March 10, 1987

Docket No. 50-338

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Mr. W. L. Stewart
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Dear Mr. Stewart:

The Commission has issued the enclosed Amendment No. 90 to Facility Operating License No. NPF-4 for the North Anna Power Station, Unit No. 1 (NA-1). The amendment revises the Technical Specifications (TS) in response to your letter dated February 6, 1986. The amendment is effective prior to restart after the forthcoming cycle 6 refueling outage.

This amendment revises the NA-1 TS Table 3.6.1, "Containment Isolation Valves," to reflect the installation of a new containment isolation valve in the letdown line for NA-1.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/

Leon B. Engle, Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 90 to NPF-4
2. Safety Evaluation

cc w/enclosures:
See next page

PA:PAD#2	PM:PAD#2	PD:PAD#2	OGC
DMiller	LEngle	LRubenstein	
2/24/87	2/24/87	3/9/87	2/27/87

Mr. W. L. Stewart
Virginia Electric & Power Company

North Anna Power Station
Units 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated February 6, 1986 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 90 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective prior to restart after the forthcoming cycle 6 refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION

L. S. Rubenstein

Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 10, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 90

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Page
3/4 6-17
3/4 6-27
3/4 6-31
3/4 6-32

TABLE 3.6-1

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	ISOLATION TIME (SEC.)
A. PHASE "A" ISOLATION		
1. MOV-1380	Reactor Coolant Pump Seal Water Return	10
2. MOV-1381	Reactor Coolant Pump Seal Water Return	10
3. Deleted		
4. Deleted		
5. TV-1204A	Reactor Coolant Letdown Line	10
6. TV-1204B	Reactor Coolant Letdown Line	10
7. TV-S 100	Nitrogen to Pressurizer Relief Tank and SI Accumulators	60
8. TV-DG100A	Primary Drains Transfer Tank Pump Discharge	60
9. TV-DG100B	Primary Drains Transfer Tank Pump Discharge	60
10. TV-DA100A	Containment Sump Pump Discharge to Waste Drain Tanks	60
11. TV-DA100B	Containment Sump Pump Discharge to Waste Drain Tanks	60
12. TV-BD100A	Steam Generator Blowdown	60
13. TV-BD100B	Steam Generator Blowdown	60
14. TV-BD100C	Steam Generator Blowdown	60
15. TV-BD100D	Steam Generator Blowdown	60
16. TV-BD100E	Steam Generator Blowdown	60
17. TV-BD100F	Steam Generator Blowdown	60

TABLE 3.6-1 (Cont.)

	VALVE NUMBER	FUNCTION	ISOLATION TIME (SEC.)
18.	TV-RM100A	Air Radiation Monitor Return	60
19.	TV-RM100D	Air Radiation Monitor Return	60
20.	TV-RM100B	Air Radiation Monitor Supply	60
21.	TV-RM100C	Air Radiation Monitor Supply	60
22.	TV-1519A	Primary Grade Water to Pressurizer Relief Tank	10
23.	TV-VG100A	Primary Vent Header	60
24.	TV-VG100B	Primary Vent Header	60
25.	TV-SI101	Safety Injection Accumulators to Waste Gas Charcoal Filters	60
26.	HCV-1936	Safety Injection Accumulators to Waste Gas Charcoal Filters	10
27.	TV-SS104A	Pressurizer Relief Tank Sample	60
28.	TV-SS104B	Pressurizer Relief Tank Sample	60
29.	TV-SS100A	Pressurizer Liquid Space Sample	60
30.	TV-SS100B	Pressurizer Liquid Space Sample	60
31.	TV-SS106A	Primary Coolant Hot Leg Sample	60
32.	TV-SS106B	Primary Coolant Hot Leg Sample	60
33.	TV-SS102A	Primary Coolant Cold Leg Sample	60
34.	TV-SS102B	Primary Coolant Cold Leg Sample	60

TABLE 3.6-1 (Cont.)

	VALVE NUMBER	FUNCTION	ISOLATION TIME (SEC.)
19.	MOV-1836*	High Head Safety Injection to RCS Except Boron Injection Line	NA
20.	MOV-1869B*	High Head Safety Injection to RCS Except Boron Injection Line	NA
21.	Deleted		
22.	Deleted		
23.	Deleted		
24.	MOV-1890A*	LHSI Pump Discharge to Reactor Coolant System Hot Legs	NA
25.	MOV-1890B*	LHSI Pump Discharge to Reactor Coolant System Hot Legs	NA
26.	MOV-1890C*	LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA
27.	MOV-1890D*	LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA
28.	FCV-1160*	Loop Fill Header	NA
29.	MOV-1289A*	Charging Line	NA
30.	MOV-1867C*	High Head Safety Injection, Boron Injection Tank	NA
31.	MOV-1867D*	High Head Safety Injection, Boron Injection Tank	NA
32.	MOV-RS-100A*	Casing Cooling to Outside Recirculation Spray Pump	NA
33.	MOV-RS-100B*	Casing Cooling to Outside Recirculation Spray Pump	NA
34.	MOV-RS-101A*	Casing Cooling to Outside Recirculation Spray Pump	NA

TABLE 3.6-1 (Cont.)

	VALVE NUMBER	FUNCTION	ISOLATION TIME (SEC.)
35.	MOV-RS-101B*	Casing Cooling to Outside Recirculation Spray Pump	NA
36.	TV-HC-108A*	Containment Atmosphere Sample Line	NA
37.	TV-HC-108B*	Containment Atmosphere Sample Line	NA
38.	TV-HC-100A	Suction Hydrogen Analyzer	NA
39.	TV-HC-100B	Suction Hydrogen Analyzer	NA
40.	TV-HC-101A	Discharge Hydrogen Analyzer	NA
41.	TV-HC-101B	Discharge Hydrogen Analyzer	NA
42.	TV-HC-102A	Suction Hydrogen Analyzer	NA
43.	TV-HC-102B	Suction Hydrogen Analyzer	NA
44.	TV-HC-103A	Discharge Hydrogen Analyzer	NA
45.	TV-HC-103B	Discharge Hydrogen Analyzer	NA
46.	TV-HC-104A	Suction Hydrogen Recombiner	NA
47.	TV-HC-104B	Suction Hydrogen Recombiner	NA
48.	TV-HC-105A	Discharge Hydrogen Recombiner	NA
49.	TV-HC-105B	Discharge Hydrogen Recombiner	NA
50.	TV-HC-106A	Suction Hydrogen Recombiner	NA
51.	TV-HC-106B	Suction Hydrogen Recombiner	NA
52.	TV-HC-107A	Discharge Hydrogen Recombiner	NA
53.	TV-HC-107B	Discharge Hydrogen Recombiner	NA

TABLE 3.6-1 (Cont.)

VALVE NUMBER	FUNCTION	ISOLATION TIME (SEC.)
G. STEAM LINE ISOLATION		
1. TV-MS-101A#	Main Steam Line Trip Valve	5
2. TV-MS-101B#	Main Steam Line Trip Valve	5
3. TV-MS-101C#	Main Steam Line Trip Valve	5
H. RELIEF		
None		

Valve not subject to Type "C" leakage test

* Valve position maintained by administrative control

NA - Not applicable

** - Requires testing per Technical Specifications 4.6.3.1.1a. or 4.6.3.1.2d.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. NPF-4

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

NORTH ANNA POWER STATION, UNIT NO. 1

DOCKET NO. 50-338

INTRODUCTION

By letter dated February 6, 1986, the Virginia Electric and Power Company (the licensee) proposed changes to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). Specifically, the changes pertain to Section 3/4.6.3, Table 3.6-1, "Containment Isolation Valves," for the NA-1&2 TS. The changes reflect a design modification for the installation of a new containment isolation valve in the letdown line of NA-1&2.

On April 4, 1986, Amendment No. 63 was issued for NA-2 which addressed the proposed changes as requested in the licensee's above noted letter dated February 6, 1986. The safety evaluation supporting Amendment No. 63 addressed the proposed changes for both NA-1&2. It was stated in the safety evaluation that an amendment for NA-1 would not be issued until the new NA-1 containment isolation valve was installed in the forthcoming cycle 6 refueling outage. With the passage of time, this appropriate NA-1 refueling outage is now imminent. The discussion and evaluation provided below are identical to the previously issued safety evaluation supporting Amendment No. 63 for NA-2. However, references to NA-2 in the previous safety evaluation have been deleted below.

DISCUSSION:

The existing three parallel valves (HCV-1200A, HCV-1200B and HCV-1200C for NA-1) in the letdown line were designated as containment isolation valves inside containment. These valves are located at an elevation that is below the maximum flood level in the containment and, therefore, do not meet the guidelines of Regulatory Guide 1.97 concerning environmental qualification. Moreover, these valves have experienced leakage problems relative to meeting the leak testing requirements of Appendix J to 10 CFR Part 50.

The licensee's proposed resolution is to add a new valve in the letdown line for NA-1 that will serve as the containment isolation valve inside containment. The new valve will be located above the maximum flood level and will meet the design criteria of containment isolation valves. Also, this single valve will provide greater assurance of leakage integrity of the line. The existing outboard containment isolation valve will remain unchanged.

The staff finds this design modification acceptable and to be in compliance with Standard Review Plan Sections 6.2.4, "Containment Isolation System," and 6.2.6, "Containment Leakage Testing."

In conjunction with the above design modification, the licensee has proposed to change the TS Table 3.6-1, "Containment Isolation Valves," to reflect the change. The containment isolation function of the valve (TV 1204A for NA-1) negates this function previously assigned to the following valves in the letdown line: the three parallel valves identified above, a relief valve (RV-1203 for NA-1), and a manual valve (HCV-1142 for NA-1). The relief valve discharges to the pressurizer relief tank, and the manual valve is in a line which connects the letdown line to the Reactor Heat Removal heat exchanger. The existing valves will not be physically removed but will no longer provide a containment isolation function.

EVALUATION

Based on the above, the staff finds the proposed changes to appropriately meet the provisions for the containment isolation letdown line and, therefore, finds the change to be acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 10, 1987

Principal Contributors:

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L. Engle