DCR 076

Docket Nos. 50-338 and 50-339 DISTRIBUTION:
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Mr. W. L. Stewart Vice President - Nuclear Operations Virginia Electric and Power Company

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Post Office Box 26666 Richmond, Virginia 23261

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Dear Mr. Stewart:

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The Commission has issued the enclosed Amendment Nos. 61 and 45 to Facility Operating Licenses No. NPF-4 and No. NPF-7 for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The amendments revise the Technical Specifications (TS) in response to your letter dated August 20, 1982 as supplemented by letters dated October 21, 1982, June 16, July 25, September 13, October 28, November 10, December 6, 1983, and April 10, May 8, and May 18, 1984.

The amendments revise the capacity of the Spent Fuel Storage Pool at NA-1&2. Specifically, the NA-1&2 Technical Specification 5.6.1 identifies a new nominal center-to-center spacing between fuel assemblies of 10 and 9/16 inches. In addition, the NA-1&2 TS 5.6.3 is revised to modify the spent fuel storage capacity to 1737 fuel assemblies. Finally, the amendments revise the NA-1&2 TS 5.6.1 and 5.6.3 to be identical with the NRC approved TS 5.6.1 and 5.6.3 in order to provide consistency between the NA-1&2 TS.

A copy of the Safety Evaluation and Environmental Assessment issued July 2, 1984 and applicable to the revised spent fuel storage capacity for 1737 fuel assemblies is also enclosed. The notice of issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

Leon B. Engle, Project Manager Operating Reactors Branch #3 Division of Licensing

Enclosure:

1. Amendment No. 61 to NPF-4

2. Amendment No. 45 to NPF-7

Safety Evaluation

4. Environmental Assessment

cc w/enclosures:
See next page

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Virginia Electric and Power Company

cc: Richard M. Foster, Esquire Musick, Williamson, Schwartz, Leavenworth & Cope, P.C. P. O. Box 4579 Boulder, Colorado 80306

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Old Dominion Electric Cooperative c/o Executive Vice President Innsbrook Corporate Center 4222 Cox Road, Suite 102 Glen Allen, Virginia 23060

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 61 License No. NPF-4

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated August 20, 1982 as supplemented by letters dated October 21, 1982, June 16, July 25, September 13, October 28, November 10, December 6, 1983, and April 10, May 8, and May 18, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 61 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James R. Miller, Chief Operating Reactors Branch #3

Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: December 21, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 61 TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Page

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5-6

IDESIGN FEATURES

- In accordance with the code requirements specified in Section
 5.2 of the FSAR, with allowance for normal degradation pursuant to the applicable Surveillance Requirements,
- b. For a pressure of 2485 psig, and
- For a temperature of 650°F, except for the pressurizer which is 680°F.

VOLUME

5.4.2 The total water and steam volume of the reactor coolant system is 9957 \pm 10 cubic feet at a nominal T_{avg} of 525°F.

5.5 METEOROLOGICAL TOWER LOCATION

5.5.1 The meteorological tower shall be located as shown on Figure 5.1-1.

5.6 FUEL STORAGE

CRITICALITY

- 5.6.1.1 The spent fuel storage racks are designed and shall be maintained with:
 - a. A k_{eff} equivalent to less than or equal to 0.95 when flooded with unborated water, which includes a conservative allowance of 3.4% delta k/k for uncertainties.
 - b. A nominal 10 9/16 inch center-to-center distance between fuel assemblies placed in the storage racks.
- 5.6.1.2 The new fuel pit storage racks are designed and shall be maintained with a nominal 21 inch center-to-center distance between new fuel assemblies such that, on a best estimate basis, keff will not exceed .98, with fuel of the highest anticipated enrichment in place, when aqueous foam moderation is assumed.
- 5.6.1.3 If new fuel for the first core loading is stored dry in the spent fuel storage racks, the center-to-center distance between the new fuel assemblies will be administratively limited to 28 inches and the $k_{\mbox{eff}}$ shall not exceed 0.98 when aqueous foam moderation is assumed.

DESIGN FEATURES

DRAINAGE

5.6.2 The spent fuel pit is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 288.83 feet. Mean Sea Level, USGS datum.

CAPACITY

5.6.3 The fuel storage pool is designed and shall be maintained with a storage capacity limited to no more than 1737 fuel assemblies.

5.7 COMPONENT CYCLIC OR TRANSIENT LIMIT

5.7.1 The components identified in Table 5.7-1 are designed and shall be maintained within the cyclic or transient limits of Table 5.7-1.



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 45 License No. NPF-7

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated August 20, 1982 as supplemented by letters dated October 21, 1982, June 16, July 25, September 13, October 28, November 10, December 6, 1983, and April 10, May 8, and May 18, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 45, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James R. Miller, Chief Operating Reactors Branch #3

Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: December 21, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page

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5.5 METEOROLOGICAL TOWER LOCATION

5.5.1 The meteorological tower shall be located as shown on Figure 5.1-1.

5.6 FUEL STORAGE

CRITICALITY

- 5.6.1.1 The spent fuel storage racks are designed and shall be maintained with:
 - a. A K_{eff} equivalent to less than or equal to 0.95 when flooded with unborated water, which includes a conservative allowance of 3.4% delta k/k for uncertainties.
 - b. A nominal 10 9/16 inch center-to-center distance between fuel assemblies placed in the storage racks.
- 5.6.1.2 The new fuel pit storage racks are designed and shall be maintained with a nominal 21 inch center-to-center distance between new fuel assemblies such that, on a best estimate basis, $k_{\mbox{eff}}$ will not exceed .98, with fuel of the highest anticipated enrichment in place, when aqueous foam moderation is assumed.
- 5.6.1.3 If new fuel for the first core loading is stored dry in the spent fuel storage racks, the center-to-center distance between the new fuel assemblies will be administratively limited to 28 inches and the $k_{\mbox{eff}}$ shall not exceed 0.98 when aqueous foam moderation is assumed.

DRAINAGE

5.6.2 The spent fuel pit is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 288.83 feet Mean Sea Level, USGS datum.

CAPACITY

5.6.3 The fuel storage pool is designed and shall be maintained with a storage capacity limited to no more than 1737 fuel assemblies.

5.7 COMPONENT CYCLIC or TRANSIENT LIMIT

5.7.1 The components identified in Table 5.7-1 are designed and shall be maintained within the cyclic or transient limits of Table 5.7-1.