

March 31, 1987

Docket Nos. 50-338
and 50-339

Mr. W. L. Stewart
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Dear Mr. Stewart:

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The Commission has issued the enclosed Amendment Nos. 92 and 77 to Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The amendments revise the Technical Specifications (TS) in response to your letter dated November 25, 1986. The amendments are effective within 14 days from the date of issuance.

The amendments modify Technical Specifications 3/4.12 (Radiological Monitoring) to reflect established practices, to agree with NRC approved documents, and to conform to NRC guidance.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/

Leon B. Engle, Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Enclosure:

1. Amendment No. 92 to NPF-4
2. Amendment No. 77 to NPF-7
3. Safety Evaluation

cc w/enclosures:
See next page

for
LA:PAD#2
DMiller
2/20/87

BE
PM:PAD#2
LEngle:hc
2/20/87

OGC
OGC:PAD#2
Mifoun
2/27/87

JSN
PD:PAD#2
LRubenstein
3/27/87

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PDR ADDCK 05000338
P PDR

Mr. W. L. Stewart
Virginia Electric & Power Company

North Anna Power Station
Units 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 92
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated November 25, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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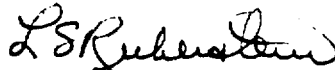
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 92, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 14 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 31, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 92

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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TABLE 4.12-1

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM*

<u>Exposure Pathway and/or Sample</u>	<u>Number of Samples and Sample Locations^a</u>	<u>Sampling and Collection Frequency</u>	<u>Type and Frequency of Analysis</u>
1. DIRECT RADIATION ^b	36 routine monitoring stations either with two or more dosimeters or with one instrument for measuring and recording dose rate continuously to be placed as follows: 1) an inner ring of stations, one in each meteorological sector within the SITE BOUNDARY; an outer ring of stations, one in each meteorological sector within 8 km range from the site; the balance of the stations to be placed in special interest areas such as population centers, nearby residences, schools, and in 1 or 2 areas to serve as control stations.	Quarterly	Gamma dose quarterly.
2. AIRBORNE Radioiodine and Particulates	Samples from 5 locations: a. 3 samples from close to the 3 SITE BOUNDARY locations (in different sectors) of the highest calculated historical annual average ground-level D/Q.	Continuous sampler (2/3 running time cycle), operation with sample collection weekly.	Radioiodine Cannister: I-131 analysis weekly. Particulate Sampler: Gross beta radioactivity analysis following filter change; ^c Gamma isotopic analysis of composite (by

*The number, media, frequency, and location of samples may vary from site to site. This table presents an acceptable minimum program for a site at which each entry is applicable. Local site characteristics must be examined to determine if pathways not covered by this table may significantly contribute to an individual's dose and should be included in the sampling program.

TABLE 4.12-1 (Continued)

RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

<u>Exposure Pathway and/or Sample</u>	<u>Number of Samples and Sample Locations^a</u>	<u>Sampling and Collection Frequency</u>	<u>Type and Frequency of Analysis</u>
	b. 1 sample from the vicinity of a community having the highest calculated annual average groundlevel D/Q.		location) quarterly. ^d
	c. 1 sample from a control location 15-40 km distant and in the least prevalent wind direction.		
3. WATERBORNE			
a. Surface	a. 1 sample circulating water discharge	Sample off upstream, downstream and cooling lagoon. Grab Monthly.	Gamma isotopic analysis ^d monthly. Composite for tritium analysis quarterly.
b. Ground	Samples from 1 or 2 sources only if likely to be affected.	Grab Quarterly	Gamma isotopic ^d and tritium analysis quarterly.
c. Sediment	1 sample from downstream area with existing or potential recreational value.	Semiannually	Gamma isotopic analysis ^d semiannually.

TABLE 4.12-2

REPORTING LEVELS FOR RADIOACTIVITY CONCENTRATIONS IN ENVIRONMENTAL SAMPLES

Reporting Levels					
Analysis	Water (pCi/l)	Airborne Particulate or Gases (pCi/m ³)	Fish (pCi/kg, wet)	Milk (pCi/l)	Food Products (pCi/kg, wet)
H-3	20,000*				
Mn-54	1,000		30,000		
Fe-59	400		10,000		
Co-58	1,000		30,000		
Co-60	300		10,000		
Zn-65	300		20,000		
Zr-Nb-95	400				
I-131	2	0.9		3	100
Cs-134	30	10	1,000	60	1,000
Cs-137	50	20	2,000	70	2,000
Ba-La-140	200			300	

* For drinking water samples

TABLE 4.12-3
DETECTION CAPABILITIES FOR ENVIRONMENTAL SAMPLE ANALYSIS^a

LOWER LIMIT OF DETECTION (LLD)^b

Analysis	Water (pCi/l)	Airborne Particulate or Gas (pCi/m ³)	Fish (pCi/kg,wet)	Milk (pCi/l)	Food Products (pCi/kg,wet)	Sediment (pCi/kg,dry)
gross beta	4	0.01				
H-3	2000		130			
Mn-54	15		260			
Fe-59	30		130			
Co-58,60	15		260			
Zn-65	30					
Zr-Nb-95	15			1	60	
I-131	1.0 ^c	0.07		15	60	150
Cs-134	15	0.05	130	18	80	180
Cs-137	18	0.06	150	15		
Ba-La-140	15					

TABLE 4.12-3 (Continued)

TABLE NOTATION

^aThis list does not mean that only these nuclides are to be considered. Other peaks that are identifiable, together with those of the above nuclides, shall also be analyzed and reported in the Annual Radiological Environmental Operating Report pursuant to Specification 6.9.1.8.

^bThe LLD is defined, for purposes of these Specifications, as the smallest concentration of radioactive material in a sample that will yield a net count, above system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system, which may include radiochemical separation):

$$LLD = \frac{4.66 s_b}{E \cdot V \cdot 2.22 \cdot Y \cdot \exp(-\lambda \Delta t)}$$

Where:

LLD is the "a priori" lower limit of detection as defined above, as picocuries per unit mass or volume,

s_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate, as counts per minute,

E is the counting efficiency, as counts per disintegration,

V is the sample size in units of mass or volume,

2.22 is the number of disintegrations per minute per picocurie,

Y is the fractional radiochemical yield, when applicable,

λ is the radioactive decay constant for the particular radionuclide, and

Δt for environmental samples is the elapsed time between sample collection, or end of the sample collection period, and time of counting

Typical values of E, V, Y and Δt should be used in the calculation.

TABLE 4.12-2 (Continued)

TABLE NOTATION

It should be recognized that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not as an a posteriori (after the fact) limit for a particular measurement. Analysis shall be performed in such a manner that the stated LLDs will be achieved under routine conditions. Occasionally background fluctuations, unavoidably small sample sizes, the presence of interfering nuclides, or other uncontrollable circumstances may render these LLDs unachievable. In such cases, the contributing factors will be identified and described in the Annual Radiological Environmental Operating Report pursuant to Specification 6.9.1.8.

^cThis LLD value is for drinking water samples.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated November 25, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

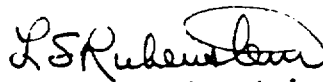
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 77, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 14 days from the the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 31, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 77

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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TABLE 4.12-1 (Continued)
RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

<u>Exposure Pathway and/or Sample</u>	<u>Number of Samples and Sample Locations^a</u>	<u>Sampling and Collection Frequency</u>	<u>Type and Frequency of Analysis</u> location) quarterly. ^d
	b. 1 sample from the vicinity of a community having the highest calculated annual average groundlevel D/Q.		
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a. Surface	a. 1 sample circulating water discharge	Sample off upstream, downstream and cooling lagoon. Grab Monthly.	Gamma isotopic analysis ^d monthly. Composite for tritium analysis quarterly.
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TABLE 4.12-3 (Continued)

TABLE NOTATION

^aThis list does not mean that only these nuclides are to be considered. Other peaks that are identifiable, together with those of the above nuclides, shall also be analyzed and reported in the Annual Radiological Environmental Operating Report pursuant to Specification 6.9.1.8.

^bThe LLD is defined, for purposes of these Specifications, as the smallest concentration of radioactive material in a sample that will yield a net count, above system background, that will be detected with 95% probability with only 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system, which may include radiochemical separation :

$$LLD = \frac{4.66 s_b}{\bar{E} \cdot V \cdot 2.22 \cdot Y \cdot \exp(-\lambda \Delta t)}$$

Where:

LLD is the "a priori" lower limit of detection as defined above, as picocuries per unit mass or volume,

s_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate, as counts per minute,

\bar{E} is the counting efficiency, as counts per disintegration,

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Y is the fractional radiochemical yield, when applicable,

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Typical values of E , V , Y and Δt should be used in the calculation.

TABLE 4.12-2 (Continued)

TABLE NOTATION

It should be recognized that the LLD is defined as an a priori (before the fact) limit representing the capability of a measurement system and not as an a posteriori (after the fact) limit for a particular measurement. Analysis shall be performed in such a manner that the stated LLDs will be achieved under routine conditions. Occasionally background fluctuations, unavoidably small sample sizes, the presence of interfering nuclides, or other uncontrollable circumstances may render these LLDs unachievable. In such cases, the contributing factors will be identified and described in the Annual Radiological Environmental Operating Report pursuant to Specification 6.9.1.8.

^cThis LLD value is for drinking water samples



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 92 AND 77 TO
FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7
VIRGINIA ELECTRIC AND POWER COMPANY
OLD DOMINION ELECTRIC COOPERATIVE
NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2
DOCKET NOS. 50-338 AND 50-339

INTRODUCTION

By letter dated November 25, 1986 (Serial No. 86-723), the Virginia Electric and Power Company (the licensee) proposed a change to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). Specifically, the proposed change would modify TS 3/4.12 (Radiological Monitoring) to reflect established practices, to agree with NRC approved documents, and to conform to NRC regulatory guidance.

DISCUSSION

The proposed changes to the NA-1&2 TS will correct an administrative error in Table 4.12-1, paragraph 2.c. This table lists the sample point for offsite monitoring to be 15 to 30 kilometers from the site. This specification came in existence when the radiological portion of the Environmental TS were incorporated into the plant TS (Amendment 48 to Unit 1 and Amendment 31 to Unit 2).

The original Environmental TS listed eight locations for continuous airborne sampling. There was 1 at the site, 3 on the site perimeter, 3 in nearby communities and 1 at the Orange District Office. These locations are also specified in the Offsite Dose Calculation Manual (ODCM) in accordance with TS requirements.

The ODCM lists site 24, the "Control" monitor at the Orange District Office, as being "22 miles" (35.4 km) from the site. This monitoring site was placed in service in December 1976 as a control location for the North Anna Station and has provided the station with a well-established 10 year base of sampling data. The location is secure from vandalism and assures a stable power source.

In order to correct this discrepancy, TS Table 4.12-1 paragraph 2.c will be changed to read "1 sample from a control location 15-40 kilometers distant and in the least prevalent wind direction." The change will also bring the TS into agreement with the ODCM which has been approved by the NRC.

Table 4.12-2 of the NA-1&2 TS lists a reporting level of 30,000 pCi/liter for H_3 . This is to be changed to 20,000 pCi/liter in accordance with NUREG-0472, Radiological Effluent Technical Specifications for PWRs, Revision 3. In addition there is a footnote being added which specifies the sample to be tested. Table 4.12-3 of the NA-1&2 TS 3/4.12.1 lists a Lower Limit of Detectability (LLD) for High Resolution Ge(Li) Gamma Spectroscopy Environmental sample of 10 pCi/liter for I_{131} and an LLD of 3000 pCi/liter for H_3 . These are to be changed to 1 pCi/liter for I_{131} and 2000 for H_3 in accordance with NUREG-0472, Radiological Effluent Technical Specifications for PWRs, Revision 3. Finally, a footnote is being deleted from Table 4.12-3 which specifies that "The LLD for Gamma isotopic analysis shall be used." A footnote being added to Table 4.12-2 and 4.12-3 will state that "The LLD value is for drinking water samples."

EVALUATION

The proposed change to NA-1&2 TS Table 4.12.1 corrects a discrepancy presently existing in the TS and brings the NA-1&2 TS into agreement with the NRC approved Offsite Dose Computational Manual. The proposed changes to the NA-1&2 TS Tables 4.12-2 and 4.12-3 impose a more restrictive requirement which is in conformance with NUREG-0472, Radiological Effluent Technical Specifications for PWRs, Revision 3. Based on the above, we find the proposed changes to be acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 31, 1987

Principal Contributor:

L. B. Engle