June 25, 1991

Docket Nos. 50-338
and 50-339

Mr. W. L. Stewart
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: NORTH ANNA UNITS 1 AND 2 - CORRECTION TO AMENDMENT NOS. 146 AND 130 (TAC NOS. 77066 AND 77067)

On June 7, 1991, we issued Amendment Nos. 146 and 130 for the North Anna Power Station, Units 1 and 2 (NA-1&2). The amendments modified the Technical Specifications (TS) by referencing the cycle-specific parameter limits in a Core Operating Limits Report (COLR). In addition, we denied your request to include boron concentration in the COLR, since we are currently sponsoring a research effort to determine if the floor value of the boron concentration should be retained or eliminated.

You have subsequently informed us of errors on TS page 6-17 for both NA-1&2. Some of your requested changes regarding boron concentration were not removed prior to issuance of the amendments. Enclosed are the correct pages 6-17 for both NA-1&2.

Sincerely,

(Original Signed By)

Leon B. Engle, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosures:
See next page

OFC	LA:PD22	PM:PD22	D:PD22		
NAME	DMiller	LEngle	HBerkow		
DATE	6/25/91	6/25/91	6/25/91		

OFFICIAL RECORD COPY
Document Name: NORTH ANNA CORRECTION

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CORE OPERATING LIMITS REPORT

6.9.1.7.a Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT before each reload cycle or any remaining part of a reload cycle for the following:

1. Moderator Temperature Coefficient BOC and EOC limits, and 300 ppm and 60 ppm surveillance limits for Specification 3/4.1.1.4,
2. Shutdown Bank Insertion Limit for Specification 3/4.1.3.5,
3. Control Bank Insertion Limits for Specification 3/4.1.3.6,
4. Axial Flux Difference limits for Specification 3/4.2.1,
5. Heat Flux Hot Channel Factor, K(Z), N(Z) for Specification 3/4.2.2, and
6. Nuclear Enthalpy Rise Hot Channel Factor, and Power Factor Multiplier, for Specification 3/4.2.3.

6.9.1.7.b The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC as identified in 6.9.1.7.e.

6.9.1.7.c The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.7.d The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.

6.9.1.7.e REFERENCES

1. VEP-FRD-42, Rev. 1-A, "Reload Nuclear Design Methodology," September 1986.

   (Methodology for LCO 3.1.1.4 - Moderator Temperature Coefficient, LCO 3.1.3.5 - Shutdown Bank Insertion Limit, LCO 3.1.3.6 - Control Bank Insertion Limits, LCO 3.2.2 - Heat Flux Hot Channel Factor, LCO 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).

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DATED: June 25, 1991

CORRECTION TO:
AMENDMENT NO. 146 TO FACILITY OPERATING LICENSE NO. NPF-4-NORTH ANNA UNIT 1
AMENDMENT NO. 130 TO FACILITY OPERATING LICENSE NO. NPF-7-NORTH ANNA UNIT 2

Docket File
NRC & Local PDRs
PDII-2 Reading
S. Varga, 14/E/4
G. Lainas, 14/H/3
H. Berkow
D. Miller
L. Engle
OGC-WF
D. Hagan, 3302 MNBB
E. Jordan, 3302 MNBB
B. Grimes, 9/A/2
G. Hill (8), P-137
Wanda Jones, P-130A
C. Grimes, 11/F/23
T. Huang, 8-E-23
ACRS (10)
GPA/PA
OC/LFMB
M. Sinkule, R-II

cc: Plant Service list