

April 14, 1998

Mr. J. P. O'Hanlon
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

SUBJECT: NORTH ANNA POWER STATION, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS
REGARDING A PROPOSED TECHNICAL SPECIFICATION CHANGE MODIFYING
TESTING REQUIREMENTS FOR THE REACTOR TRIP BYPASS BREAKERS
(TAC NOS. MA0872 AND MA0873)

Dear Mr. O'Hanlon:

The Commission has issued the enclosed Amendment Nos. 209 and 190 to Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station, Units No. 1 and No. 2 (NAPS-1&2). The amendments consist of changes to the Technical Specifications (TS) in response to your letter dated February 3, 1998.

The amendments in the form of changes to the TS revise the Surveillance Requirement Tables 3.3-1 and 4.3-1 for both units modifying the testing requirements for the reactor trip bypass breaker.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

N. Kalyanam, Project Manager
Project Directorate II-1

Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-338 and 50-339

Enclosures:

1. Amendment No. 209 to NPF-4
2. Amendment No. 190 to NPF-7
3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION

See attached sheet

NRC FILE CENTER COPY

DFOI

FILENAME - G:\NOANNA\MA0872-3.AMN

OFFICE	PM:PDII-1	LA:PDII-1	D:PDII-1	OGC <i>EB</i>	DRCH:HICB
NAME	NKalyanam <i>Kaly</i>	Dunnington <i>ED</i>	PKR <i>PKR</i>	R Bachmann	JW <i>JW</i>
DATE	3 / 13 / 98	3 / 13 / 98	3 / 30 / 98	3 / 15 / 98	3 / 17 / 98
COPY	(Yes) / No	(Yes) / No	Yes / No	Yes / No	(Yes) / No

9804160413 980414
PDR ADOCK 05000338
P PDR

DATED: April 14, 1998

AMENDMENT NO. 209 TO FACILITY OPERATING LICENSE NO. NPF-4-NORTH ANNA UNIT 1
AMENDMENT NO. 190 TO FACILITY OPERATING LICENSE NO. NPF-7-NORTH ANNA UNIT 2

~~Docket File~~

PUBLIC

PDII-1 RF

J. Zwolinski, 14/E/4

G. Hill (4), TWFN 5/C/3

W. Beckner

ACRS

L. Plisco, RII

Mr. J. P. O'Hanlon
Virginia Electric & Power Company

North Anna Power Station
Units 1 and 2

cc:

Mr. J. Jeffrey Lunsford
County Administrator
Louisa County
P.O. Box 160
Louisa, Virginia 23093

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth St., SW, Suite 23T85
Atlanta, Georgia 30303

Michael W. Maupin, Esquire
Hunton and Williams
Riverfront Plaza, East Tower
951 E. Byrd Street
Richmond, Virginia 23219

Mr. W. R. Matthews, Manager
North Anna Power Station
P. O. Box 402
Mineral, Virginia 23117

Dr. W. T. Lough
Virginia State Corporation
Commission
Division of Energy Regulation
P. O. Box 1197
Richmond, Virginia 23209

Mr. R. C. Haag
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth St., SW, Suite 23T85
Atlanta, Georgia 30303

Old Dominion Electric Cooperative
4201 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. J. P. O'Hanlon
Senior Vice President - Nuclear
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. J. H. McCarthy, Manager
Nuclear Licensing & Operations
Support
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. David Christian, Manager
Surry Power Station
Virginia Electric and Power Company
5570 Hog Island Road
Surry, Virginia 23883

Office of the Attorney General
Commonwealth of Virginia
900 East Main Street
Richmond, Virginia 23219

Robert B. Strobe, M.D., M.P.H.
State Health Commissioner
Office of the Commissioner
Virginia Department of Health
P.O. Box 2448
Richmond, Virginia 23218

Senior Resident Inspector
North Anna Power Station
U.S. Nuclear Regulatory Commission
1024 Haley Drive
Mineral, Virginia 23117



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 209
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated February 3, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

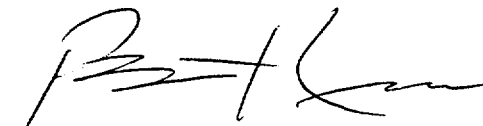
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 209, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



P. T. Kuo, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 209

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. Overleaf page 3/4 3-3 is included for document completeness.

Remove Pages

3/4 3-4
3/4 3-14

Insert Pages

3/4 3-4
3/4 3-14

TABLE 3.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
8. Overpower ΔT					
Three Loop Operation	3	2	2	1, 2	7#
Two Loop Operation	3	1**	2	1, 2	9
9. Pressurizer Pressure-Low	3	2	2	1, 2	7#
10. Pressurizer Pressure--High	3	2	2	1, 2	7#
11. Pressurizer Water Level--High	3	2	2	1, 2	7#
12. Loss of Flow - Single Loop (Above P-8)	3/loop	2/loop in any oper- ating loop	2/loop in each oper- ating loop	1	7#
13. Loss of Flow - Two Loops (Above P-7 and below P-8)	3/loop	2/loop in two oper- ating loops	2/loop each oper- ating loop	1	7#
14. Steam Generator Water Level--Low-Low	3/loop	2/loop in any oper- ating loops	2/loop in each oper- ating loop	1, 2	7#
15. Steam/Feedwater Flow Mismatch and Low Steam Generator Water Level	2/loop-level and 2/loop-flow mismatch	1/loop-level coincident with 1/loop-flow mismatch in same loop	1/loop level and 2/loop-flow mismatch or 2/loop-level and 1/loop-flow mismatch	1, 2	7#

TABLE 3.3-1 (Continued)
REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
16. Undervoltage – Reactor Coolant Pump Busses	3 – 1/bus	2	2	1	7#
17. Underfrequency – Reactor Coolant Pump Busses	3 – 1/bus	2	2	1	7#
18. Turbine Trip					
A. Low Auto Stop Oil Pressure	3	2	2	1	7#
B. Turbine Stop Valve Closure	4	4	4	1	7#
19. Safety Injection Input from ESF	2	1	2	1, 2	1
20. Reactor Coolant Pump Breaker Position Trip					
A. Above P-8	1/breaker	1	1/breaker	1	10
B. Above P-7	1/breaker	2	1/breaker per operating loop	1	11
21. A. Reactor Trip Breakers	2	1	2	1, 2	1, 14
	2	1	2	3*, 4*, 5*	15
B. Reactor Trip Bypass Breakers	2	1	1	***	13
22. Automatic Trip Logic	2	1	2	1, 2	1
	2	1	2	3*, 4*, 5*	15

TABLE 4.3-1 (Continued)

NOTATION

- * – With the reactor trip system breakers closed and the control rod drive system capable of rod withdrawal.
- *** – Below P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.
- (1) – If not performed in previous 7 days.
- (2) – Heat balance only, above 15% of RATED THERMAL POWER.
- (3) – Compare incore to excore axial offset above 15% of RATED THERMAL POWER. Adjust channel if absolute difference ≥ 3 percent.
- (4) – Manual ESF functional input check every 18 months.
- (5) – Each train or logic channel shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (6) – Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (7) – The CHANNEL FUNCTIONAL TEST shall independently verify the OPERABILITY of the undervoltage and shunt trip circuits for the manual reactor trip function. The test shall also verify the operability of the Bypass Breaker Trip circuit(s).
- (8) – Local manual shunt trip the reactor trip bypass breaker immediately after placing the bypass breaker into service, but prior to commencing reactor trip system testing or reactor trip breaker maintenance.
- (9) – Automatic undervoltage trip.
- (10) – The CHANNEL FUNCTIONAL TEST shall independently verify the OPERABILITY of the undervoltage and shunt trip attachments of the Reactor Trip Breakers.
- (11) – Monthly Surveillance in Modes 3*, 4* and 5* shall also include verification that Permissives P-6 and P-10 are in their required state for existing plant conditions by observation of the permissive annunciator window.
- (12) – Detector plateau curves shall be obtained and evaluated. The provisions of Specification 4.0.4 are not applicable for entry into Mode 2 or 1.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 190
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated February 3, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

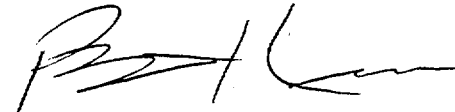
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 190 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



P. T. Kuo, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 190

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. Overleaf page 3/4 3-3 is included for document completeness.

Remove Pages

3/4 3-4

3/4 3-14

Insert Pages

3/4 3-4

3/4 3-14

TABLE 3.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
8. Overpower ΔT Three Loop Operation	3	2	2	1, 2	7#
Two Loop Operation	3	1**	2	1, 2	9
9. Pressurizer Pressure-Low	3	2	2	1, 2	7#
10. Pressurizer Pressure--High	3	2	2	1, 2	7#
11. Pressurizer Water Level--High	3	2	2	1, 2	7#
12. Loss of Flow - Single Loop (Above P-8)	3/loop	2/loop in any oper- ating loop	2/loop in each oper- ating loop	1	7#
13. Loss of Flow - Two Loops (Above P-7 and below P-8)	3/loop	2/loop in two oper- ating loops	2/loop each oper- ating loop	1	7#
14. Steam Generator Water Level--Low-Low	3/loop	2/loop in any oper- ating loops	2/loop in each oper- ating loop	1, 2	7#
15. Steam/Feedwater Flow Mismatch and Low Steam Generator Water Level	2/loop-level and 2/loop-flow mismatch	1/loop-level coincident with 1/loop-flow mismatch in same loop	1/loop level and 2/loop-flow mismatch or 2/loop-level and 1/loop-flow mismatch	1, 2	7#

TABLE 3.3-1 (CONTINUED)
REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
16. Undervoltage-Reactor Coolant Pump Busses	3-1/bus	2	2	1	7#
17. Underfrequency-Reactor Coolant Pump Busses	3-1/bus	2	2	1	7#
18. Turbine Trip					
A. Low Auto Stop Oil Pressure	3	2	2	1	7#
B. Turbine Stop Valve Closure	4	4	4	1	7#
19. Safety Injection Input from ESF	2	1	2	1, 2	1
20. Reactor Coolant Pump Breaker Position Trip					
A. Above P-8	1/breaker	1	1/breaker	1	10
B. Above P-7	1/breaker	2	1/breaker per operating loop	1	11
21. A. Reactor Trip Breakers	2	1	2	1, 2	1, 14
	2	1	2	3*, 4*, 5*	15
B. Reactor Trip Bypass Breakers	2	1	1	***	13
22. Automatic Trip logic	2	1	2	1, 2	1
	2	1	2	3*, 4*, 5*	15

TABLE 4.3-1(CONTINUED)

NOTATION

- * – With the reactor trip system breakers closed and the control rod drive system capable of rod withdrawal.
- ** – Surveillance requirements for the manual ESF functional test of the safety injection input to the reactor trip breakers is suspended for the duration of Cycle 9 operation.
- *** – Below P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.
- (1) – If not performed in previous 7 days.
- (2) – Heat balance only, above 15% of RATED THERMAL POWER. Adjust channel if absolute difference >2 percent.
- (3) – Compare incore to excore axial offset above 15% of RATED THERMAL POWER. Recalibrate if absolute difference \geq 3 percent.
- (4) – Manual ESF functional input check every 18 months.
- (5) – Each train or logic channel shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (6) – Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (7) – Below the P-6, (Block of Source Range Reactor Trip) Setpoint.
- (8) – The CHANNEL FUNCTIONAL TEST shall independently verify the OPERABILITY of the undervoltage and shunt trip circuits for the Manual Reactor Trip Function. The test shall also verify the OPERABILITY of the Bypass Breaker trip circuit(s).
- (9) – Local manual shunt trip the reactor trip bypass breaker immediately after placing the bypass breaker into service, but prior to commencing reactor trip system testing or reactor trip breaker maintenance.
- (10) – Automatic undervoltage trip.
- (11) – The CHANNEL FUNCTIONAL TEST shall independently verify the OPERABILITY of the undervoltage and shunt trip attachments of the Reactor Trip Breakers.
- (12) – Monthly Surveillance in Modes 3*, 4* and 5* shall also include verification that permissives P-6 and P-10 are in their required state for existing plant conditions by observation of the permissive annunciator window.
- (13) – Detector plateau curves shall be obtained and evaluated. The provisions of Specification 4.0.4 are not applicable for entry into Mode 2 or 1.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 209 AND 190 TO
FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7
VIRGINIA ELECTRIC AND POWER COMPANY
OLD DOMINION ELECTRIC COOPERATIVE
NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2
DOCKET NOS. 50-338 AND 50-339

1.0 INTRODUCTION

By a letter dated February 3, 1998, Virginia Electric and Power Company (VEPCO), the licensee for North Anna Power Station, Units 1 and 2, requested NRC's approval to implement amendments to its Operating Licenses NPF-4 and NPF-7, by incorporating modifications to the Technical Specifications (TS). The proposed TS modifications will modify the TS test sequence for the reactor trip bypass breakers.

2.0 BACKGROUND

Two reactor-trip (trip) breakers and two reactor-trip-bypass (bypass) breakers at North Anna, Units 1 and 2, are arranged in a series/parallel combination such that for each trip breaker there is a bypass breaker connected in parallel to it. Normally, bypass breakers are in a racked-in position with the contacts open and are placed in the service position (i.e. racked-in and contacts closed) only during testing of the trip breakers so that power will be provided to the control rods. Bypass and trip breakers are interlocked such that only one bypass breaker can be closed at a given time.

Current TS require a staggered Channel Functional Test for each trip and bypass breaker every 62 days. Also, Note 8 in the North Anna Unit 1 TS (Note 9 for Unit 2 TS) to Table 4.3-1 stipulates that testing of the local-manual-shunt-trip of each bypass breaker must be done prior to placing the bypass breaker in service. Testing of the local-manual-shunt-trip, per

9804160424 980414
PDR ADOCK 05000338
P PDR

current plant procedures, is conducted, first by placing the bypass breaker in its service position, and then tripping it by using the local shunt trip device. Thus, this test sequence always resulted in a duration when the breaker has been placed in the service position (i.e. racked-in and contacts closed) without prior testing for its local-manual-shunt-trip operability. This situation placed the plant in a "non-compliance" status during the trip breaker test, as far as the requirements of Note 8 (Note 9 for Unit 2) of TS Table 4.3-1 are concerned. On October 10, 1996, the licensee identified this non-compliance condition in their Deviation Report N 96-2304, and corrected the deficiency by testing the manual shunt trip immediately after closing the normally racked-in bypass breaker. To address this issue, the licensee issued LER N1-96-009-00 dated November 6, 1996, and the NRC issued non-cited Violation 50-338, 339/96012-02.

To resolve the compliance issue, the licensee as an interim measure, revised procedures to include a modified test sequence. The modified test sequence requires the bypass breaker to be racked from its normal racked-in position to the test position. The breaker is then closed and tripped locally with the local-manual-shunt-trip device and then returned to the racked-in position, whereby the bypass breaker is placed in service for testing and/or maintenance of the trip breaker to commence. This modified test sequence requires racking breakers to the test position, which increases wear and tear on the breakers. Therefore, the licensee has proposed a revision to the TS test sequence for the bypass breakers.

3.0 PROPOSED CHANGES AND EVALUATION

In their submittal, the licensee stated that the proposed changes to the TS notations will return the test sequence to the initial testing sequence which was followed until October 10, 1996, and the proposed notations will also clarify that the bypass breaker functional test is coordinated with the trip breaker functional test to ensure that the bypass breaker is tested prior to its associated trip breaker.

- 3.1 Proposed change: Table 4.3-1 Note 8 for Unit 1, and Table 4.3-1 Note 9 for Unit 2, revise notation to read from "Local manual shunt trip prior to placing the bypass breaker in to service", to read "Local manual shunt trip the reactor trip bypass breaker immediately after placing the bypass breaker into service, but prior to commencing reactor trip system testing or reactor trip breaker maintenance."

Evaluation: The proposed change clarifies the intent of this Note and resolves the "non-compliance" state as was identified on October 10, 1996. Testing without racking the breaker to its test position eliminates unnecessary movement thus reducing wear and tear of components and alignment problems caused over time due to several rack-in and rack-out operations. The licensee stated that although the proposed test sequence will render the bypass breakers in a service status for a short time before its local-shunt-trip device is tested, the operability of the breaker is established based on a satisfactory test conducted during the previous surveillance interval. The word "immediately" in the Note will reduce this time to a minimum as practicable to reestablish the bypass breaker's operability prior to beginning the test and/or maintenance activities on the reactor trip system. The proposed revision to this Note is acceptable to the staff, as it meets the intent of the TS-required testing for the reactor trip and bypass breakers.

3.2 Proposed change: Table 3.3-1, Functional Unit 21B, Reactor Trip Bypass Breakers, revise Minimum Channels Operable from two to one.

Evaluation: The inoperability of any bypass breaker could mean the breaker will either fail to close or fail to open. If one out of two bypass breakers is inoperable due to its failure to close, it will prevent test and/or maintenance activities on its associated trip breaker. If the bypass breaker is inoperable due to failure to open (welded contacts), it will provide a continuous parallel path for its associated trip breaker and thereby diminish the ability of the trip breakers to trip the reactor. The effect of this situation on plant safety is enveloped by the failure-analysis of the associated reactor trip breaker (single failure), and in this situation, a reactor trip could be achieved by tripping the redundant trip breaker. The bypass breakers are interlocked such that only one bypass breaker can be closed at a given time. Also, the plant procedures preclude closing of both bypass breakers at the same time for test and/or maintenance. The licensee in their submittal has stated that changing the minimum channels operable requirement for the bypass breakers does not effect operation of the reactor trip system since only one bypass breaker can be placed in service for trip breaker test and maintenance at any time. Therefore, the proposed change is acceptable to the staff.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comment.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change a surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (63 FR 11925). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

Based on the our review of the licensee's submittal, the staff finds that the proposed TS changes for the test sequence and the revised notation for the North Anna Units 1 and 2 reactor trip and bypass breaker testing will preclude the plant from being in non-compliance for future TS-required surveillances and will provide clarification for coordinating the trip breakers and bypass breakers functional tests as originally intended in the TS. Therefore, the staff finds the proposed TS changes acceptable.

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Reviewer: S. V. Athavale

Date: April 14, 1998