

November 10, 1986

Docket Nos. 50-338  
and 50-339

Mr. W. L. Stewart  
Vice President - Nuclear Operations  
Virginia Electric and Power Company  
Post Office Box 26666  
Richmond, Virginia 23261

Dear Mr. Stewart:

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The Commission has issued the enclosed Amendment Nos. 87 and 73 to Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The amendments revise the Technical Specifications (TS) in response to your letter dated August 29, 1986. The amendments are effective as of the date of issuance.

The amendments revise the Technical Specifications (TS) Section 6, Administrative Controls, to reflect recent corporate and station title changes and realign the Quality Control Program from operations to construction in order to enhance the independence of the Quality Assurance Program.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/

Leon B. Engle, Project Manager  
PWR Project Directorate #2  
Division of PWR Licensing-A  
Office of Nuclear Reactor Regulation

Enclosure:

1. Amendment No. 87 to NPF-4
2. Amendment No. 73 to NPF-7
3. Safety Evaluation

cc w/enclosures:  
See next page

LA:PAD#2  
DM:11/11/86  
10/28/86

PM:PAD#2  
LEngle:ab  
10/29/86

OGC  
10/20/86

PD:PAD#2  
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10/29/86

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Mr. W. L. Stewart  
Virginia Electric & Power Company

North Anna Power Station  
Units 1 and 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 87  
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated August 29, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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P PDR

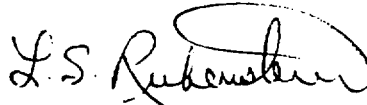
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 87, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director  
PWR Project Directorate #2  
Division of PWR Licensing-A  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 10, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 87

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Page

6-1a

6-2

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## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The Station Manager shall be responsible for overall facility operation. In his absence, the Assistant Station Manager (Operations and Maintenance) shall be responsible for overall facility operation. During the absence of both, the Station Manager shall delegate in writing the succession to this responsibility.

6.1.2 The Shift Supervisor (or during his absence from the Control Room, a designated individual) shall be responsible for the Control Room command function and shall be the only individual that may direct the licensed activities of licensed operators. A management directive to this effect, signed by the Senior Vice President - Power Operations, shall be reissued to all station personnel on an annual basis.

### 6.2 ORGANIZATION

#### OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown in Figure 6.2-1.

#### FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODES 1, 2, 3 or 4, at least one licensed Senior Reactor Operator shall be in the Control Room.
- c. A health physics technician# shall be onsite when fuel is in the reactor.
- d. ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- e. A Fire Brigade of at least 5 members shall be maintained onsite at all times#. The Fire Brigade shall not include the minimum shift crew shown in Table 6.2-1 or any personnel required for other essential functions during a fire emergency.

#The health physics technician and Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

## ADMINISTRATIVE CONTROLS

### 6.2.3 NUCLEAR SAFETY ENGINEERING (NSE)

#### FUNCTION

6.2.3.1 NSE shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources which may indicate areas for improving plant safety.

#### COMPOSITION

6.2.3.2 NSE shall be composed of at least five dedicated, full-time engineers located onsite.

#### RESPONSIBILITIES

6.2.3.3 NSE shall be responsible for maintaining surveillance of plant activities to provide independent verification\* that these activities are performed correctly and that human errors are reduced as much as practical.

6.2.3.4 NSE shall disseminate relevant operational experience.

#### AUTHORITY

6.2.3.5 The NSE shall make detailed recommendations for revised procedures, equipment modifications, or other means of improving plant safety to the Assistant Station Manager (Nuclear Safety and Licensing).

### 6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall serve in an advisory capacity to Shift Supervisor on matters pertaining to the engineering aspects of assuring safe operation of the unit.

\*Not responsible for sign-off function

Figure 6.2-1 Offsite Organization for Facility Management and Technical Support

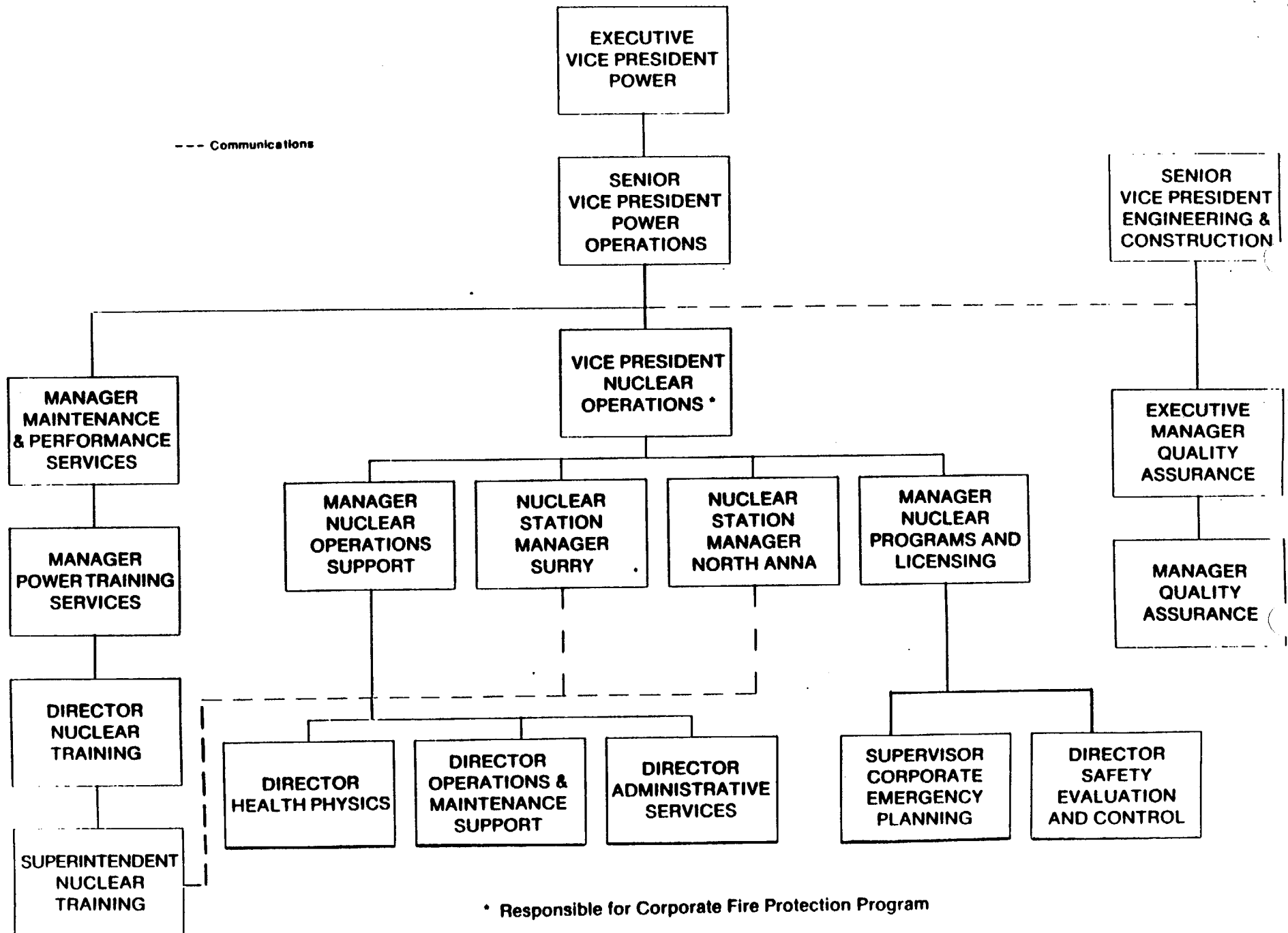




Figure 6.2-2 Facility Organization — North Anna — Units 1 and 2

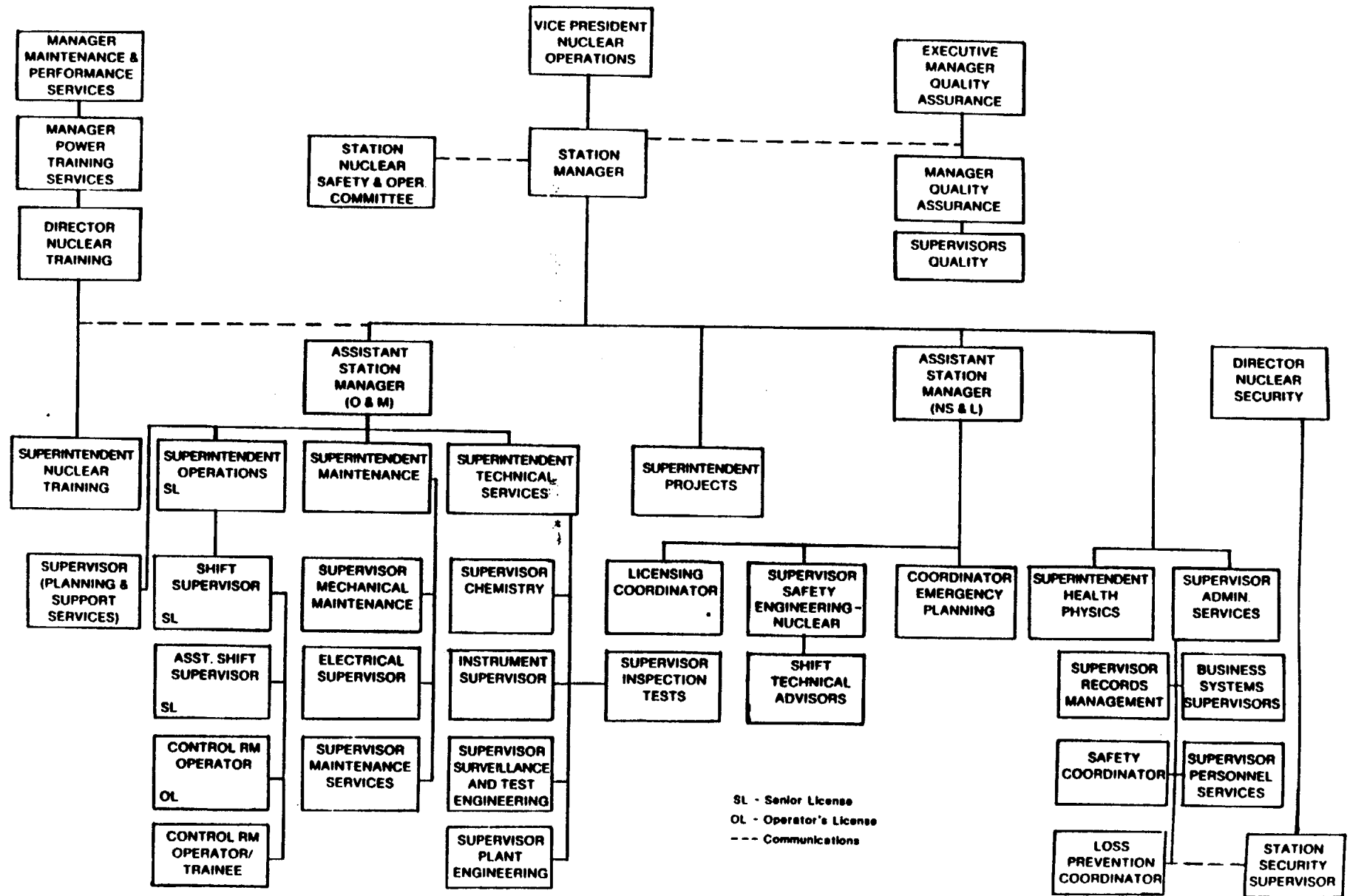


TABLE 6.2-1  
MINIMUM SHIFT CREW COMPOSITION

WITH UNIT 2 IN MODE 5 OR 6 OR DE-FUELED		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3, & 4	MODES 5 & 6
SS	1 <sup>a</sup>	1 <sup>a</sup>
SRO	1	none
RO	2	1
AO	2	2 <sup>b</sup>
STA	1	none

WITH UNIT 2 IN MODES 1, 2, 3, OR 4		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3, & 4	MODES 5 & 6
SS	1 <sup>a</sup>	1 <sup>a</sup>
SRO	1 <sup>a</sup>	none
RO	2 <sup>b</sup>	1
AO	2 <sup>b</sup>	1
STA	1 <sup>a</sup>	none

a/ Individual may fill the same position of Unit 2.

b/ One of the two required individuals may fill the same position on Unit 2.

## ADMINISTRATIVE CONTRLS

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANS 3.1-(12/79 Draft) for comparable positions and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, except for (1) the Superintendent - Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

### 6.4 TRAINING

6.4.1 The Station Manager is responsible for ensuring that retraining and replacement training programs for the facility staff are maintained and that such programs meet or exceed the requirements and recommendations of Section 5 of ANS 3.1-(12/79 Draft) and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the NSE.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 STATION NUCLEAR SAFETY AND OPERATING COMMITTEE (SNSOC)

##### FUNCTION

6.5.1.1 The SNSOC shall function to advise the Station Manager on all matters related to nuclear safety.

##### COMPOSITION

6.5.1.2 The SNSOC shall be composed of the :

Chairman:	Assistant Station Manager (Nuclear Safety and Licensing)
Vice Chairman:	Assistant Station Manager (Operations and Maintenance)
Member:	Superintendent-Operations
Member:	Superintendent-Maintenance
Member:	Superintendent-Technical Services
Member:	Superintendent-Health Physics

##### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SNSOC Chairman to serve on a temporary basis; however, no more than one alternate shall participate as a voting member in SNSOC activities at any one time.

## ADMINISTRATIVE CONTROLS

### MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

### QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

### RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

- a. Review of 1) all procedures required by Specifications 6.8.1, 6.8.2 and 6.8.3 and changes thereto, 2) all programs required by Specification 6.8.4 and changes thereto, 3) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Station Manager.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President-Nuclear Operations and the Director-Safety Evaluation and Control.
- f. Review of all REPORTABLE EVENTS and Special Reports.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Station Manager.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Station Manager.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Station Manager.

## ADMINISTRATIVE CONTROLS

- m. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.

### AUTHORITY

6.5.3.2 The Quality Assurance Department shall report to and advise the Executive Manager - Quality Assurance, who shall advise the Senior Vice President - Engineering and Construction on those areas of responsibility specified in Section 6.5.3.1.

### RECORDS

6.5.3.3 Records of the Quality Assurance Department audits shall be prepared and maintained in the department files. Audit reports shall be disseminated as indicated below:

1. Vice President-Nuclear Operations
2. Nuclear Power Station Manager
3. Manager-Nuclear Operations Support
4. Manager-Nuclear Programs and Licensing
5. Executive Manager-Quality Assurance
6. Director-Safety Evaluation and Control
7. Nuclear Power Station Manager Quality Assurance
8. Supervisor(s) of Area Audited

## ADMINISTRATIVE CONTROLS

### 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SNSOC and the results of this review shall be submitted to the Director-Safety Evaluation and Control and the Vice President-Nuclear Operations.

### 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President-Nuclear Operations, and the Director-Safety Evaluation and Control shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSOC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the Director-Safety Evaluation and Control and the Vice President-Nuclear Operations within 14 days of the violation.

### 6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73  
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated August 29, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director  
PWR Project Directorate #2  
Division of PWR Licensing-A  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 10, 1986



ATTACHMENT TO LICENSE AMENDMENT NO. 73  
TO FACILITY OPERATING LICENSE NO. NPF-7  
DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Page  
6-1a  
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## ADMINISTRATIVE CONTROLS

### 6.2.3 NUCLEAR SAFETY ENGINEERING (NSE)

#### FUNCTION

6.2.3.1 NSE shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources which may indicate areas for improving plant safety.

#### COMPOSITION

6.2.3.2 NSE shall be composed of at least five dedicated, full-time engineers located onsite.

#### RESPONSIBILITIES

6.2.3.3 NSE shall be responsible for maintaining surveillance of plant activities to provide independent verification\* that these activities are performed correctly and that human errors are reduced as much as practical.

6.2.3.4 NSE shall disseminate relevant operational experience.

#### AUTHORITY

6.2.3.5 NSE shall make detailed recommendations for revised procedures, equipment modifications, or other means of improving plant safety to the Assistant Station Manager (Nuclear Safety and Licensing).

### 6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall serve in an advisory capacity to Shift Supervisor on matters pertaining to the engineering aspects of assuring safe operation of the unit.

\*Not responsible for sign-off function.

Figure 6.2-1 Offsite Organization for Facility Management and Technical Support

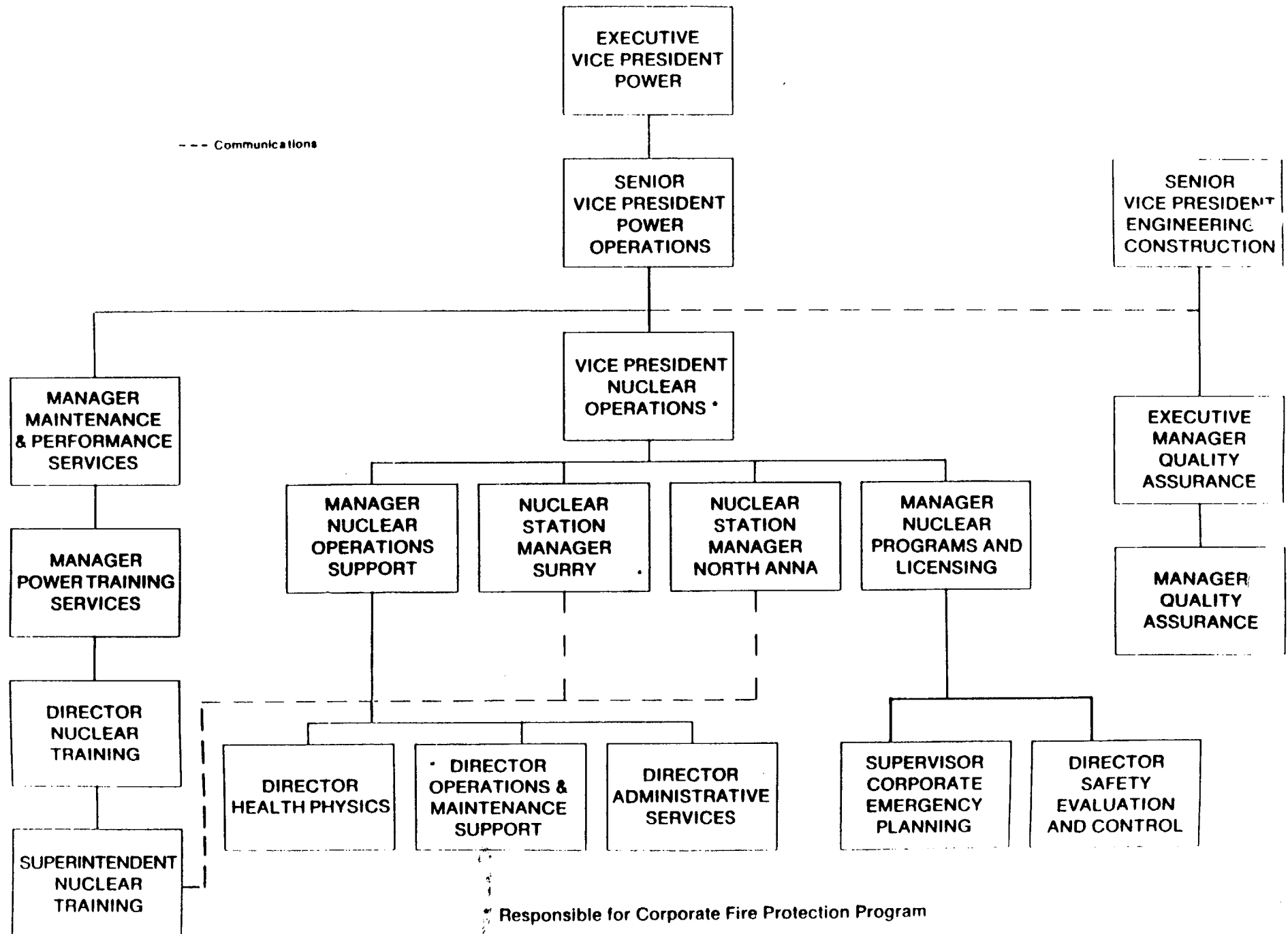


Figure 6.2-2 Facility Organization — North Anna — Units 1 and 2

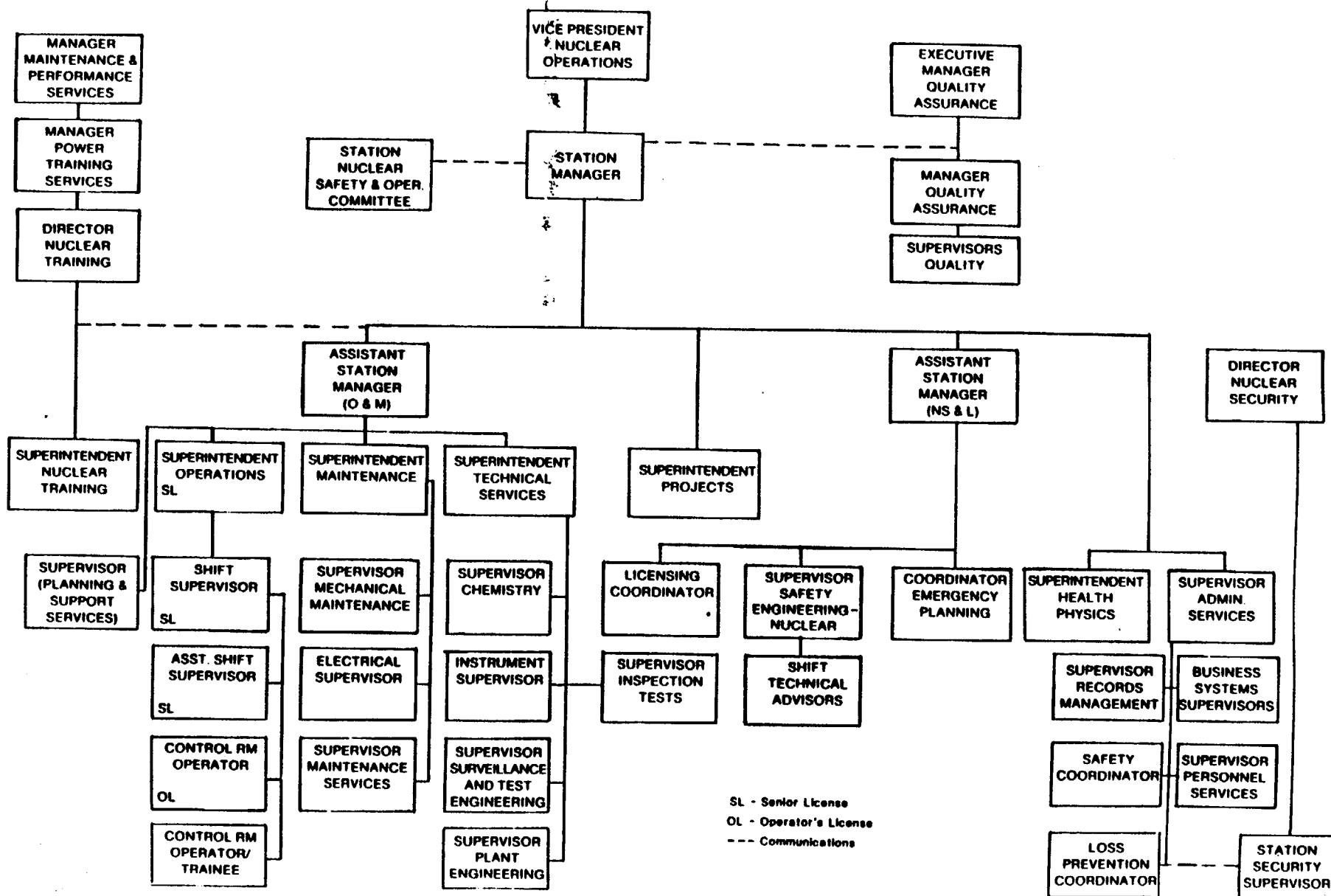


TABLE 6.2-1  
MINIMUM SHIFT CREW COMPOSITION

WITH UNIT 1 IN MODE 5 OR 6 OR DE-FUELED		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3, & 4	MODES 5 & 6
SS	1 <sup>a</sup>	1 <sup>a</sup>
SRO	1	none
RO	2	1
AO	2	2 <sup>b</sup>
STA	1	none

WITH UNIT 1 IN MODES 1, 2, 3, OR 4		
POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3, & 4	MODES 5 & 6
SS	1 <sup>a</sup>	1 <sup>a</sup>
SRO	1 <sup>a</sup>	none
RO	2 <sup>b</sup>	1
AO	2 <sup>b</sup>	1
STA	1 <sup>a</sup>	none

a/ Individual may fill the same position of Unit 1.

b/ One of the two required individuals may fill the same position on Unit 1.

TABLE 6.2-1 (Continued)

SS - Shift Supervisor with a Senior Reactor Operators License on Unit 2.  
SRO - Individual with a Senior Reactor Operators License on Unit 2.  
RO - Individual with a Reactor Operators License on Unit 2.  
AO - Auxiliary Operator  
STA - Shift Technical Advisor

Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 1, 2, 3 or 4, an individual (other than the Shift Technical Advisor) with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 5 or 6, an individual with a valid RO license (other than the Shift Technical Advisor) shall be designated to assume the Control Room command function.

Procedures will be established to insure that NRC policy statement guidelines regarding working hours established for employees are followed. In addition, procedures will provide for documentation of authorized deviations from these guidelines and that the documentation is available for NRC review.

## ADMINISTRATIVE CONTRLS

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANS 3.1-(12/79 Draft) for comparable positions and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, except for (1) the Superintendent - Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

### 6.4 TRAINING

6.4.1 The Station Manager is responsible for ensuring that retraining and replacement training programs for the facility staff are maintained and that such programs meet or exceed the requirements and recommendations of Section 5 of ANS 3.1-(12/79 Draft) and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the NSE.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 STATION NUCLEAR SAFETY AND OPERATING COMMITTEE (SNSOC)

##### FUNCTION

6.5.1.1 The SNSOC shall function to advise the Station Manager on all matters related to nuclear safety.

##### COMPOSITION

6.5.1.2 The SNSOC shall be composed of the :

Chairman: Assistant Station Manager (Nuclear Safety and Licensing)  
Vice Chairman: Assistant Station Manager (Operations and Maintenance)  
Member: Superintendent-Operations  
Member: Superintendent-Maintenance  
Member: Superintendent-Technical Services  
Member: Superintendent-Health Physics

##### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SNSOC Chairman to serve on a temporary basis; however, no more than one alternate shall participate as a voting member in SNSOC activities at any one time.

## ADMINISTRATIVE CONTROLS

### RECORDS (Cont'd)

4. Manager-Nuclear Programs and Licensing
5. Executive Manager-Quality Assurance
6. Others that the Director-Safety Evaluation and Control may designate.

### 6.5.3 QUALITY ASSURANCE DEPARTMENT

#### FUNCTION

6.5.3.1 The Quality Assurance Department shall function to audit station activities. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Station Emergency Plan and implementing procedures at least once per 12 months.
- f. The Station Security Plan and implementing procedures at least once per 12 months.
- g. Any other area of facility operation considered appropriate by the Executive Manager-Quality Assurance or the Senior Vice President-Power Operations.
- h. The Station Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.



## ADMINISTRATIVE CONTROLS

- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.
- l. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
- m. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.

### AUTHORITY

6.5.3.2 The Quality Assurance Department shall report to and advise the Executive Manager-Quality Assurance, who shall advise the Senior Vice President-Engineering and Construction on those areas of responsibility specified in Section 6.5.3.1.

### RECORDS

6.5.3.3 Records of the Quality Assurance Department audits shall be prepared and maintained in the department files. Audit reports shall be disseminated as indicated below:

- 1. Vice President-Nuclear Operations
- 2. Nuclear Power Station Manager
- 3. Manager-Nuclear Operations Support
- 4. Manager-Nuclear Programs & Licensing
- 5. Executive Manager-Quality Assurance
- 6. Director-Safety Evaluation and Control
- 7. Nuclear Power Station Manager-Quality Assurance
- 8. Supervisor(s) of Area Audited



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 87 AND 73 TO

FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2

DOCKET NOS. 50-338 AND 50-339

INTRODUCTION:

By letter dated August 29, 1986, the Virginia Electric and Power Company (the licensee) requested a change to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). Specifically, the proposed change would revise Section 6, Administrative Controls, to reflect current reorganizational changes which would enhance the Quality Assurance (QA) organization. In addition, the proposed changes would reflect recent corporate and station title changes.

DISCUSSION:

The proposed changes for NA-1&2 TS, Section 6, Administrative Controls, are as follows:

- (1) Reflect a recent company reorganization in which the Quality Assurance organization will now report to the Senior Vice President-Engineering and Construction, instead of the Senior Vice President-Power Operations;
- (2) Change the title of the QC Supervisor reporting to the Manager, QA from "Supervisor - Quality Control - Q.A. Activities" to "Supervisor - Quality,"
- (3) Change the title of "Supervisor Health Physics" to "Superintendent - Health Physics,"
- (4) Change the title of "Director - Emergency Planning" to "Supervisor - Corporate Emergency Planning," and
- (5) Change the facility organization chart to reflect the above recent administrative title changes.

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Since the major emphasis of the licensee's nuclear program is on operations rather than construction, it is appropriate that the QA organization be realigned with construction. Item 1 above will enhance the independence of the QA organization from operations and thereby ensure the independence required for the QA organization to perform its intended function. The other items are administrative in nature and involve changes in nomenclature as well as achieving consistency with the licensee's NRC approved QA Topical Report.

#### EVALUATION:

The proposed changes specified above fall within the NA-1&2 Administrative Section of the TS. As such, the changes are administrative in nature. However, the proposed change to remove the QA organization from operations will enhance the independence of the QA organization to properly perform its intended functions. Also, the proposed changes will provide consistency with the licensee's NRC approved QA Topical Report. Therefore, based on all of the above, we find the proposed NA-1&2 TS change to be acceptable.

#### ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: November 10, 1986

Principal Contributor: L. B. Engle