



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 14, 1991

Docket Nos. 50-338
and 50-339

Mr. W. L. Stewart
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: NORTH ANNA UNITS 1 AND 2 - CORRECTIONS TO AMENDMENTS NOS. 143 and 126
(TAC NOS. 73790 AND 73791)

On February 20, 1991, the Commission issued Amendment Nos. 143 and 126 for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The amendments added valves to the NA-1&2 Technical Specifications (TS) which do not need to be vented to the containment atmosphere nor drained of water during Type A (containment integrated leak rate) tests and the leakage rates measured by the Type C tests on these valves need not be added to the Type A test results. The amendments also identified valves that are associated with "water-filled" penetrations for which a Type C test penalty would not be added to the Type A test results.

By letter dated March 8, 1991, you stated that you noted an administrative error and an implementation issue during your review of the amendments. The administrative error is contained on page 3/4 6-30, valve number 1-FP-272, for NA-1. In your request for amendments, the pound sign (#) for valve number 1-FP-272 was deleted, in order to denote that the valve is subject to Type C leakage testing. However, the amendment for NA-1 was issued with the pound sign retained. Enclosed is the corrected page 3/4 6-30 for NA-1.

The implementation issue concerns valve number 2-FP-79 on page 3/4 6-30 for NA-2. Valve 2-FP-79 was proposed to be added to Table 3.6-1 (without a pound sign) and thus would be included in the Type C test program. However, Type C testing of valve 2-FP-79 was not performed during the recent NA-2 refueling outage which was completed on November 2, 1990, since the amendment had not yet been issued. NA-2 is currently operating at full power and Type C testing at this time is impractical. You noted in your March 8, 1991 letter that it is your intent to perform Type C testing on this valve during the next scheduled refueling outage. You also stated that the fire protection containment penetration for this valve also contains two manual isolation valves in series which are located outside containment, and are currently in the Type C test program. These valves were successfully Type C tested during the 1990 NA-2 refueling outage.

DFOL
11

March 14, 1991

Mr. W.L. Stewart

- 2 -

Due to this implementation issue, enclosed is a corrected page 3/4 6-30 for NA-2, with a footnote added which allows you to perform Type C testing for valve 2-FP-79 during the next scheduled refueling outage.

Sincerely,

Original signed by

Leon B. Engle, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc w/enclosures:
See next page

DISTRIBUTION
See next page

OFC	:LA:PDII-2	:PM:PDII-2	:D:PDII-2	:	:
NAME	:D Miller	:L Engle: jkd	:H Berkow	:	:
DATE	:03/14/91	:03/14/91	:03/14/91	:	:

OFFICIAL RECORD COPY

Document Name: CORREC TO AMEND 143/126 NA1&2

9103250336 910314
PDR ADOCK 05000338
P PDR

Mr. W. L. Stewart
Virginia Electric & Power Company

North Anna Power Station
Units 1 and 2

cc:

Mr. William C. Porter, Jr.
County Administrator
Louisa County
P.O. Box 160
Louisa, Virginia 23093

C. M. G. Buttery, M.D., M.P.H.
Department of Health
109 Governor Street
Richmond, Virginia 23219

Michael W. Maupin, Esq.
Hunton and Williams
P. O. Box 1535
Richmond, Virginia 23212

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, Georgia 30323

Mr. W. T. Lough
Virginia Corporation Commission
Division of Energy Regulation
P. O. Box 1197
Richmond, Virginia 23209

Mr. G. E. Kane, Manager
North Anna Power Station
P.O. Box 402
Mineral, Virginia 23117

Old Dominion Electric Cooperative
c/o Executive Vice President
Innsbrook Corporate Center
4222 Cox Road, Suite 102
Glen Allen, Virginia 23060

Mr. J. P. O'Hanlon
Vice President - Nuclear Services
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. E. Wayne Harrell
Vice President - Nuclear Operations
Virginia Electric and Power Co.
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. Martin Bowling
Manager - Nuclear Licensing
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. Patrick A. O'Hare
Office of the Attorney General
Supreme Court Building
101 North 8th Street
Richmond, Virginia 23219

Senior Resident Inspector
North Anna Power Station
U.S. Nuclear Regulatory Commission
Route 2, Box 78
Mineral, Virginia 23117

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
F. CHECK		
1. 1-CC-193	Component Cooling Water to RHR System and Excess Letdown Heat Exchanger	NA
2. 1-CC-198	Component Cooling Water to RHR System and Excess Letdown Heat Exchanger	NA
3. 1-SI-79##	High Head Safety Injection, Boron Injection to RCS	NA
4. 1-CC-572	Component Cooling Water to Containment Air Recirculation Coils	NA
5. 1-CC-559	Component Cooling Water to Containment Air Recirculation Coils	NA
6. 1-CC-546	Component Cooling Water to Containment Air Recirculation Coils	NA
7. 1-CH-322##	Charging Line	NA
8. 1-CC-154	Component Cooling Water to Reactor Coolant Pumps	NA
9. 1-CC-119	Component Cooling Water to Reactor Coolant Pumps	NA
10. 1-CC-84	Component Cooling Water to Reactor Coolant Pumps	NA
11. 1-CH-402	Reactor Coolant Pumps, Seal Water Return	NA
12. 1-SI-110	Safety Injection Accumulator Make Up	NA
13. 1-SI-185##	High Head Safety Injection to RCS Except Boron Injection Line	NA
14. 1-HC-18	Discharge From Containment Atmosphere Cleanup System (Hydrogen Recombiner)	NA
15. 1-HC-14	Discharge From Containment Atmosphere Cleanup System (Hydrogen Recombiner)	NA
16. 1-CH-380#	Reactor Coolant Pump Seal Water Supply	NA
17. 1-CH-336#	Reactor Coolant Pump Seal Water Supply	NA
18. 1-CH-358#	Reactor Coolant Pump Seal Water Supply	NA
19. 1-IA-149	Air Radiation Monitor Return	NA
20. 1-RC-149	Primary Grade Water to Pressurizer Relief Tank	NA
21. 1-CH-330##	Loop Fill Header	NA

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
22. 1-IA-55	Instrument Air Line	NA
23. 1-SI-106	Nitrogen to Pressurizer Relief Tank and SI Accumulators	NA
24. 1-SI-206##	LHSI Pump Discharge to Reactor Coolant System Hot Legs	NA
25. 1-SI-207##	LHSI Pump Discharge to Reactor Coolant System Hot Legs	NA
26. 1-SI-195##	LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA
27. 1-SI-197##	LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA
28. 1-SI-199##	LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA
29. 1-QS-19**	Quench Spray Pump Discharge	NA
30. 1-QS-11**	Quench Spray Pump Discharge	NA
31. 1-RS-27**	Recirculation Spray Pump Discharge	NA
32. 1-RS-18**	Recirculation Spray Pump Discharge	NA
33. 1-VP-12	Air Ejector Vent	NA
34. 1-SI-90##	High Head Safety Injection to RCS Except Boron Injection Line	NA
35. 1-SI-201##	High Head Safety Injection to RCS Except Boron Injection Line	NA
36. Deleted		
37. 1-FW-47#	Feedwater to Steam Generators	NA
38. 1-FW-111#	Feedwater to Steam Generators	NA
39. 1-FW-79#	Feedwater to Steam Generators	NA
40. 1-WT-50#	Chemical Feed Lines	NA
41. 1-WT-66#	Chemical Feed Lines	NA
42. 1-WT-38#	Chemical Feed Lines	NA
43. 1-FW-68#	Auxiliary Feedwater to Steam Generator	NA
44. 1-FW-100#	Auxiliary Feedwater to Steam Generator	NA
45. 1-FW-132#	Auxiliary Feedwater to Steam Generator	NA
46. 1-FP-272	Fire Protection Hose Standpipe	NA
47. 1-RS-123#	Casing Cooling to Outside Recirc Spray Pump	NA
48. 1-RS-138#	Casing Cooling to Outside Recirc Spray Pump	NA

NORTH ANNA - UNIT 1

3/4 6-30

Amendment No. 76, 43, 143,

TABLE 3.6-1 (Continued)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC.)</u>
37. 2-FW-94#	Feedwater to Steam Generators	NA
38. 2-FW-126#	Feedwater to Steam Generators	NA
39. 2-WT-41#	Chemical Feed Lines	NA
40. 2-WT-53#	Chemical Feed Lines	NA
41. 2-WT-69#	Chemical Feed Lines	NA
42. 2-FW-70#	Auxiliary Feedwater to Steam Generator	NA
43. 2-FW-102#	Auxiliary Feedwater to Steam Generator	NA
44. 2-FW-134#	Auxiliary Feedwater to Steam Generator	NA
45. 2-RS-103#	Casing Cooling to Outside Recirc Spray Pump	NA
46. 2-RS-118#	Casing Cooling to Outside Recirc Spray Pump	NA
47. 2-FP-79 (1)	Fire Protection Supply (Penetration 34)	NA

(1) Type C testing for valve 2-FP-79 shall be implemented prior to restart for Cycle 9 operation.

TABLE 3.6-1 (Continued)

	VALVE NUMBER	FUNCTION	ISOLATION TIME (SEC.)
G.	STEAM LINE ISOLATION		
	1. TV-MS-101A#	Main Steam Line Trip Valve	5
	2. TV-MS-101B#	Main Steam Line Trip Valve	5
	3. TV-MS-101C#	Main Steam Line Trip Valve	5

H. RELIEF

None

-
- # Valve not subject to Type "C" leakage test
 - * Valve position maintained by administrative control
 - NA - Not Applicable
 - ** - Requires testing per Technical Specifications 4.6.3.1.1a. or 4.6.3.1.2d.
 - ## Type "C" leakage values are not required to be added to the Type A leakage rate.

DATED: March 14, 1991

CORRECTION TO:

AMENDMENT NO. 143 TO FACILITY OPERATING LICENSE NO. NPF-4-NORTH ANNA UNIT 1

AMENDMENT NO. 126 TO FACILITY OPERATING LICENSE NO. NPF-7-NORTH ANNA UNIT 2

Docket File

NRC & Local PDRs

PDII-2 Reading

S. Varga, 14/E/4

G. Lainas, 14/H/3

H. Berkow

D. Miller

L. Engle

OGC-WF

D. Hagan, 3302 MNBB

E. Jordan, 3701 MNBB

B. Grimes, 9/A/2

G. Hill (8), P-137

Wanda Jones, P-130A

J. Calvo, 11/F/23

ACRS (10)

GPA/PA

OC/LFMB

M. Sinkule, R-II

Others as required

cc: Plant Service list