UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
December 29, 1988

Docket Nos. 50-338
and 50-339

Mr. W. R. Cartwright
Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. Cartwright:

SUBJECT: NORTH ANNA UNITS 1 AND 2 - CORRECTION TO AMENDMENT NOS. 109 AND 95 (TAC NOS. 67602 AND 67603) AND AMENDMENT NOS. 110 AND 96 (TAC NOS. 67535 AND 67536)

On December 12, 1988, the Commission issued Amendment Nos. 109 and 95 for the North Anna Power Station, Units 1 and 2 (NA-1&2). The amendments implemented more stringent primary-to-secondary coolant systems leakage limits and established surveillance requirements to assure operability of the existing and new N-16 instrumentation necessary to assure compliance with the revised leakage limits.

Also, on December 14, 1988, The Commission issued Amendment Nos. 110 and 96 to the NA-1&2 Technical Specifications (TS) which revised the containment air temperature upper limit from 105°F to 120°F. In addition, the volume of water available from the refueling water storage tank was redefined and reduced to permit the use of wide range level instrumentation for TS surveillance.

On December 19, 1988, you informed us of administrative errors in both of these amendments. In Amendment Nos. 109 and 95, the reference for the footnote in Table 3.3-14 should not have been changed from an asterisk to "a". Enclosure 1 contains the corrected pages 3/4 3-67 (for Unit 1) and 3/4 3-62 (for Unit 2), as well as the corresponding overleaf page, to be inserted into the NA-1&2 TS.

DF01

An Amendment Nos. 110 and 96, a footnote had been added to NA-1 TS page 3/4 6-2 which stated that "For Specification 3/4.6.1.2 only, Pa shall be 40.6 psig until completion of the Cycle 6 to 7 refueling outage...." However, the correct cycle for NA-1 is the Cycle 7 to 8 refueling outage. Enclosure 2 contains the correct page 3/4 6-2 for NA-1, as well as the corresponding overleaf page to be inserted into the NA-1 TS.

The staff has determined that the correction of these errors does not change the staff's evaluations and conclusions which supported the changes for both amendments to the NA-1&2 Technical Specifications.

Sincerely,

ORIGINAL SIGNED BY
Leon B. Engle, Project Manager
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Enclosures:
As stated

cc w/enclosures:
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GPA/PA
ARM/LFMB
Others as required

[CORRECTIONS TO AMEND/NA-1&2]

LA:PDII-2	PM:PDII-2	D:PDII-2
DMiller	LEngle:jd	HBerkow
12/29/88	12/29/88	12/29/88

Mr. W. R. Cartwright
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North Anna Power Station
Units 1 and 2

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TABLE 3.3-14

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
1. PROCESS VENT SYSTEM			
a. Noble Gas Activity Monitor - Providing Alarm and Automatic Termination of Release	1	*	31,33
b. Iodine Sampler	1	*	31,34
c. Particulate Sampler	1	*	31,34
d. Process Vent Flow Rate Measuring Device	1	*	30
e. Sampler Flow Rate Measuring Device	1	*	30
2. WASTE GAS HOLDUP SYSTEM EXPLOSIVE GAS MONITORING SYSTEM (Shared with Unit 2)			
a. Hydrogen Monitor	1	**	32
b. Oxygen Monitor	1	**	32

TABLE 3.3-14 (Continued)

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
3.	CONDENSER AIR EJECTOR SYSTEM			
	a. Gross Activity Monitor	1	*	31A
	b. Flow Rate Monitor	1	*	30
4.	VENTILATION VENT SYSTEM (Shared with Unit 2)			
	a. Noble Gas Activity Monitor	1a	*	31
	b. Iodine Sampler	1a	*	31
	c. Particulate Sampler	1a	*	31
	d. Flow Rate Monitor	1a	*	30
	e. Sampler Flow Rate Monitor	1a	*	30

*One per vent stack.

TABLE 3.3-13

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
1.	PROCESS VENT SYSTEM			
	a. Noble Gas Activity Monitor - Providing Alarm and Automatic Termination of Release	1	*	31,33
	b. Iodine Sampler	1	*	31,34
	c. Particulate Sampler	1	*	31,34
	d. Process Vent Flow Rate Measuring Device	1	*	30
	e. Sampler Flow Rate Measuring Device	1	*	30
2.	WASTE GAS HOLDUP SYSTEM EXPLOSIVE GAS MONITORING SYSTEM (Shared with Unit 1)			
	a. Hydrogen Monitor	1	**	32
	b. Oxygen Monitor	1	**	32

TABLE 3.3-13 (Continued)

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
3. CONDENSER AIR EJECTOR SYSTEM			
a. Gross Activity Monitor	1	*	31A
b. Flow Rate Monitor	1	*	30
4. VENTILATION VENT SYSTEM (Shared with Unit 1)			
a. Noble Gas Activity Monitor	1a	*	31
b. Iodine Sampler	1a	*	31
c. Particulate Sampler	1a	*	31
d. Flow Rate Monitor	1a	*	30
e. Sampler Flow Rate Monitor	1a	*	30

*One per vent stack.

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

a. At least once per 31 days by verifying that:

1. All penetrations* not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.3.1., and

2. All equipment hatches are closed and sealed

b. By verifying that each containment air lock is OPERABLE per Specification 3.6.1.3.

*Except valves, blind flanges and deactiviated automatic valves which are located inside the containment and are locked sealed or otherwise sealed in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days.

CONTAINMENT SYSTEMS

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:**

a. An overall integrated leakage rate of:

1. ≤ L_a, 0.1 percent by weight of the containment air per 24 hours at P_a, ≥ 44.1 psig, or

b. A combined leakage rate of ≤ 0.60 L_a for all penetrations and valves subject to Type B and C tests, when pressurized to P_a, ≥ 44.1 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding 0.75 L_a or (b) with the measured combined leakage rate for all penetrations and valves subject to Type B and C tests exceeding 0.60 L_a, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of either ANSI N45.4-1972 for leakage rate point data analysis or ANSI/ANS-56.8-1987 for mass point data analysis with a minimum test duration of 24 hours.**

a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at $P_a \geq 44.1$ psig during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.*

*The third test of the first 10-year service period shall be conducted during the 1989 Refueling/10-Year ISI Outage.

**For Specification 3/4.6.1.2 only, P_a shall be 40.6 psig until completion of the Cycle 7 to 8 refueling outage. Following this outage, P_a shall be 44.1 psig.