September 7, 1993

Docket Nos. 50-338
and 50-339

DISTRIBUTION
See attached sheet

Mr. W. L. Stewart
Senior Vice President - Nuclear
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: NORTH ANNA UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: (TAC NOS. M87023 AND M87024)

The Commission has issued the enclosed Amendment Nos. 173 and 154 to Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The amendments revise the Technical Specifications (TS) in response to your letter dated July 2, 1993.

The amendments revise the NA-1&2 TS 3/4.6.1.2 by removing the schedular requirements for Type A tests to be performed specifically at 40±10 month intervals and instead reference Type A testing in accordance with Appendix J to 10 CFR Part 50. The proposed changes also include several editorial/administrative changes.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed By)
Leon B. Engle, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 173 to NPF-4
2. Amendment No. 154 to NPF-7
3. Safety Evaluation

Distribution
See next page

cc w/enclosures:
See next page **OFFICIAL RECORD COPY DOCUMENT NAME:\C:\AUTOS\NOANNA\NA87023.AMD**

OFFICE	LA:PDII-2	PM:PDII-2	D:PDII-2	OGC	SCSB
NAME	ETana ETT	LEngle	HBerkow	R Bachmann	RBarrett
DATE	07/28/93	07/29/93	07/29/93	8/11/93	8/12/93

CP-1

With attached Insert

D01

cc:
Mr. William C. Porter, Jr.
County Administrator
Louisa County
P.O. Box 160
Louisa, Virginia 23093

Michael W. Maupin, Esq.
Hunton and Williams
Riverfront Plaza, East Tower
951 E. Byrd Street
Richmond, Virginia 23219

Dr. W. T. Lough
Virginia State Corporation Commission
Division of Energy Regulation
P.O. Box 1197
Richmond, Virginia 23209

Old Dominion Electric Cooperative
4201 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. M. L. Bowling, Manager
Nuclear Licensing & Programs
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Office of the Attorney General
Supreme Court Building
101 North 8th Street
Richmond, Virginia 23219

Senior Resident Inspector
North Anna Power Station
U.S. Nuclear Regulatory Commission
Route 2, Box 78
Mineral, Virginia 231172

Robert B. Strobe, M.D., M.P.H.
State Health Commissioner
Office of the Commissioner
Virginia Department of Health
P.O. Box 2448
Richmond, Virginia 23218

Regional Administrator, RII
U.S. Nuclear Regulatory Commission
101 Marietta Street, N.W., Suite 2900
Atlanta, Georgia 30323

Mr. G. E. Kane, Manager
North Anna Power Station
P.O. Box 402
Mineral, Virginia 23117

MEMORANDUM DATED: September 7, 1993

AMENDMENT NO. 173 TO FACILITY OPERATING LICENSE NO. NPF-4-NORTH ANNA UNIT 1
AMENDMENT NO. 154 TO FACILITY OPERATING LICENSE NO. NPF-7-NORTH ANNA UNIT 2

Docket File
NRC & Local PDRs
PDII-2 Reading
S. Varga, 14/E/4
G. Lainas, 14/H/3
H. Berkow
E. Tana
L. Engle
OGC
D. Hagan, 3302 MNBB
G. Hill (2), P-137
C. Grimes, 11/F/23
ACRS (10)
OPA
OC/LFMB
M. Sinkule, R-II

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 173
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated July 2, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 173 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 7, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 173

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages	Insert Pages
3/4 6-1	3/4 6-1
3/4 6-2	3/4 6-2
3/4 6-3	3/4 6-3

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

a. At least once per 31 days by verifying that all penetrations* not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.3.1., and

b. By verifying that each containment air lock is OPERABLE per Specification 3.6.1.3.

c. After each closing of the equipment hatch, by leak rate testing the equipment hatch seals with gas at P_a, greater than or equal to 44.1 psig, and verifying that when the measured leakage rate for these seals is added to the leakage rates determined pursuant to Specification 4.6.1.2.b for all other Type B and C penetrations, the combined leakage rate is less than or equal to 0.60 L_a.

d. Each time containment integrity is established after vacuum has been broken by pressure testing the butterfly isolation valves in the containment purge lines and the containment vacuum ejector line.

* Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked sealed or otherwise sealed in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days.

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. An overall integrated leakage rate of less than or equal to L_a, 0.1 percent by weight of the containment air per 24 hours, at the calculated peak containment pressure P_a, greater than or equal to 44.1 psig.

b. A combined leakage rate of less than or equal to 0.60 L_a for all penetrations and valves subject to Type B and C tests, when pressurized to P_a, greater than or equal to 44.1 psig.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding 0.75 L_a or (b) with the measured combined leakage rate for all penetrations and valves subject to Type B and C tests exceeding 0.60 L_a, restore the overall integrated leakage rate to less than 0.75 L_a and the combined leakage rate for all penetrations subject to Type B and C tests to less than or equal to 0.60 L_a prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of either ANSI N45.4-1972 for leakage rate point data analysis or ANSI/ANS-56.8-1987 for mass point data analysis with a minimum test duration of 24 hours.

a. Type A (Overall Integrated Containment Leakage Rate) tests shall be conducted in accordance with the requirements specified in Appendix J to 10 CFR 50.

b. Type B and C tests shall be conducted with gas at P_a, greater than or equal to 44.1 psig, at intervals no greater than 24 months except for tests involving:

1. Air locks.

2. Penetrations using continuous leakage monitoring systems.

SURVEILLANCE REQUIREMENTS (Continued)

c. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.

d. Type B tests for penetrations employing a continuous leakage monitoring system shall be conducted at Pa, greater than or equal to 44.1 psig, at intervals no greater than once per 3 years.

e. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.

f. The provisions of Specification 4.0.2 are not applicable.

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be OPERABLE with:

a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and

b. An overall air lock leakage rate of less than or equal to 0.05 L_a at P_a greater than or equal to 44.1 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With one containment air lock door inoperable:

1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.+

2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be locked closed at least once per 31 days.

3. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

4. The provisions of Specification 3.0.4 are not applicable.

b. With a containment air lock inoperable, except as the result of an inoperable air lock door, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

a. *Within 72 hours following closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying that the seal leakage is less than 0.01 L_a as determined by precision flow measurements when measured for at least 30 seconds with the volume between the seals at a pressure of greater than or equal to 44.1 psig.

b. At least once per 6 months by conducting an overall air lock leakage test at greater than or equal to P_a, 44.1 psig, and by verifying that the overall air lock leakage rate is within its limit#, and

c. At least once per 18 months during shutdown by verifying that only one door in each air lock can be opened at a time.

+Entry to repair the inner air lock door, if inoperable, is allowed.
*Exempt to Appendix "J" of 10 CFR Part 50.
#The provisions of Specification 4.0.2 are not applicable.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 154
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated July 2, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 154 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 7, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 154

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages	Insert Pages
3/4 6-1	3/4 6-1
3/4 6-2	3/4 6-2
3/4 6-3	3/4 6-3

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

a. At least once per 31 days by verifying that all penetrations* not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.3.1., and

b. By verifying that each containment air lock is OPERABLE per Specification 3.6.1.3.

c. After each closing of the equipment hatch, by leak rate testing the equipment hatch seals with gas at P_a, greater than or equal to 44.1 psig, and verifying that when the measured leakage rate for these seals is added to the leakage rates determined pursuant to Specification 4.6.1.2.b for all other Type B and C penetrations, the combined leakage rate is less than or equal to 0.60 L_a.

d. Each time containment integrity is established after vacuum has been broken by pressure testing the butterfly isolation valves in the containment purge lines and the containment vacuum ejector line.

* Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked sealed or otherwise sealed in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days.

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. An overall integrated leakage rate of less than or equal to L_a, 0.1 percent by weight of the containment air per 24 hours, at the calculated peak containment pressure P_a, greater than or equal to 44.1 psig.

b. A combined leakage rate of less than or equal to 0.60 L_a for all penetrations and valves subject to Type B and C tests, when pressurized to P_a, greater than or equal to 44.1 psig.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding 0.75 L_a or (b) with the measured combined leakage rate for all penetrations and valves subject to Type B and C tests exceeding 0.60 L_a, restore the overall integrated leakage rate to less than 0.75 L_a and the combined leakage rate for all penetrations subject to Type B and C tests to less than or equal to 0.60 L_a prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of either ANSI N45.4-1972 for leakage rate point data analysis or ANSI/ANS-56.8-1987 for mass point data analysis with a minimum test duration of 24 hours.

a. Type A (Overall Integrated Containment Leakage Rate) tests shall be conducted in accordance with the requirements specified in Appendix J to 10 CFR 50.

b. Type B and C tests shall be conducted with gas at P_a, greater than or equal to 44.1 psig, at intervals no greater than 24 months except for tests involving:

   1. Air locks.

   2. Penetrations using continuous leakage monitoring systems.

SURVEILLANCE REQUIREMENTS (Continued)

c. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.

d. Type B tests for penetrations employing a continuous leakage monitoring system shall be conducted at Pa, greater than or equal to 44.1 psig, at intervals no greater than once per 3 years.

e. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.

f. The provisions of Specification 4.0.2 are not applicable.

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be OPERABLE with:

a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and

b. An overall air lock leakage rate of less than or equal to 0.05 L_a at P_a, greater than or equal to 44.1 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With one containment air lock door inoperable:

1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.+

2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be locked closed at least once per 31 days.

3. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

4. The provisions of Specification 3.0.4 are not applicable.

b. With a containment air lock inoperable, except as the result of an inoperable air lock door, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

a. *Within 72 hours following closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying that the seal leakage is less than 0.01 L_a as determined by precision flow measurements when measured for at least 30 seconds with the volume between the seals at a pressure of greater than or equal to 44.1 psig.

+Entry to repair the inner air lock door, if inoperable, is allowed.
*Exempt to Appendix "J" of 10 CFR Part 50.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 173 AND 154 TO

FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2

DOCKET NOS. 50-338 AND 50-339

1.0 INTRODUCTION

By letter dated July 2, 1993, the Virginia Electric and Power Company (the licensee) proposed changes to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1&2 (NA-1&2). Specifically, the changes would revise the NA-1&2 TS 3/4.6.1.2 by removing the schedular requirements for Type A tests to be performed at 40±10 month intervals and instead reference Type A testing in accordance with Appendix J to 10 CFR Part 50. The changes would also include several editorial/administrative changes. The NA-1&2 TS currently require a set of three containment integrated leakage rate (Type A) tests be performed specifically at 40±10 month intervals during each 10-year service period with the third test of each set performed during the shutdown for the 10-year plant inservice inspection. Appendix J to 10 CFR Part 50 requires that a Type A test of the containment be performed periodically. These tests are required to be scheduled as a set of three tests, to be performed at approximately equal intervals, during each 10-year service period with the third test of each set to coincide with the shutdown for the 10-year plant inservice inspection. While the NA-1&2 TS leakage rate testing requirements essentially duplicate the requirements in Appendix J, the TS additionally require the Type A test be performed at 40±10 month intervals. This TS requirement to conduct Type A tests at 40±10 month intervals is too restrictive. Therefore, the licensee proposes to revise the NA-1&2 TS to delete the detailed surveillance schedule for the Type A tests and instead reference performance of Type A testing in accordance with Appendix J to 10 CFR 50.

2.0 Discussion

The TS-required schedule provides only a 20-month window for scheduling of Type A tests. This requirement is not appropriate for a facility like NA-1&2 which is on an 18-month fuel cycle. An 18-month fuel cycle does not provide sufficient flexibility for three tests within a 10-year service period when limited by the stipulation that the tests be performed at 40±10 months intervals.

During the first 10-year service period for both NA-1&2, TS change requests were submitted requesting approval to deviate from the TS-specified Type A test schedule. Each of these change requests proposed to alter the schedule by only a few months. However, since such a change has been needed in the past and needed for the current and future service period, the TS requirement is evidently too restrictive.

For 18-month fuel cycles, the Type A tests must be conducted every other refueling outage to be within the 30- to 50-month range allowed by the TS for the first and second test of the service period. Then, however, an extension is needed to make the third and last test coincide with the 10-year inservice inspection outage. Therefore, the licensee would revise the NA-1&2 TS to refer to 10 CFR Part 50, Appendix J, which requires only that the tests be conducted "at approximately equal intervals" during the 10-year service period with the third test being conducted during the service inspection outage. Typically, this would result in tests being conducted at intervals of 4½ years, 3 years, and 3 years over a 10-year period.

3.0 Technical Specfication Changes:

TS Surveillance Requirement 4.6.1.2.a would be revised to remove the specific requirement that the Type A tests be performed at 40±10 month intervals with the third test of each set conducted during the shutdown for the 10-year plant inservice inspection. The revised requirement would read:

> "Type A (Overall Integrated Containment Leakage Rate) tests shall be conducted in accordance with the requirements specified in Appendix J to 10 CFR 50."
>
> TS Surveillance Requirements 4.6.1.2.b and 4.6.1.2.c would be deleted. The licensee stated that these surveillance requirements are also specified in Appendix J to 10 CFR 50 and need not be reiterated in the technical specifications. However, surveillance requirement 4.6.1.2.c.3 is not specified in Appendix J to the same level of detail. This surveillance requirement concerns the accuracy of supplemental testing. Section III.A.3(b) of Appendix J states the requirements for the accuracy of supplemental testing in terms of a fraction of L_a, while surveillance requirement 4.6.1.2.c.3 is related to a specific test and its success criteria. Although the requirement is not specified in the same level of detail in Appendix J as in the technical specification surveillance requirement, the staff finds the deletion of this surveillance requirement acceptable because: (1) the requirement for a supplemental test and general requirements for the accuracy of the test are specified in Appendix J, and (2) it is not necessary for the technical specifications to contain the level of detail specified in section 4.6.1.2.c.3. In fact, the new Westinghouse Standard Technical Specifications are consistent with the licensee's proposal in this respect. The remaining Surveillance Requirements would be renumbered for continuity. Consistent with this renumbering, the reference in TS

4.6.1.1.c is revised from "Specification 4.6.1.2.d" to "Specification 4.6.1.2.b."

The format of TS 3.6.1.2.a would be revised to eliminate the single subsubparagraph and delete the word "or" at the end of the statement. Both the subsubparagraph format and the conjunction word are unnecessary. The original format was taken from the Standard Technical Specifications (STS) for Westinghouse Pressurized Water Reactors. The STS had two options under Specification 3.6.1.2.a; however, NA-1&2 were not licensed for the second option. This proposed format change does not change the intent of the TS.

TS 3.6.1.2.a would be revised to add the phrase "the calculated peak containment pressure" as a description in front of the term "P_a." The addition of this phrase provides clarification and does not change the intent of the TS.

Remove the footnotes on the bottom of the NA-1&2 TS Page 3/4 6-2. Removing these special requirement footnotes would not constitute a change to the TS. The special requirements were applicable to a specific cycle or outage. The special requirements have been completed and the footnotes are no longer applicable.

4.0 Evaluation

Satisfactory leakage results are a requirement for the establishment of containment operability. Neither the general frequency nor the required number of Type A tests would be changed by the proposed changes. Also, the maximum allowable leakage rate at the calculated peak containment pressure would not be changed. Only the detailed 40±10 month test interval would be changed to provide more flexibility. Type A, B, and C tests would continue to be performed in accordance with Appendix J to 10 CFR 50. Type A test acceptance criteria would not be changed and combined leakage of penetrations subject to Type B and C tests would be maintained within the required limits. Also, the proposed changes do not impact the design basis of the containment and would not change the response of containment during a design basis accident. Finally, the testing method, acceptance criteria, and Bases to the TS are not changed by the proposed revisions to the TS. Therefore, based on all of the above, the staff finds the proposed changes to be acceptable.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comment.

6.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (58 FR 41517). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Leon Engle

Date: September 7, 1993