

August 7, 1990

Docket Nos. 50-338  
and 50-339

DISTRIBUTION  
See attached sheet

Mr. W. L. Stewart  
Senior Vice President - Nuclear  
Virginia Electric and Power Company  
5000 Dominion Blvd.  
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: NORTH ANNA UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: INDEPENDENT/  
OPERATIONAL EVENT REVIEW GROUP (TAC NOS. 75156 AND 75157)

The Commission has issued the enclosed Amendment Nos. 135 and 118 to Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The amendments revise the Technical Specifications (TS) in response to your letter dated February 1, 1990, as supplemented July 30, 1990.

The amendments revise NA-1&2 TS Section 6.5 and bring the NA-1&2 TS in conformance with the Westinghouse Standard TS. The changes replace the Independent/Operational Event Review Group with a new Management Safety Review Committee (MSRC) as the organization responsible for the NA-1&2 offsite review committee. Also, the audits required by the TS that were the responsibility of the Quality Assurance Department will now be performed under the cognizance of the MSRC. Finally, the amendments delete or modify management titles to reflect recent changes.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Leon B. Engle, Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 135 to NPF-4
2. Amendment No. 118 to NPF-7
3. Safety Evaluation

cc w/enclosures:

See next page

\*Previously concurred

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NAME	: DMiller*	: LEngle	: HBankow	: *	:	:	:
DATE	: 4/19/90	: 07/17/90	: 7/7/90	: 4/20/90	:	:	:

Mr. W. L. Stewart  
Virginia Electric & Power Company

North Anna Power Station  
Units 1 and 2

cc:

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Mr. J. P. O'Hanlon  
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Mr. Patrick A. O'Hare  
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Senior Resident Inspector  
North Anna Power Station  
U.S. Nuclear Regulatory Commission  
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Mineral, Virginia 23117

DATED: August 7, 1990

AMENDMENT NO. 135 TO FACILITY OPERATING LICENSE NO. NPF-4-NORTH ANNA UNIT 1  
AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. NPF-7-NORTH ANNA UNIT 2

Docket File

NRC & Local PDRs

PDII-2 Reading

S. Varga, 14/E/4

G. Lainas, 14/H/3

H. Berkow

D. Miller

L. Engle

OGC-WF

D. Hagan, 3302 MNBB

E. Jordan, 3302 MNBB

B. Grimes, 9/A/2

G. Hill (8), P1-137

Wanda Jones, P-130A

J. Calvo, 11/F/23

ACRS (10)

GPA/PA

OC/LFMB

PD Plant-specific file [Gray File]

M. Sinkule, R-II

Others as required

cc: Plant Service list

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 135  
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated February 1, 1990, as supplemented July 30, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

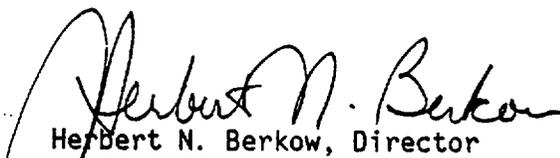
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 135, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 7, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 135

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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## ADMINISTRATIVE CONTROLS

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- e. The management position responsible for health physics shall have direct access to that onsite individual having responsibility for overall facility management. Health physics personnel shall have the authority to cease any work activity when worker safety is jeopardized or in the event of unnecessary personnel radiation exposures.

### FACILITY STAFF

- 6.2.2 The Facility organization shall be as shown in the UFSAR.
  - a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
  - b. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODES 1, 2, 3 or 4, at least one licensed Senior Reactor Operator shall be in the Control Room.
  - c. A health physics technician# shall be onsite when fuel is in the reactor.
  - d. ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
  - e. A Fire Brigade of at least 5 members shall be maintained onsite at all times#. The Fire Brigade shall not include the minimum shift crew shown in Table 6.2-1 or any personnel required for other essential functions during a fire emergency.

#The health physics technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The Station Manager shall be responsible for overall facility operation. In his absence, the Assistant Station Manager (Operations and Maintenance) shall be responsible for overall facility operation. During the absence of both, the Station Manager shall delegate in writing the succession to this responsibility.

6.1.2 The Shift Supervisor (or during his absence from the Control Room, a designated individual) shall be responsible for the Control Room command function and shall be the only individual that may direct the licensed activities of licensed operators. A management directive to this effect, signed by the Senior Vice President - Nuclear, shall be reissued to all station personnel on an annual basis.

### 6.2 ORGANIZATION

#### ONSITE AND OFFSITE ORGANIZATION

##### 6.2.1 Onsite and Offsite Organization

An onsite and an offsite organization shall be established for facility operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the UFSAR.
- b. The Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President - Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The management position responsible for training of the operating staff and the management position responsible for the quality assurance functions shall have sufficient organizational freedom including sufficient independence from cost and schedule when opposed to safety considerations.

## ADMINISTRATIVE CONTRLS

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANS 3.1-(12/79 Draft) for comparable positions and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, except for (1) the Superintendent - Radiological Protection who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

### 6.4 TRAINING

6.4.1 The Manager - Nuclear Training is responsible for ensuring that retraining and replacement training programs for the facility staff are maintained and that such programs meet or exceed the requirements and recommendations of Section 5 of ANS 3.1-(12/79 Draft) and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the NSE.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 STATION NUCLEAR SAFETY AND OPERATING COMMITTEE (SNSOC)

##### FUNCTION

6.5.1.1 The SNSOC shall function to advise the Station Manager on all matters related to nuclear safety.

##### COMPOSITION

6.5.1.2 The SNSOC shall be composed of the :

Chairman:	Assistant Station Manager (Nuclear Safety and Licensing)
Vice Chairman and Member:	Assistant Station Manager (Operations and Maintenance)
Member:	Superintendent-Operations
Member:	Superintendent-Maintenance
Member:	Superintendent-Radiological Protection
Member:	Superintendent-Engineering

##### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SNSOC Chairman to serve on a temporary basis; however, no more than one alternate shall participate as a voting member in SNSOC activities at any one time.

## ADMINISTRATIVE CONTROLS

### MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

### QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

### RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

- a. Review of 1) all procedures required by Specifications 6.8.1, 6.8.2 and 6.8.3 and changes thereto, 2) all programs required by Specification 6.8.4 and changes thereto, 3) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Station Manager.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President-Nuclear Operations and the MSRC.
- f. Review of all REPORTABLE EVENTS and Special Reports.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Station Manager.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Station Manager.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Station Manager.

## ADMINISTRATIVE CONTROLS

- k. Review of every unplanned onsite release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President-Nuclear Operations and the Management Safety Review Committee.
- l. Review changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.

### AUTHORITY

6.5.1.7 The SNSOC shall:

- a. Provide written approval or disapproval of items considered under 6.5.1.6(a) through (c) above. SNSOC approval shall be certified in writing by an Assistant Station Manager.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Nuclear Operations and the Management Safety Review Committee (MSRC) of disagreement between the SNSOC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

### RECORDS

6.5.1.8 The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Station Manager, Vice President-Nuclear Operations and the MSRC.

### 6.5.2 MANAGEMENT SAFETY REVIEW COMMITTEE (MSRC)

#### FUNCTION

6.5.2.1. The MSRC shall function to provide independent review of designated activities in the areas of:

- a. Station Operations
- b. Maintenance
- c. Reactivity Management
- d. Engineering
- e. Chemistry and Radiochemistry
- f. Radiological Safety
- g. Quality Assurance Practices
- h. Emergency Preparedness

## ADMINISTRATIVE CONTROLS

### COMPOSITION

6.5.2.2 The MSRC shall be composed of the MSRC Chairman and a minimum of four MSRC members. The Chairman and all members of the MSRC shall have qualifications that meet the requirements of Section 4.7 of ANSI/ANS 3.1-1979 Rev. 1 (Draft).

### ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the MSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in MSRC activities at any one time.

### CONSULTANTS

6.5.2.4 Consultants should be utilized as determined by the MSRC Chairman to provide expert advice to the MSRC.

### MEETING FREQUENCY

6.5.2.5 The MSRC shall meet at least once per calendar quarter.

### QUORUM

6.5.2.6 The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

### REVIEW

6.5.2.7 The MSRC shall be responsible for the review of:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.

## ADMINISTRATIVE CONTROLS

- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meetings minutes of the SNSOC.

## AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Emergency Plan and implementing procedures at least once per 12 months.
- f. The Security Plan and implementing procedures at least once per 12 months.
- g. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.
- h. The Fire Protection Program and implementing procedures at least once per 24 months.

## ADMINISTRATIVE CONTROLS

- i. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- l. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 12 months.
- m. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 12 months.

### AUTHORITY

6.5.2.9 The MSRC shall report to and advise the Senior Vice President - Nuclear on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

### RECORDS

6.5.2.10 Records of MSRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each MSRC meeting shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days of each meeting.
- b. Reports of reviews with safety significant findings encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

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## ADMINISTRATIVE CONTROLS

### 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SNSOC and the results of this review shall be submitted to the Vice President-Nuclear Operations and the MSRC.

### 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President-Nuclear Operations and MSRC shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSOC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the Vice President-Nuclear Operations and the MSRC within 14 days of the violation.

### 6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118  
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated February 1, 1990, as supplemented July 30, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

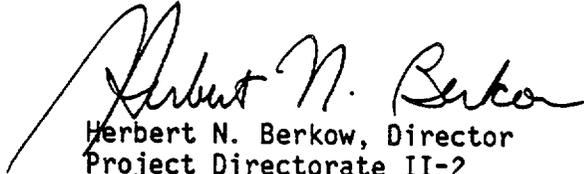
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 118, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 7, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 118

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The Station Manager shall be responsible for overall facility operation. In his absence, the Assistant Station Manager (Operations and Maintenance) shall be responsible for overall facility operation. During the absence of both, the Station Manager shall delegate in writing the succession to this responsibility.

6.1.2 The Shift Supervisor (or during his absence from the Control Room, a designated individual) shall be responsible for the Control Room command function and shall be the only individual that may direct the licensed activities of licensed operators. A management directive to this effect, signed by the Senior Vice President - Nuclear, shall be reissued to all station personnel on an annual basis.

### 6.2 ORGANIZATION

#### ONSITE AND OFFSITE ORGANIZATION

##### 6.2.1 Onsite and Offsite Organization

An onsite and an offsite organization shall be established for facility operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

- a. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the UFSAR.
- b. The Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c. The Vice President-Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
- d. The management position responsible for training of the operating staff and the management position responsible for the quality assurance functions shall have sufficient organizational freedom including sufficient independence from cost and schedule when opposed to safety considerations.

TABLE 6.2-1 (Continued)

- SS - Shift Supervisor with a Senior Reactor Operators License on Unit 2.
- SRO - Individual with a Senior Reactor Operators License on Unit 2.
- RO - Individual with a Reactor Operators License on Unit 2.
- AO - Auxiliary Operator
- STA - Shift Technical Advisor

Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 1, 2, 3 or 4, an individual (other than the Shift Technical Advisor) with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 5 or 6, an individual with a valid RO license (other than the Shift Technical Advisor) shall be designated to assume the Control Room command function.

Procedures will be established to insure that NRC policy statement guidelines regarding working hours established for employees are followed. In addition, procedures will provide for documentation of authorized deviations from these guidelines and that the documentation is available for NRC review.

## ADMINISTRATIVE CONTRLS

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANS 3.1-(12/79 Draft) for comparable positions and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, except for (1) the Superintendent - Radiological Protection who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

### 6.4 TRAINING

6.4.1 The Manager - Nuclear Training is responsible for ensuring that retraining and replacement training programs for the facility staff are maintained and that such programs meet or exceed the requirements and recommendations of Section 5 of ANS 3.1-(12/79 Draft) and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the NSE.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 STATION NUCLEAR SAFETY AND OPERATING COMMITTEE (SNSOC)

##### FUNCTION

6.5.1.1 The SNSOC shall function to advise the Station Manager on all matters related to nuclear safety.

##### COMPOSITION

6.5.1.2 The SNSOC shall be composed of the :

Chairman: Assistant Station Manager (Nuclear Safety and Licensing)  
Vice Chairman and Member: Assistant Station Manager (Operations and Maintenance)  
Member: Superintendent-Operations  
Member: Superintendent-Maintenance  
Member: Superintendent-Radiological Protection  
Member: Superintendent-Engineering

##### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SNSOC Chairman to serve on a temporary basis; however, no more than one alternate shall participate as a voting member in SNSOC activities at any one time.

## ADMINISTRATIVE CONTROLS

### MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

### QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

### RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

- a. Review of 1) all procedures required by Specifications 6.8.1, 6.8.2 and 6.8.3 and changes thereto, 2) all programs required by Specification 6.8.4 and changes thereto, 3) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Station Manager.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President-Nuclear Operations and the MSRC.
- f. Review of all REPORTABLE EVENTS and Special Reports.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Station Manager.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Station Manager.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Station Manager.

## ADMINISTRATIVE CONTROLS

- k. Review of every unplanned onsite release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President-Nuclear Operations and the Management Safety Review Committee.
- l. Review changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.

### AUTHORITY

6.5.1.7 The SNSOC shall:

- a. Provide written approval or disapproval of items considered under 6.5.1.6(a) through (c) above. SNSOC approval shall be certified in writing by an Assistant Station Manager.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President-Nuclear Operations and the Management Safety Review Committee (MSRC) of disagreement between the SNSOC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

### RECORDS

6.5.1.8 The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Station Manager, the Vice President-Nuclear Operations and the MSRC.

### 6.5.2 MANAGEMENT SAFETY REVIEW COMMITTEE (MSRC)

#### FUNCTION

6.5.2.1 The MSRC shall function to provide independent review of designated activities in the areas of:

- a. Station Operations
- b. Maintenance
- c. Reactivity Management
- d. Engineering
- e. Chemistry and Radiochemistry
- f. Radiological Safety
- g. Quality Assurance Practices
- h. Emergency Preparedness

## ADMINISTRATIVE CONTROLS

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### COMPOSITION

6.5.2.2 The MSRC shall be composed of the MSRC Chairman and a minimum of four MSRC members. The Chairman and all members of the MSRC shall have qualifications that meet the requirements of Section 4.7 of ANSI/ANS 3.1-1979 Rev. 1 (Draft).

### ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the MSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in MSRC activities at any one time.

### CONSULTANTS

6.5.2.4 Consultants should be utilized as determined by the MSRC Chairman to provide expert advice to the MSRC.

### MEETING FREQUENCY

6.5.2.5 The MSRC shall meet at least once per calendar quarter.

### QUORUM

6.5.2.6 The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

### REVIEW

- 6.5.2.7 The MSRC shall be responsible for the review of:
- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
  - b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
  - c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
  - d. Proposed changes to Technical Specifications or this Operating License.

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- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meetings minutes of the SNSOC.

## AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Emergency Plan and implementing procedures at least once per 12 months.
- f. The Security Plan and implementing procedures at least once per 12 months.
- g. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.
- h. The Fire Protection Program and implementing procedures at least once per 24 months.

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- i. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- l. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 12 months.
- m. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 12 months.

### AUTHORITY

6.5.2.9 The MSRC shall report to and advise the Senior Vice President - Nuclear on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

### RECORDS

6.5.2.10 Records of MSRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each MSRC meeting shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days of each meeting.
- b. Reports of reviews with safety significant findings encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

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## ADMINISTRATIVE CONTROLS

### 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SNSOC and the results of this review shall be submitted to the Vice President-Nuclear Operations and the MSRC.

### 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President-Nuclear Operations and the MSRC shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSOC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the Vice President-Nuclear Operations and the MSRC within 14 days of the violation.

### 6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.

## ADMINISTRATIVE CONTROLS

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- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each procedure of 6.8.1 above, except 6.8.1.d and 6.8.1.e and changes thereto, shall be reviewed and approved by the SNSOC prior to implementation and reviewed periodically as set forth in administrative procedures. Procedures of 6.8.1.d and 6.8.1.e shall be reviewed and approved as per 6.5.1.6.i and 6.5.1.6.j. SNSOC approval shall be certified in writing by an Assistant Station Manager.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant supervisory staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed and approved by the SNSOC within 14 days of implementation. SNSOC approval shall be certified in writing by an Assistant Station Manager.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the recirculation spray, safety injection, chemical and volume control, gas stripper, and hydrogen recombiners. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 135 AND 118 TO

FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2

DOCKET NOS. 50-338 AND 50-339

INTRODUCTION

By letter dated February 1, 1990, as supplemented July 30, 1990, the Virginia Electric and Power Company (the licensee) proposed changes to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). Specifically, the proposed changes would replace the Independent/Operational Event Review (IOER) Group with a Management Safety Review Committee (MSRC) as the organization responsible for the offsite review function. The MSRC will meet quarterly to review and assess plant activities. In addition, the audits required by the NA-1&2 TS that are now the responsibility of the Quality Assurance (QA) department would be performed under the cognizance of the MSRC. The proposed changes would also bring the NA-1&2 TS in conformance with the Westinghouse (W) Standard TS Section 6.5.

The July 30, 1990 letter provided additional information requested by the staff regarding the constitution of a quorum and the timeliness of the MSRC meeting minutes. The additional information did not alter in any way the staff's initial determination of no significant hazards consideration as noticed in the Federal Register on March 7, 1990 (55 FR 8237).

DISCUSSION

The licensee has established an offsite management review group in addition to the IOER Group that assesses and makes recommendations for the safe overall operation of NA-1&2. The licensee is proposing that the TS review and audit function requirements be performed by the MSRC which would replace the IOER Group. This change would put the licensee's review and audit process in line with Standard Technical Specifications; remove the responsibility for the overall review of station activities from staff personnel; and increase senior company management's involvement in the review and assessment of NA-1&2 activities. The IOER Group is not being dissolved, but would be utilized by the MSRC to perform reviews and assessments of plant activities.

The proposed TS changes will eliminate TS Sections 6.5.2-Independent Operational Event Review Group and 6.5.3 - Quality Assurance Audits in the existing TS. These sections will be replaced with the appropriate Westinghouse (W) Standard TS requirements for the offsite safety review and audit function (i.e., STS Section 6.5.2 - Company Nuclear Review and Audit Group). The following differences exist between the existing NA-1&2 TS and the proposed Standard TS:

- . The offsite review function would now be the responsibility of the MSRC. The majority of the reviews would be performed by a subcommittee of qualified staff specialists and the results reported to the committee.
- . Section 6.5.2.1, Function - The list of areas and activities to be reviewed by the MSRC would be updated to more closely parallel plant and engineering activities.
- . Section 6.5.2.7, Reviews - The existing TS contain a requirement for the IOER staff to review the Quality Assurance audit program once per 12 months. This requirement was incorporated into the existing NA-1&2 TS because the QA Department is responsible for the TS-required audits. This requirement provided the necessary interface for the two groups. These same TS-required audits would now be performed under the cognizance of the MSRC, which will provide the required interface with the group performing the audit.
- . Section 6.5.2.8, Quality Assurance Department - The Quality Assurance Department is currently responsible for the TS-required audits. In the proposed change the MSRC would be required to have the audits performed under their cognizance. The MSRC will have an input to the audit schedule, the program attributes and activities audited. These audits will normally be performed by the Quality Assurance Department and the results reported to MSRC. Three audits in the present NA-1&2 TS are not included in the Standard Technical Specification list of required audits. These three audits: 1) the radiological environmental monitoring program, 2) the offsite dose calculation manual, and 3) the Process Control Program and radwaste procedures, will remain in the present TS to meet previous commitments.
- . Section 6.5.2.10.a, Records - Minutes of each MSRC meeting would be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days of each meeting. Safety significant findings would be reported to the Senior Vice President - Nuclear within 14 days as required by Section 6.5.2.10.b.

To clarify that the Vice-Chairman can act as a voting member when the Chairman is presiding over a Station Nuclear Safety and Operating Committee (SNSOC) meeting, Section 6.5.1.2, Composition of SNSOC, is being modified to include the Vice-Chairman as a member.

In addition to the above, these proposed TS changes delete or modify titles, as necessary, to reflect a recent organizational change. In the reorganization certain responsibilities have changed. The changes are as follows:

- The Senior Vice President - Power title has changed to Senior Vice President - Nuclear. This change is in TS Sections 6.1, 6.2 and 6.5.
- The Vice President - Nuclear title has changed to Vice President - Nuclear Operations. This change is throughout TS Sections 6.2, 6.5, 6.6 and 6.7.
- The Superintendent - Technical Services position had been eliminated. These functions have been charged to the Superintendent of Maintenance and Superintendent of Engineering. Therefore that position is being eliminated from the membership of SNSOC as noted in TS Section 6.5.1.2.
- The Superintendent - Health Physics title has changed to Superintendent - Radiological Protection (TS Section 6.5.1.2).
- Training is now the functional responsibility of the Manager - Nuclear Training and the TS has been changed to reflect this responsibility in Section 6.4.1.

#### EVALUATION

The proposed administrative changes will bring the NA-1&2 TS into conformance with the W Standard TS. In addition, the changes will increase the licensee's senior management's responsibility and involvement in the review and assessment of NA-1&2 activities. Based on the above, we find the proposed changes to be acceptable.

#### ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 7, 1990

Principal Contributor: Leon Engle