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Docket No.: 50-339.

Mr. J. H. Ferguson
 Executive Vice President - Power
 Virginia Electric and Power Company
 Post Office Box 26666
 Richmond, Virginia 23261

Dear Mr. Ferguson:

SUBJECT: ISSUANCE OF AMENDMENT NO. 1 TO FACILITY OPERATING LICENSE
 NPF-7 NORTH ANNA POWER STATION, UNIT NO. 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 1 to Facility Operating License NPF-7.

This Amendment changes Technical Specification 4.4.6.2.2. regarding leak testing of certain pressure isolation valves prior to entering Mode 2 rather than Mode 4. The change to this Technical Specification is described in the enclosed Safety Evaluation supporting Amendment No. 1 to NPF-7.

A copy of a related notice which has been forwarded to the Office of the Federal Register is enclosed for your information and use.

Sincerely,

13

B. J. Youngblood, Chief
 Licensing Branch No. 1
 Division of Licensing

Enclosures:

1. Amendment No. 1 to NPF-7
2. Federal Register Notice
3. Safety Evaluation

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See previous yellow for concurrences

no formal objection

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SURNAME	MRushbrook	ADromerick	D. Swanson/ S. Treby	BJYoungblood
DATE	11/14/80	11/12/80	11/14/80	11/17/80

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Sincerely,

B. J. Youngblood, Chief
 Licensing Branch No. 1
 Division of Licensing

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3. Safety Evaluation

no legal objection

OFFICE	DL:LB#1	DL:LB#1	OELD	DL:LB#1		
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DATE	11/12/80	11/12/80	11/19/80	11/ /80		

November 17, 1980

Mr. J. H. Ferguson
Executive Vice President - Power
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Board of Supervisors of Louisa
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Trevillians, Virginia 23170

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Washington, D.C. 20460

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U. S. Environmental Protection Agency
Region III Office
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Philadelphia, Pennsylvania 19106

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-7
Amendment No. 1

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Virginia Electric and Power Company (licensee), dated November 4, 1980, for the North Anna Power Station, Unit No. 2 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the license, as amended, the provisions of the Act and the regulations of the Commission,
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changing the following Technical Specification contained in Facility Operating License NPF-7 to read:

4.4.6.2.2 b. Prior to entering Mode 2 whenever the plant has been in COLD SHUT DOWN for 72 hours or more and if leakage testing has not been performed in the previous 9 months.

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2.C.(2) Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 1, are hereby incorporated into the license.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Attachment:
Page 3/4 4-18 to Technical
Specification Appendix A

Date of Issuance: NOV 17 1980

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SURNAME ▶	MRushbrooks	AD... <i>13</i>	D Swanson	BJ Youngblood		
DATE ▶	11/12/80	11/13/80	11/19/80	11/17/80		

ATTACHMENT TO LICENSE AMENDMENT NO. 1

FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Change the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by the Amendment Number and contains vertical lines indicating the area of change.

Page

3/4 4-18

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REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS

4.4.6.2.1 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by:

- a. Monitoring the containment atmosphere particulate radioactivity monitor at least once per 12 hours.
- b. Monitoring the containment sump inventory and discharge at least once per 12 hours.
- c. Measurement of the CONTROLLED LEAKAGE to the reactor coolant pump seals when the Reactor Coolant System pressure is $2235 + 20$ psig at least once per 31 days with the modulating valve fully open. The provisions of Specification 4.0.4 are not applicable for entry into MODE 4.
- d. Performance of a Reactor Coolant System water inventory balance at least once per 72 hours.
- e. Monitoring the reactor head flange leakoff temperature at least once per 24 hours.

4.4.6.2.2 Each Reactor Coolant System Pressure Isolation Valve specified in Table 3.4-1 shall be demonstrated OPERABLE pursuant to Specification 4.0.5, except that in lieu of any leakage testing required by Specification 4.0.5, each valve shall be demonstrated OPERABLE by verifying leakage to be within its limit:

- a. At least once per 18 months.
- b. Prior to entering MODE 2 whenever the plant has been in COLD SHUTDOWN for 72 hours or more and if leakage testing has not been performed in the previous 9 months.
- c. Prior to returning the valve to service following maintenance, repair or replacement work on the valve.
- d. Within 24 hours following valve actuation due to automatic or manual action or flow through the valve.

Amendment No. 1

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-339

VIRGINIA ELECTRIC AND POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 1 to the Facility Operating License No. NPF-7, issued to Virginia Electric and Power Company for the North Anna Power Station, Unit No. 2, which changes a Technical Specification contained in Facility Operating License NPF-7. The amendment is effective as of its date of issuance.

The amendment changes one Technical Specification regarding leak testing of certain pressure isolation valves prior to entering Mode 2 rather than Mode 4.

The application for the license complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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DATE ▶						

For further details with respect to this action, see (1) Virginia Electric and Power Company letter dated November 4, 1980, (2) Amendment No. 1 to Facility Operating License No. NPF-7, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. 20555 and at the Board of Supervisor's Office, Louisa County Courthouse, Louisa, Virginia 23093 and at the Alderman Library, Manuscripts Department, University of Virginia, Charlottesville, Virginia 22901. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 17th day of November 1980 .

FOR THE NUCLEAR REGULATORY COMMISSION

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B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

*subject to modification on p. 1
no legal objection*

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SURNAME ▶	MRushbrook / ds	ADromerick	D Swanson	BJYoungblood		
DATE ▶	11/12/80	11/13/80	11/13/80	11/17/80		

NOV 17 1980

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

IN SUPPORT OF CHANGE TO TECHNICAL SPECIFICATION 4.4.6.2.2

FOR NORTH ANNA POWER STATION, UNIT 2

INTRODUCTION

In a letter, dated November 4, 1980, the Virginia Electric and Power Company (VEPCO) requested a change to the North Anna Power Station Unit 2 Technical Specification 4.4.6.2.2. The Technical Specification now requires all pressure isolation valves to be leak tested prior to entering hot shutdown (Mode 4) on a periodic testing interval or each time the valve is disturbed. VEPCO has requested that the Technical Specification be changed to permit leak testing prior to entering startup (Mode 2) for all pressure isolation valves identified in Table 3.4-1 of the Technical Specifications.

EVALUATION

The basis for requesting this change is as follows. Leak testing the pressure isolation valves prior to entering Mode 2 will (1) allow high pressure testing at actual functional pressure, (2) allow testing in lower operational modes, and (3) gives more accurate data because correction factors applied to measured leak rates will not be used.

We have concluded that leak testing at a higher pressure with a larger differential pressure across the valve will produce a more accurate

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calculation of leak rate and more closely simulate actual operating conditions since extrapolation methods are not required. Also, high pressure testing will ensure proper seating of check valves used as pressure isolation valves allowing a more meaningful leak test at operating conditions. We also conclude that reasonable assurance will be provided that the design pressure of low pressure systems which interface with the reactor coolant system will not be exceeded. Thus, the probability of an intersystem LOCA will not be increased.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered or a significant decrease in any safety margin, it does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by

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operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations.

Thus, the issuance of this amendment to change the North Anna Power Station, Unit 2 Technical Specification 4.4.6.2.2 to permit leak testing prior to entering startup (Mode 2) for all pressure isolation valves identified in Table 3.4-1 of the Technical Specifications, will not be inimical to the common defense and security or to the health and safety of the public. Also, we reaffirm our conclusions as otherwise stated in our Safety Evaluation Report and its Supplements.

Dated NOV 17 1980