

March 12, 1986

Docket Nos. 50-338
and 50-339

DISTRIBUTION

<u>Docket File</u>	B. Grimes
NRC PDR	J. Partlow
Local PDR	T. Barnhart (8)
PAD#2 rdg	W. Jones
T. Novak	V. Benaroya
P. Kreutzer	Tech Branch
L. Engle	ACRS (10)
OELD	OPA
L. Harmon	C. Miles, LFMB
E. Jordan	Gray File
	K. W. VanDyne

Mr. W. L. Stewart
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Dear Mr. Stewart:

The Commission has issued the enclosed Amendment Nos. 75 and 62 to Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The amendments revise the Technical Specifications (TS) in response to your letter dated May 2 and revised September 19, 1985. The amendments are effective as of the date of issuance.

The amendments clarify the NA-1&2 TS 3.6.1.3, Action Statement A.1, to permit entries into the airlock for repair of an inoperable inner air lock door. Also, the amendments change the allowable seal leakage from zero to a small measurable amount. The allowable seal leakage is only a small part of either the total Type B and C leakage limits specified by 10 CFR Part 50, Appendix J.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

/s/

Leon B. Engle, Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A

Enclosure:

1. Amendment No. 75 to NPF-4
2. Amendment No. 62 to NPF-7
3. Safety Evaluation

cc w/enclosures:

See next page

LA:PBD-8
PKreutzer
3/15/86

PM: *[Signature]*
LEngle:hc
3/16/86

D:PAD-7
LRubenstein
3/16/86

OELD
3/10/86

*No legal objection
McGowan*

Mr. W. L. Stewart
Virginia Electric & Power Company

Richard M. Foster, Esq.
Musick, Williamson, Schwartz,
Leavenworth & Cope, P.C.
P.O. Box 4579
Boulder, Colorado 80306

Michael W. Mauphin, Esq.
Hunton, Williams, Gay and Gibson
P.O. Box 1535
Richmond, Virginia 23212

Mr. W. T. Lough
Virginia Corporation Commission
Division of Energy Regulation
P.O. Box 1197
Richmond, Virginia 23209

Mr. J. T. Rhodes
Executive Vice President - Power
Virginia Electric and Power Co.
Post Office Box 26666
Richmond, Virginia 23219

Mr. Anthony Gambardella
Office of the Attorney General
Supreme Court Building
101 North 8th Street
Richmond, Virginia 23219

Resident Inspector/North Anna
c/o U.S. NRC
Senior Resident Inspector
Route 2, Box 78
Mineral, Virginia 23117

Mrs. Margaret Dietrich
Route 2, Box 568
Gordonsville, Virginia 22042

Mr. Paul W. Purdom
Environmental Studies Institute
Drexel University
32nd and Chestnut Streets
Philadelphia, Pennsylvania 19104

North Anna Power Station

Atomic Safety and Licensing Appeal
Board Panel
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Regional Administrator
USNRC, Region II
101 Marietta Street, N.W., Suite 2900
Atlanta, Georgia 30323

Mr. E. W. Harrell
P.O. Box 402
Mineral, Virginia 23117

Old Dominion Electric Cooperative
c/o Executive Vice President
Innsbrook Corporate Center
4222 Cox Road, Suite 102
Glen Allen, Virginia 23060

Mr. Richard C. Klepper
Board of Supervisors
Louisa County Courthouse
P.O. Box 27
Louisa, Virginia 23093



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated May 2 and revised September 19, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8603210280 860312
PDR ADDCK 05000338
P PDR

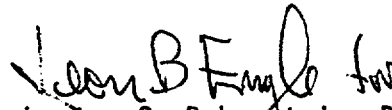
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 75, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 12, 1986

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- b. If any periodic Type A test fails to meet $.75 L_a$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet $.75 L_a$, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet $.75 L_a$ at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
1. Confirms the accuracy of the Type A test by verifying that the difference between supplemental and Type A test data is within $0.25 L_a$
 2. Has a duration sufficient to establish accurately the change in leakage between the Type A test and the supplemental test.
 3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage rate at $P_a \geq 40.6$ psig.
- d. Type B and C tests shall be conducted with gas at $P_a \geq 40.6$ psig, at intervals no greater than 24 months except^a for tests involving:
1. Air locks,
 2. Penetrations using continuous leakage monitoring systems
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Type B test for penetrations employing a continuous leakage monitoring system shall be conducted at $P_a \geq 40.6$ psig at intervals no greater than once per 3 years.
- g. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.
- h. The provisions of Specification 4.0.2 are not applicable.

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
- b. An overall air lock leakage rate of less than or equal to $0.05 L_a$ at P_a greater than or equal to 40.6 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With one containment air lock door inoperable:
 1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.+
 2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be locked closed at least once per 31 days.
 3. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
 4. The provisions of Specification 3.0.4 are not applicable.
- b. With a containment air lock inoperable, except as the result of an inoperable air lock door, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

- a. *Within 72 hours following closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying that the seal leakage is less than $0.01 L_a$ as determined by precision flow measurements when measured for at least 30 seconds with the volume between the seals at a pressure of greater than or equal to 40.6 psig.
- b. At least once per 6 months by conducting an overall air lock leakage test at greater than or equal to P_a , 40.6 psig, and by verifying that the overall air lock leakage rate is within its limit#, and
- c. At least once per 18 months during shutdown by verifying that only one door in each air lock can be opened at a time.

+Entry to repair the inner air lock door, if inoperable, is allowed.

*Exempt to Appendix "J" of 10 CFR Part 50.

#The provisions of Specification 4.0.2 are not applicable.

ATTACHMENT TO LICENSE AMENDMENT NO. 75

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page

3/4 6-4



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 62
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated May 2 and revised September 19, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

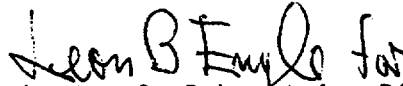
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 62, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 12, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 62

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page

3/4 6-4

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- b. If any periodic Type A test fails to meet $.75 L_a$ the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet $.75 L_a$ a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet $.75 L_a$ at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
 - 1. Confirms the accuracy of the Type A test by verifying that the difference between supplemental and Type A test data is within $0.25 L_a$
 - 2. Has a duration sufficient to establish accurately the change in leakage between the Type A test and the supplemental test.
 - 3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage rate at P_a , greater than or equal to 40.6 psig.
- d. Type B and C tests shall be conducted with gas at P_a , greater than or equal to 40.6 psig, at intervals no greater than 24 months except for tests involving:
 - 1. Air locks,
 - 2. Penetrations using continuous leakage monitoring systems
- e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- f. Type B test for penetrations employing a continuous leakage monitoring system shall be conducted at P_a , greater than or equal to 40.6 psig, at intervals no greater than once per 3 years.
- g. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.
- h. The provisions of Specification 4.0.2 are not applicable.

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
- b. An overall air lock leakage rate of less than or equal to $0.05 L_a$ at P_a , greater than or equal to 40.6 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With one containment air lock door inoperable:
 1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.+
 2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be locked closed at least once per 31 days.
 3. Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
 4. The provisions of Specification 3.0.4 are not applicable.
- b. With a containment air lock inoperable, except as the result of an inoperable air lock door, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

- a. *Within 72 hours following closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying that the seal leakage is less than $0.01 L_a$ as determined by precision flow measurements when measured for at least 30 seconds with the volume between the seals at a pressure of greater than or equal to 40.6 psig.

+Entry to repair the inner air lock door, if inoperable, is allowed.

*Exempt to Appendix "J" of 10 CFR Part 50.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 75 AND 62 TO

FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2

DOCKET NOS. 50-338 AND 50-339

Introduction:

By letter dated May 2 and revised September 19, 1985 (Serial Nos. 85-162 and 85-162A, respectively), the Virginia Electric and Power Company (the licensee) requested a change to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). Specifically, the proposed changes would provide consistency between the TS for NA-1&2 as specified in the Standard TS (STS) for Westinghouse Pressurized Water Reactors, Revision 4 and Revision 5 (draft). In addition, the proposed changes would clarify that entry into an air lock is permitted for the repair of an inoperable inner air lock door. The proposed changes would also revise the allowable seal leakage from zero to a small permissible amount well within the limits established 10 CFR Part 50, Appendix J.

Discussion:

A footnote would be added to Surveillance Requirement 4.6.1.3.b to make the provisions of TS 4.0.2 (Time Interval Specified For Surveillance Requirements) not applicable. The proposed change would provide consistency between the NA-1&2 TS and also with the Standard TS for Westinghouse Pressurized Water Reactors, Revision 4 and Revision 5 (Draft). In addition, the footnote would be in conformance with the requirements of 10 CFR Part 50, Appendix J.

Two of the proposed changes are not in explicit conformance with the Westinghouse Standard TS. They are:

1. A footnote would be added to TS 3.6.1.3, Action Statement A.1, to clarify that entry into an air lock is permitted for the repair of an inoperable inner air lock door. This clarification has been accepted by the NRC staff on previous occasions for other facilities.
2. Surveillance Requirement 4.6.1.3.a would be changed to allow less than 0.01 L seal leakage as determined by precision flow measurements when measured for at least 30 seconds. This leakage rate is less than the overall air lock leakage limit and well within the total Type B and C leakage limit as established by 10 CFR Part 50 Appendix J.

Evaluation:

As discussed above, the proposed changes are in conformance with the NRC approved Westinghouse Standard TS, Revision 4 and with the requirements of 10 CFR Part 50, Appendix J. Also, the proposed change regarding the repair of an inoperable inner air lock door has already been approved by the NRC for other facilities and is appropriately applied to NA-1&2. Therefore, based on the above, the staff finds the proposed changes to be acceptable for NA-1&2.

Environmental Consideration

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 12, 1986

Principal Contributors: K. W. VanDyne and L. B. Engle