

May 2, 1995

DISTRIBUTION  
See attached sheet

Mr. J. P. O'Hanlon  
Senior Vice President - Nuclear  
Virginia Electric and Power Company  
5000 Dominion Blvd.  
Glen Allen, Virginia 23060

SUBJECT: NORTH ANNA UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE:  
RESPONSIBILITIES OF STATION NUCLEAR SAFETY AND OPERATING  
COMMITTEE AND MANAGEMENT SAFETY REVIEW COMMITTEE  
(TAC NOS. M88467 AND M88468)

Dear Mr. O'Hanlon:

The Commission has issued the enclosed Amendment Nos. 191 and 172 to Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The amendments revise the Technical Specifications (TS) in response to your letter dated December 27, 1993, as supplemented by letters dated September 6, 1994, and March 7, 1995.

The amendments revise the NA-1&2 TS regarding the review responsibilities of the Station Nuclear Safety and Operating Committee and the Management Safety Review Committee.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed By)  
Leon B. Engle, Project Manager  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-338  
and 50-339

Enclosures:

1. Amendment No. 191 to NPF-4
2. Amendment No. 172 to NPF-7
3. Safety Evaluation

cc w/enclosures:  
See next page

FILENAME - A:\NA88467.AMD

\*Previously Concurred

OFFICE	LA:PDII-1	PM:PDII-1	TQMB*	D:RD	OGC*
NAME	Dunnington <sup>ETD</sup>	Engle <sup>BF</sup>	Black	Matthews	CPW
DATE	4/28/95	4/18/95	04/19/95	5/2/95	04/20/95
COPY	Yes/No	Yes/No	Yes/No	Yes/No	Yes/No

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Mr. J. P. O'Hanlon  
Virginia Electric & Power Company

North Anna Power Station  
Units 1 and 2

cc:

Mr. William C. Porter, Jr.  
County Administrator  
Louisa County  
P.O. Box 160  
Louisa, Virginia 23093

Michael W. Maupin, Esq.  
Hunton and Williams  
Riverfront Plaza, East Tower  
951 E. Byrd Street  
Richmond, Virginia 23219

Dr. W. T. Lough  
Virginia State Corporation  
Commission  
Division of Energy Regulation  
P.O. Box 1197  
Richmond, Virginia 23209

Old Dominion Electric Cooperative  
4201 Dominion Blvd.  
Glen Allen, Virginia 23060

Mr. M. L. Bowling, Manager  
Nuclear Licensing & Programs  
Virginia Electric and Power Company  
Innsbrook Technical Center  
5000 Dominion Blvd.  
Glen Allen, Virginia 23060

Office of the Attorney General  
Supreme Court Building  
101 North 8th Street  
Richmond, Virginia 23219

Senior Resident Inspector  
North Anna Power Station  
U.S. Nuclear Regulatory Commission  
Route 2, Box 78  
Mineral, Virginia 23117

Robert B. Strobe, M.D., M.P.H.  
State Health Commissioner  
Office of the Commissioner  
Virginia Department of Health  
P.O. Box 2448  
Richmond, Virginia 23218

Regional Administrator, RII  
U.S. Nuclear Regulatory Commission  
101 Marietta Street, N.W.,  
Suite 2900  
Atlanta, Georgia 30323

Mr. J. A. Stall, Manager  
North Anna Power Station  
P.O. Box 402  
Mineral, Virginia 23117



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 191  
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated December 27, 1993, as supplemented by letters dated September 6, 1994, and March 7, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 191, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 2, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 191

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages

6-6  
6-8  
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6-13

Insert Pages

6-6  
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6-13

## ADMINISTRATIVE CONTROLS

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### MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

### QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

### RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

- a. Review of 1) all new procedures required by Specifications 6.8.1 and 6.8.2, 2) all procedure changes that require a safety evaluation, 3) all programs required by Specification 6.8.4 and changes thereto, and 4) any other procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Station Manager.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and the MSRC.
- f. Review of all REPORTABLE EVENTS and Special Reports.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Station Manager.
- i. Deleted.
- j. Deleted.

- k. Review of every unplanned onsite release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President-Nuclear Operations and the Management Safety Review Committee.
- l. Review changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.
- m. Review of the Fire Protection Program and implementing procedures and shall submit recommended changes to the Station Manager.

AUTHORITY

6.5.1.7 The SNSOC shall:

- a. Provide written approval or disapproval of items considered under 6.5.1.6(a) through (c) above. SNSOC approval shall be certified in writing by an Assistant Station Manager.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Nuclear Operations and the Management Safety Review Committee (MSRC) of disagreement between the SNSOC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Station Manager, Vice President-Nuclear Operations and the MSRC.

6.5.2 MANAGEMENT SAFETY REVIEW COMMITTEE (MSRC)

FUNCTION

6.5.2.1 The MSRC shall function to provide independent review of designated activities in the areas of:

- a. Station Operations
- b. Maintenance
- c. Reactivity Management
- d. Engineering
- e. Chemistry and Radiochemistry
- f. Radiological Safety
- g. Quality Assurance Practices
- h. Emergency Preparedness

## ADMINISTRATIVE CONTROLS

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### COMPOSITION

6.5.2.2 The MSRC shall be composed of the MSRC Chairman and a minimum of four MSRC members. The Chairman and all members of the MSRC shall have qualifications that meet the requirements of Section 4.7 of ANSI/ANS 3.1-1979 Rev. 1 (Draft).

### ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the MSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in MSRC activities at any one time.

### CONSULTANTS

6.5.2.4 Consultants should be utilized as determined by the MSRC Chairman to provide expert advice to the MSRC.

### MEETING FREQUENCY

6.5.2.5 The MSRC shall meet at least once per calendar quarter.

### QUORUM

6.5.2.6 The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

### REVIEW

6.5.2.7 The MSRC shall be responsible for the review of:

- a. Safety evaluations as programmatically discussed in the Updated Final Safety Analysis Report for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to assess the effectiveness of the safety evaluation program and to verify that the reviewed actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.



## ADMINISTRATIVE CONTROLS

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- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. A representative sample of reports and meetings minutes of the SNSOC.

### AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training and qualifications of the entire facility staff.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50.
- e. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.
- f. The Fire Protection Program and implementing procedures.
- g. An independent fire protection and loss prevention inspection and audit shall be performed utilizing an outside qualified fire consultant.
- h. The Radiological Environmental Monitoring Program and the results thereof.

## ADMINISTRATIVE CONTROLS

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- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each new procedure of 6.8.1 above, except 6.8.1.d, 6.8.1.e, and 6.8.1.f shall be reviewed and approved by the SNSOC prior to implementation as set forth in administrative procedures. Procedures of 6.8.1.d, 6.8.1.e, and 6.8.1.f shall be reviewed and approved as set forth in the facility's Security Plan, Emergency Plan, and section 6.5.1.6.m of the Technical Specifications, respectively.

6.8.3 Procedure changes that require a safety evaluation shall also be reviewed and approved by SNSOC. All other changes shall be independently reviewed and approved as programmatically discussed in the Updated Final Safety Analysis Report.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the recirculation spray, safety injection, chemical and volume control, gas stripper, and hydrogen recombiners. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 172  
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated December 27, 1993, as supplemented by letters dated September 6, 1994, and March 7, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 172 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 2, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 172

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages

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MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

RESPONSIBILITIES :

6.5.1.6 The SNSOC shall be responsible for:

- a. Review of 1) all new procedures required by Specifications 6.8.1 and 6.8.2, 2) all procedure changes that require a safety evaluation, 3) all programs required by Specification 6.8.4 and changes thereto, and 4) any other procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Station Manager.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and the MSRC.
- f. Review of all REPORTABLE EVENTS and Special Reports.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Station Manager.
- i. Deleted.
- j. Deleted.

## ADMINISTRATIVE CONTROLS

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### COMPOSITION

6.5.2.2 The MSRC shall be composed of the MSRC Chairman and a minimum of four MSRC members. The Chairman and all members of the MSRC shall have qualifications that meet the requirements of Section 4.7 of ANSI/ANS 3.1-1979 Rev. 1 (Draft).

### ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the MSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in MSRC activities at any one time.

### CONSULTANTS

6.5.2.4 Consultants should be utilized as determined by the MSRC Chairman to provide expert advice to the MSRC.

### MEETING FREQUENCY

6.5.2.5 The MSRC shall meet at least once per calendar quarter.

### QUORUM

6.5.2.6 The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

### REVIEW

6.5.2.7 The MSRC shall be responsible for the review of:

- a. Safety evaluations as programmatically discussed in the Updated Final Safety Analysis Report for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to assess the effectiveness of the safety evaluation program and to verify that the reviewed actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.

## ADMINISTRATIVE CONTROLS

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- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. A representative sample of reports and meetings minutes of the SNSOC.

### AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training and qualifications of the entire facility staff.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50.
- e. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.
- f. The Fire Protection Program and implementing procedures.
- g. An independent fire protection and loss prevention inspection and audit shall be performed utilizing an outside qualified fire consultant.
- h. The Radiological Environmental Monitoring Program and the results thereof.



- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each new procedure of 6.8.1 above, except 6.8.1.d, 6.8.1.e, and 6.8.1.f shall be reviewed and approved by the SNSOC prior to implementation as set forth in administrative procedures. Procedures of 6.8.1.d, 6.8.1.e, and 6.8.1.f shall be reviewed and approved as set forth in the facility's Security Plan, Emergency Plan, and section 6.5.1.6.m of the Technical Specifications, respectively.

6.8.3 Procedure changes that require a safety evaluation shall also be reviewed and approved by SNSOC. All other changes shall be independently reviewed and approved as programmatically discussed in the Updated Final Safety Analysis Report.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the recirculation spray, safety injection, chemical and volume control, gas stripper, and hydrogen recombiners. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NOS. 191 AND 172 TO  
FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7  
VIRGINIA ELECTRIC AND POWER COMPANY  
OLD DOMINION ELECTRIC COOPERATIVE  
NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2  
DOCKET NOS. 50-338 AND 50-339

1.0 INTRODUCTION

By letter dated December 27, 1993, as supplemented by letters dated September 6, 1994, and March 7, 1995, the Virginia Electric and Power Company (the licensee) proposed changes to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). Specifically, the proposed changes revise the review responsibilities of the Station Nuclear Safety and Operating Committee (SNSOC) and the Management Safety Review Committee (MSRC). The responsibilities of the SNSOC include the review of new procedures and changes to procedures that affect nuclear safety. The MSRC review responsibilities include the review of safety evaluations (SEs) and SNSOC meeting minutes and reports. The licensee has proposed that the extent of these review activities be revised in the TS to ensure the two review groups are focusing on nuclear safety issues and not spending an unnecessary amount of time on administrative activities of minimal safety significance.

The letters dated September 6, 1994, and March 7, 1995, provided additional information and did not affect the staff's initial proposed determination of no significant hazards consideration as noticed in the Federal Register on February 16, 1994 (59 FR 7700).

An evaluation of the proposed changes is provided below.

2.0 EVALUATION

The licensee proposes to revise the current TS requirements (Sections 6.5.1.6.a, 6.8.1 and 6.8.2) that the SNSOC review all changes to normal, abnormal, and emergency operating procedures, and all maintenance procedures and changes thereto. The proposed change would require the SNSOC to review only new normal, abnormal, and emergency operating procedures and all new maintenance procedures, and any procedure changes that require a safety evaluation.

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The licensee states that procedures undergo a screening process to determine whether a safety evaluation is required which determines whether a procedure goes to the SNSOC. The staff has reviewed the screening provisions described in VEPCO Station Administrative Procedure VPAP-3001 and finds them acceptable. VEPCO also stated in their March 7, 1995 letter that the screening process would be specified in their Operational Quality Assurance Program Topical Report (OQAPTR) and that procedure changes that do not require a safety evaluation must be approved by cognizant management and a senior reactor operator.

Based on the screening process and procedure change approval by cognizant management and a senior reactor operator, the staff finds the proposed change to Section 6.5.1.6.a with respect to the review of procedures acceptable.

The licensee proposes to revise the current requirement (Sections 6.5.2.7.a and 6.5.2.7.1) that the MSRC shall be responsible for the review of safety evaluations for changes to procedures, equipment or systems and tests or experiments completed under the provisions of 10 CFR 50.59, to verify that such actions do not constitute an unreviewed safety question. The proposed change would limit the MSRC review to samples of safety evaluations after 100% review to select the most safety significant as programmatically discussed in the Updated Final Safety Analysis Report (UFSAR). The licensee will also continue to assess the effectiveness of the safety evaluation program.

The licensee, in their letter dated March 7, 1995, stated that they will add to their OQAPTR a statement that the MSRC will screen 100% of all safety evaluations for safety significance; and additionally do in-depth reviews of a sample of safety evaluations based on safety significance and a sampling plan criteria.

Based on the commitment for the MSRC to perform a 100% screening of safety evaluations and the performance of in-depth review of a sample of safety evaluations based on safety significance, the staff finds the requested change with respect to the MSRC review of safety evaluations acceptable.

The licensee proposes to delete the special requirements (Section 6.8.3) for the review and approval of temporary changes to procedures. These procedures would then be processed as all other procedures and procedure changes. The staff finds these changes acceptable as it does not diminish the level of review and approval of temporary procedures.

Finally, the licensee proposes to revise the Section 6.5.2.7.1 requirement that the MSRC review reports and meeting minutes of the SNSOC. The requirement would be revised to require reviewing only a representative sample of reports and meeting minutes of the SNSOC. The staff finds this change acceptable as it is consistent with the Improved Standard TS.

### 3.0 SUMMARY

The staff finds that the licensee's proposed changes meet the appropriate design criteria of Sections 13.4 and 13.5.1 of NUREG 0800, the Standard Review Plan, and are, therefore, acceptable.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendment. The State official had no comment.

### 5.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Allenspach

Date: May 2, 1995