June 23, 1998

Mr. J. P. O'Hanlon
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

SUBJECT: NORTH ANNA POWER STATION, UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS REGARDING A PROPOSED TECHNICAL SPECIFICATION CHANGE ON STATION MANAGEMENT TITLE CHANGES (TAC NOS. MA1350 AND MA1351)

Dear Mr. O'Hanlon:

The Commission has issued the enclosed Amendment Nos. 212 and 193 to Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station, Units No. 1 and No. 2. The amendments consist of changes to the Technical Specifications (TS) in response to your letter dated March 25, 1998.

The amendments in the form of changes to the TS revise the Sections 6.1.1; 6.2.1.b; 6.2.3.5; 6.5.1.1; 6.5.1.2; 6.5.1.6.a,d,h and m; 6.5.1.7.a and c; 6.5.1.8, 6.14.1.2 and 6.15.b for Unit 1 and Sections 6.1.1; 6.2.1.b; 6.2.3.5; 6.5.1.1; 6.5.1.2; 6.5.1.6.a,d,h and m; 6.5.1.7.a and c; 6.5.1.8: 6.13.b and 6.14.b for Unit 2, modifying the station management titles.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,
Original signed by:
N. Kalyanam, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-338 and 50-339

Enclosures:
1. Amendment No. 212 to NPF-4
2. Amendment No. 193 to NPF-7
3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION
See attached sheet

NRC FILE CENTER COPY

FILENAME - G:\NOANNA\MA1350-1.AMN *previously concurred

OFFICE	PM:PDII-1	LA:PDII-1	D:PDII-1	OGC*	
NAME	NKalyanam	Dunnington	PTKuo		
DATE	5/28/98	6/9/98	6/9/98	/ /98	/ /98
COPY	Yes/No	Yes/No	Yes/No	Yes/No	Yes/No

9806260118 980623
PDR ADOCK 05000338
P PDR

DATED: June 23, 1998

AMENDMENT NO. 212 TO FACILITY OPERATING LICENSE NO. NPF-4-NORTH ANNA UNIT 1
AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE NO. NPF-7-NORTH ANNA UNIT 2

Docket File
PUBLIC
PDII-1 RF
J. Zwolinski, 14/E/4
G. Hill (4), TWFN 5/C/3
W. Beckner
ACRS
L. Plisco, RII
G. Edison

Mr. J. P. O'Hanlon
Virginia Electric & Power Company

North Anna Power Station
Units 1 and 2

cc:
Mr. J. Jeffrey Lunsford
County Administrator
Louisa County
P.O. Box 160
Louisa, Virginia 23093

Michael W. Maupin, Esquire
Hunton and Williams
Riverfront Plaza, East Tower
951 E. Byrd Street
Richmond, Virginia 23219

Dr. W. T. Lough
Virginia State Corporation
Commission
Division of Energy Regulation
P. O. Box 1197
Richmond, Virginia 23209

Old Dominion Electric Cooperative
4201 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. J. H. McCarthy, Manager
Nuclear Licensing & Operations
Support
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Office of the Attorney General
Commonwealth of Virginia
900 East Main Street
Richmond, Virginia 23219

Senior Resident Inspector
North Anna Power Station
U.S. Nuclear Regulatory Commission
1024 Haley Drive
Mineral, Virginia 23117

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth St., SW, Suite 23T85
Atlanta, Georgia 30303

Mr. W. R. Matthews, Manager
North Anna Power Station
P. O. Box 402
Mineral, Virginia 23117

Mr. R. C. Haag
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth St., SW, Suite 23T85
Atlanta, Georgia 30303

Mr. David Christian, Manager
Surry Power Station
Virginia Electric and Power Company
5570 Hog Island Road
Surry, Virginia 23883

Robert B. Strobe, M.D., M.P.H.
State Health Commissioner
Office of the Commissioner
Virginia Department of Health
P.O. Box 2448
Richmond, Virginia 23218

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 212
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated March 25, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9806260123 980623
PDR ADOCK 05000338
P PDR

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 212 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

P. T. Kuo, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 23, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 212

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. Overleaf page 6-1a is included for document completeness.

Remove Pages	Insert Pages
6-1	6-1
6-1b	6-1b
6-5	6-5
6-5a	6-5a
6-6	6-6
6-7	6-7
6-25	6-25
6-26	6-26

6.1 RESPONSIBILITY

6.1.1 The Site Vice President shall be responsible for overall facility operation. In his absence, the Manager - Station Operations and Maintenance shall be responsible for overall facility operation. During the absence of both, the Site Vice President shall delegate in writing the succession to this responsibility.

6.1.2 The Shift Supervisor (or during his absence from the Control Room, a designated individual) shall be responsible for the Control Room command function and shall be the only individual that may direct the licensed activities of licensed operators. A management directive to this effect, signed by the Senior Vice President - Nuclear, shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

ONSITE AND OFFSITE ORGANIZATION

6.2.1 Onsite and Offsite Organization

An onsite and an offsite organization shall be established for facility operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the UFSAR.

b. The Site Vice President shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.

c. The Vice President - Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

d. The management position responsible for training of the operating staff and the management position responsible for the quality assurance functions shall have sufficient organizational freedom including sufficient independence from cost and schedule when opposed to safety considerations.

e. The management position responsible for health physics shall have direct access to that onsite individual having responsibility for overall facility management. Health physics personnel shall have the authority to cease any work activity when worker safety is jeopardized or in the event of unnecessary personnel radiation exposures.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown in the UFSAR.

a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.

b. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODES 1, 2, 3 or 4, at least one licensed Senior Reactor Operator shall be in the Control Room.

c. A health physics technician# shall be onsite when fuel is in the reactor.

d. ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

The health physics technician composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

6.2.3 STATION NUCLEAR SAFETY (SNS)

FUNCTION

6.2.3.1 SNS shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources which may indicate areas for improving plant safety.

COMPOSITION

6.2.3.2 SNS shall be composed of at least five dedicated, full-time engineers located onsite.

RESPONSIBILITIES

6.2.3.3 SNS shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

6.2.3.4 SNS shall disseminate relevant operational experience.

AUTHORITY

6.2.3.5 SNS shall make detailed recommendations for revised procedures, equipment modifications, or other means of improving plant safety to the Manager - Station Safety and Licensing.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall serve in an advisory capacity to Shift Supervisor on matters pertaining to the engineering aspects of assuring safe operation of the unit.

* Not responsible for sign-off function

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANS 3.1 (12/79 Draft)* for comparable positions, except for:

1. The Superintendent - Radiological Protection shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.
2. Incumbents in the positions of Shift Supervisor, Assistant Shift Supervisor (SRO), Control Room Operator - Nuclear (RO), and Shift Technical Advisor, shall meet or exceed the requirements of 10 CFR 55.59(c) and 55.31(a)(4).
3. The Superintendent Operations shall hold (or have previously held) a Senior Reactor Operator License for North Anna Power Station or a similar design Pressurized Water Reactor plant.
4. The Supervisor Shift Operations shall hold an active Senior Reactor Operator License for North Anna Power Station.

6.4 TRAINING

6.4.1 The Manager - Nuclear Training is responsible for ensuring that retraining and replacement training programs for the licensed facility staff meet or exceed the requirements of 10 CFR 55.59(c) and 55.31(a)(4). Also, a retraining and replacement training program for non-licensed facility staff shall meet or exceed the recommendations of Section 5 of ANS 3.1 (12/79 Draft)*.

6.5 REVIEW AND AUDIT

6.5.1 STATION NUCLEAR SAFETY AND OPERATING COMMITTEE (SNSOC)

FUNCTION

6.5.1.1 The SNSOC shall function to advise the Site Vice President on all matters related to nuclear safety.

---

* Exceptions to this requirement are specified in VEPCO's QA Topical Report, VEP-1, "Quality Assurance Program, Operational Phase."

COMPOSITION

6.5.1.2 The SNSOC shall be composed of:

Chairman:	Manager - Station Safety and Licensing
Vice Chairman and Member:	Manager - Station Operations and Maintenance
Member:	Superintendent - Operations
Member:	Superintendent - Maintenance
Member:	Superintendent - Radiological Protection
Member:	Superintendent - Engineering

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SNSOC Chairman to serve on a temporary basis; however, no more than one alternate shall participate as a voting member in SNSOC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

a. Review of 1) all new procedures required by Specifications 6.8.1 and 6.8.2, 2) all procedure changes that require a safety evaluation, 3) all programs required by Specification 6.8.4 and changes thereto, and 4) any other procedures or changes thereto as determined by the Site Vice President to affect nuclear safety.

b. Review of all proposed tests and experiments that affect nuclear safety.

c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Site Vice President.

e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and the MSRC.

f. Review of all REPORTABLE EVENTS and Special Reports.

g. Review of facility operations to detect potential nuclear safety hazards.

h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Site Vice President.

i. Deleted.

j. Deleted.

k. Review of every unplanned onsite release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President-Nuclear Operations and the Management Safety Review Committee.

l. Review changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.

m. Review of the Fire Protection Program and implementing procedures and shall submit recommended changes to the Site Vice President.

AUTHORITY

6.5.1.7 The SNSOC shall:

a. Provide written approval or disapproval of items considered under 6.5.1.6(a) through (c) above. SNSOC approval shall be certified in writing by either the Manager - Station Operations and Maintenance or the Manager - Station Safety and Licensing.

b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.

c. Provide written notification within 24 hours to the Vice President- Nuclear Operations and the Management Safety Review Committee (MSRC) of disagreement between the SNSOC and the Site Vice President; however, the Site Vice President shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Site Vice President, Vice President-Nuclear Operations and the MSRC.

6.5.2 MANAGEMENT SAFETY REVIEW COMMITTEE (MSRC)

FUNCTION

6.5.2.1 The MSRC shall function to provide independent review of designated activities in the areas of:

a. Station Operations

b. Maintenance

c. Reactivity Management

d. Engineering

e. Chemistry and Radiochemistry

f. Radiological Safety

g. Quality Assurance Practices

h. Emergency Preparedness

6.13 DELETED

6.14 PROCESS CONTROL PROGRAM (PCP)

6.14.1 Changes to the PCP:

1. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.r. This documentation shall contain:

   a) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

   b) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.

2. Shall become effective after review and acceptance by the SNSOC and the approval of the Site Vice President.

6.15 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Changes to the ODCM:

a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.r. This documentation shall contain:

   1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

   2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.

b. Shall become effective after review and acceptance by the SNSOC and the approval of the Site Vice President.

c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

6.16 DELETED

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 193
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated March 25, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 193 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

P. T. Kuo, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 23, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 193

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. Overleaf page 6-5 is included for document completeness.

Remove Pages	Insert Pages
6-1	6-1
6-1b	6-1b
6-6	6-6
6-6a	6-6a
6-7	6-7
6-8	6-8
6-25	6-25

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Site Vice President shall be responsible for overall facility operation. In his absence, the Manager - Station Operations and Maintenance shall be responsible for overall facility operation. During the absence of both, the Site Vice President shall delegate in writing the succession to this responsibility.

6.1.2 The Shift Supervisor (or during his absence from the Control Room, a designated individual) shall be responsible for the Control Room command function and shall be the only individual that may direct the licensed activities of licensed operators. A management directive to this effect, signed by the Senior Vice President - Nuclear, shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

ONSITE AND OFFSITE ORGANIZATION

6.2.1 Onsite and Offsite Organization

An onsite and an offsite organization shall be established for facility operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the UFSAR.

b. The Site Vice President shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.

c. The Vice President - Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

d. The management position responsible for training of the operating staff and the management position responsible for the quality assurance functions shall have sufficient organizational freedom including sufficient independence from cost and schedule when opposed to safety considerations.

6.2.3 STATION NUCLEAR SAFETY (SNS)

FUNCTION

6.2.3.1 SNS shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources which may indicate areas for improving plant safety.

COMPOSITION

6.2.3.2 SNS shall be composed of at least five dedicated, full-time engineers located onsite.

RESPONSIBILITIES

6.2.3.3 SNS shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

6.2.3.4 SNS shall disseminate relevant operational experience.

AUTHORITY

6.2.3.5 SNS shall make detailed recommendations for revised procedures, equipment modifications, or other means of improving plant safety to the Manager - Station Safety and Licensing.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall serve in an advisory capacity to the Shift Supervisor on matters pertaining to the engineering aspects of assuring safe operation of the unit.

---

* Not responsible for sign-off function

TABLE 6.2-1 (Continued)

SS - Shift Supervisor with a Senior Reactor Operators License on Unit 2.
SRO - Individual with a Senior Reactor Operators License on Unit 2.
RO - Individual with a Reactor Operators License on Unit 2.
AO - Auxiliary Operator
STA - Shift Technical Advisor

Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 1, 2, 3 or 4, an individual (other than the Shift Technical Advisor) with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 5 or 6, an individual with a valid RO license (other than the Shift Technical Advisor) shall be designated to assume the Control Room command function.

Procedures will be established to insure that NRC policy statement guidelines regarding working hours established for employees are followed. In addition, procedures will provide for documentation of authorized deviations from these guidelines and that the documentation is available for NRC review.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANS 3.1 (12/79 Draft)* for comparable positions, except for:

1. The Superintendent - Radiological Protection shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.
2. Incumbents in the positions of Shift Supervisor, Assistant Shift Supervisor (SRO), Control Room Operator - Nuclear (RO), and Shift Technical Advisor, shall meet or exceed the requirements of 10 CFR 55.59(c) and 55.31(a)(4).
3. The Superintendent Operations shall hold (or have previously held) a Senior Reactor Operator License for North Anna Power Station or a similar design Pressurized Water Reactor plant.
4. The Supervisor Shift Operations shall hold an active Senior Reactor Operator License for North Anna Power Station.

6.4 TRAINING

6.4.1 The Manager - Nuclear Training is responsible for ensuring that retraining and replacement training programs for the licensed facility staff meet or exceed the requirements of 10 CFR 55.59(c) and 55.31(a)(4). Also, a retraining and replacement training program for non-licensed facility staff shall meet or exceed the recommendations of Section 5 of ANS 3.1 (12/79 Draft)*.

6.5 REVIEW AND AUDIT

6.5.1 STATION NUCLEAR SAFETY AND OPERATING COMMITTEE (SNSOC)

FUNCTION

6.5.1.1 The SNSOC shall function to advise the Site Vice President on all matters related to nuclear safety.

---

* Exceptions to this requirement are specified in VEPCO's QA Topical Report, VEP-1, "Quality Assurance Program, Operational Phase."

COMPOSITION

6.5.1.2 The SNSOC shall be composed of:

Chairman: Manager - Station Safety and Licensing

Vice Chairman and Member: Manager - Station Operations and Maintenance

Member: Superintendent - Operations

Member: Superintendent - Maintenance

Member: Superintendent - Radiological Protection

Member: Superintendent - Engineering

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SNSOC Chairman to serve on a temporary basis; however, no more than one alternate shall participate as a voting member in SNSOC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

a. Review of 1) all new procedures required by Specifications 6.8.1 and 6.8.2, 2) all procedure changes that require a safety evaluation, 3) all programs required by Specification 6.8.4 and changes thereto, and 4) any other procedures or changes thereto as determined by the Site Vice President to affect nuclear safety.

b. Review of all proposed tests and experiments that affect nuclear safety.

c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Site Vice President.

e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and the MSRC.

f. Review of all REPORTABLE EVENTS and Special Reports.

g. Review of facility operations to detect potential nuclear safety hazards.

h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Site Vice President.

i. Deleted.

j. Deleted.

k. Review of every unplanned onsite release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President-Nuclear Operations and the Management Safety Review Committee.

l. Review changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.

m. Review of the Fire Protection Program and implementing procedures and shall submit recommended changes to the Site Vice President.

AUTHORITY

6.5.1.7 The SNSOC shall:

a. Provide written approval or disapproval of items considered under 6.5.1.6(a) through (c) above. SNSOC approval shall be certified in writing by either the Manager - Station Operations and Maintenance or the Manager - Station Safety and Licensing.

b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.

c. Provide written notification within 24 hours to the Vice President- Nuclear Operations and the Management Safety Review Committee (MSRC) of disagreement between the SNSOC and the Site Vice President; however, the Site Vice President shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Site Vice President, Vice President-Nuclear Operations and the MSRC.

6.5.2 MANAGEMENT SAFETY REVIEW COMMITTEE (MSRC)

FUNCTION

6.5.2.1 The MSRC shall function to provide independent review of designated activities in the areas of:

a. Station Operations

b. Maintenance

c. Reactivity Management

d. Engineering

e. Chemistry and Radiochemistry

f. Radiological Safety

g. Quality Assurance Practices

h. Emergency Preparedness

6.13 PROCESS CONTROL PROGRAM (PCP)

Changes to the PCP:

a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.r. This documentation shall contain:

   1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

   2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.

b. Shall become effective after review and acceptance by the SNSOC and the approval of the Site Vice President.

6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)

Changes to the ODCM:

a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.r. This documentation shall contain:

   1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

   2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.

b. Shall become effective after review and acceptance by the SNSOC and the approval of the Site Vice President.

c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 212 AND 193 TO

FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2

DOCKET NOS. 50-338 AND 50-339

1.0 INTRODUCTION

By letter dated March 25, 1998, Virginia Electric and Power Company (VEPCO), the licensee for North Anna Power Station, Units 1 and 2, requested NRC's approval to implement amendments to its Operating Licenses NPF-4 and NPF-7, by incorporating modifications to the Technical Specifications (TS). The proposed TS changes revise station management titles. Specifically, the Station Manager's title will be changed to Site Vice President; the title of Assistant Station Manager Operations and Maintenence will be changed to Manager - Station Operations and Maintenence (O&M) and that of Assistant Station Manager Nuclear Safety and Licensing will be changed to Manager - Station Safety and Licensing (S&L).

2.0 EVALUATION

The proposed changes will revise station management titles to reflect the new positions approved by the Virginia Power Board of Directors on February 20, 1998. As a result of these changes, the Station Manager, who is responsible for overall unit safe operation and has control over those onsite activities necessary for safe operation and maintenance of the plant now carries the title of Site Vice President. The title of the Assistant Station Manager (Operations and Maintenance), who is responsible for overall facility operation in the absence of the Station Manager now changes to Manager - Station Operations and Maintenance. Similarly, the title of Assistant Station Manager (Nuclear Safety and Licensing), to whom

9806260124 980623
PDR ADOCK 05000338
P PDR

detailed recommendations for revised procedures, equipment modifications, or other means of improving plant safety are made, now changes to Manager - Station Safety and Licensing.

The requested changes do not alter the intent, meaning or the substance of the TS and reflects only the new organizational titles. The proposed changes, as described above, are administrative in nature and the responsibility and authority of the individuals remain the same as before and are not altered by their new titles.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comment.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping or administrative procedures or requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22.(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

6.0 CONCLUSION

Based on the review of the licensee's submittal, the staff finds that the proposed TS changes on the change of the titles of the station management for the North Anna Units 1 and 2 do not alter the intent, meaning or substance of the TS. Therefore, the staff finds the proposed TS changes acceptable.

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such

activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: N. Kalyanam

Date: June 23, 1998