation valve status as follows:

- a. Power to all Pzr PORV isolation valves AVAILABLE
- b. All Pzr PORVs CLOSED
- C. At least one Pzr PORV isolation valve OPEN
- d. IF AT ANY TIME a Pzr PORV opens due to high pressure, THEN after Pzr pressure decreases to less than 2315 PSIG, ensure the valve closes or is isolated.

Verify S/I termination criteria as follows:

a. NC subcooling based on core exit TICs - GREATER THAN 00F. Determines that S/I termination criteria is not met and informs SRO.

- RO e. Monitor S/I termination criteria. REFER TO Enclosure 2 (S/I Termination Criteria).
 - f. IF AT ANY TIME S/I termination criteria is met while in this procedure, THEN RETURN TO Step 6.

NUREG-1021	10 of2	Rev. 8
Appendix D	Operator Actions	Form ES-D-2
Op-Test No.: NRC Scenario No.: 4		Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at Power Time Position Applicant's Actions or Behavior

- BOPVerify proper NS pump operation as follows:
 - a. At least one NS pump ON
 - b. Verify the following valves OPEN:
 - 1 FW-27A (ND Pump IA Suct From FWST)
 - 1FW-5SB (ND Pump IB Suct From FWST)
 - c. Containment pressure LESS THAN 2.4 PSIG.
 - d. Ensure S/I RESET:
 - 1) ECCS
 - 2) D/G load sequencers
 - 3) IF AT ANY TIME a BIO occurs, THEN restart S/I equipment previously on.
 - e. ResetNS.
 - f. Stop NS pumps.
 - g. Close the following valves:
 - 1NS-29A (NS Spray Hdr IA Cont Isol)
 - 1NS-32A (NS Spray Hdr IA Cont Isol)
 - 1NS-15B (NS Spray Hdr IB Cont Isol)

1NS-12B (NS Spray Hdr IB Cont Isol)
 BOP Verify criteria to stop operating ND pumps as follows:

a. NC pressure - GREATER THAN 285 PSIG (NO) Determines that pressure is less than 285 psig and informs SRO.

SROTransition to Step 8.a RNO and then GO TO Step 10.NUREG-102111 of2Rev. 8Appendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 4Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at PowerTimePositionApplicant's Actions or Behavior

will occur when FWST level reaches 37%. Also, at this point, ND Pump IA has previously tripped. ROIBOP Monitor Enclosure 1 (Foldout Page).

SRO CAUTION: S/I recirculation flow to NC System must be maintained at all times.

Performs Steps 3 through 8 without delay. CSFs should not be

implemented prior to completion of these steps.

Verify containment sump level - GREATER THAN 3.5 FT.

Verify KC flow to ND heat exchangers - GREATER THAN 5000 GPM. Ensure S/I - RESET:

- a. ECCS
- b. DIG load sequencers
- c. IF AT ANY TIME a B/O occurs, THEN restart S/I equipment previously on.

Align S/I system for recirc as follows:

- a. Verify following valves OPEN:
- 1NI-185A (ND Pump IA Cont Sump Suct)
- 1NI-184B (ND Pump IB Cont Sump Suct)

b. Verify following valves- CLOSED:

NUREG-102112 of2Rev. 8Appendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 4Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at Power Time Position Applicant's Actions or Behavior

- 1FW-27A (ND Pump IA Suct From FWST)
- IFW-55B (ND Pump IB Suct From FWST)

C. Verify ND pumps - ON.

Determines that IAND pump is not on and informs SRO.

Transitions to Step 6.c RNO and directs operators.

Perform the following:

- 1) Start ND pump(s) with suction aligned to an open containment sump suction valve.
- 2) 'Eno ND pump can be started OR no ND train can be aligned for recirc, THEN:

Determines than this step does not apply and transitions to Step 6.d NER column and directs operators.

- d. Isolate NI pump miniflow as follows:
 - 1) Verify NC pressure LESS THAN 1620 PSIG.
 - 2) Close the following valves:
 - INI-115A (NI Pump IA Miniflow Isol)
 - INI-144A (NI Pump IB Miniflow Isol)

- 3) Place "PWR DISCON FOR INI-147B" switch in "ENABLE"
- 4) Close iNI-147B (NI Miniflow Hdr To FWST Isol)
- e. Close the following valves:
 - 1ND-32A (ND Train IA Hot Leg Inj Isol)
 - 1ND-65B (ND Train IB Hot Leg Inj 1501)
- f. Verify at least one of the following NV pumps minmow valves CLOSED:

1 NV-203A (NV Pumps A&B Recirc Isol)
NUREG-1021 13 of2 Rev. 8
 Appendix D Operator Actions Form ES-D-2
Op-Test No.: NRC Scenario No.: 4 Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at Power Time Position Applicant's Actions or Behavior

OR

• 1NV-202B (NV Pumps A&B Recirc Isol)

9. Align ND train discharges to NI and NV pump suctions as follows:

- 1) Open the following valves:
 - 1NI-332A (NI Pump Suct X-Over From ND)
 - 1NI-333B (NI Rump Suct From ND)
- 2) Ensure 1 NI-334B (NI Pump Suct X-Over From ND) OPEN.
- 3) Open the following valves:
 - 1 ND-28A (ND Supply To NV & IA NI Pmps)
 - 1NI-136B (ND Supply To NI Pump IB)
- h. Isolate FWST from NV and NI pumps as follows:
 - 1) Place "PWR DISCON FOR iNI-IOOB" switch in "ENABLE"
 - 2) Close iNI-IOOB (NI Pmps Suct From FWST)
 - 3) Close the following valves:
 - 1NV-252A (NV Pumps Suct From FWST)

• 1NV-253B (NV Pumps Suct From FWST) i. Verify proper Recirc Flow as follows:

- "NV S/I FLOW" INDICATING FLOW
- NI pumps INDICATING FLOW
- ND pumps INDICATING FLOW

NUREG-102114 of2Rev. SAppendix DOperator ActionsForm ES-D-2Op-Test No.: NRC Scenario No.: 4Event No.:7-10

Event Description: Main Steam Isolation Valve Closes at Power Time Position Applicant's Actions or Behavior WHEN FWST level decreases to 11% (1AD-9, E/8 UFWST LO-LO BOPLEVEL" alarm lit), THEN perform the following:

a. Stop NS Pumps

b. Align NS for recirc. REFER TO Enclosure 2 (Alignment MS for Recirculation).

Verify proper recirc flow as follows:

- "NV S/I FLOW" INDICATING FLOW
- NI pumps INDICATING FLOW
- ND pumps INDICATING FLOW
- SRO EP/1~A/5000/F-0 (Critical Safety Function Status Trees) may now be implemented)

TERMINATE SCENARIO WHEN RECIRC FLOW IS VERIFIED NUREG-1021 15 of2 Rev. 8