

August 29, 1985

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Docket Nos. 50-338  
and 50-339

Mr. W. L. Stewart  
Vice President - Nuclear Operations  
Virginia Electric and Power Company  
Post Office Box 26666  
Richmond, Virginia 23261

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Dear Mr. Stewart:

The Commission has issued the enclosed Amendment Nos. 67 and 53 to Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The amendments revise the Technical Specifications (TS) in response to your letter dated September 28, 1984. The amendments are effective as of the date of issuance.

The amendments are administrative in nature and correct discrepancies presently existing in the NA-1&2 TS which relate to the Radiological Effluent Technical Specifications (RETS). The amendments add the Containment Vacuum Steam Ejector (Hogger) as a gaseous release path that is monitored and specify the figure for the Low Population Zone in the appropriate TS. Also, the location of the Meteorology tower is included in the appropriate TS figure and correct numbers are assigned to appropriate TS Table numbers.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

/S/

Leon B. Engle, Project Manager  
Operating Reactors Branch #3  
Division of Licensing

Enclosure:

1. Amendment No.67 to NPF-4
2. Amendment No.53 to NPF-7
3. Safety Evaluation

cc w/enclosures:  
See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.67  
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated September 28, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 67, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Edward J. Butcher, Acting Chief  
Operating Reactors Branch #3  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 29, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 67

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Page

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TABLE 4.11-2

RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) <sup>a</sup> (uCi/ml)
A. Waste Gas Storage Tank	P Each Tank Grab Sample	P Each Tank	Principal Gamma Emitters <sup>b</sup>	1x10 <sup>-4</sup>
B. Containment PURGE	P Each PURGE Grab Sample	P Each PURGE	Principal Gamma Emitters <sup>b</sup>	1x10 <sup>-4</sup>
			H-3	1x10 <sup>-6</sup>
C. Process Vent Vent. Vent A Vent. Vent B	M <sup>c,d,e</sup> Grab Sample	M <sup>c</sup>	Principal Gamma Emitters <sup>b</sup>	1x10 <sup>-4</sup>
			H-3	1x10 <sup>-6</sup>
D. All Release Types as listed in A, B, C above.	Continuous <sup>d</sup>	W <sup>e</sup> Charcoal Sample	I-131	1x10 <sup>-12</sup>
	Continuous <sup>d</sup>	W <sup>e</sup> Particulate Sample	Principal Gamma Emitters <sup>b</sup>	1x10 <sup>-11</sup>
	Continuous <sup>d</sup>	M Composite Particulate Sample	Gross Alpha	1x10 <sup>-11</sup>
	Continuous <sup>d</sup>	Q Composite Particulate Sample	Sr-89, Sr-90	1x10 <sup>-11</sup>
	Continuous <sup>d</sup>	Noble Gas Monitor	Noble Gases Gross Beta or Gamma	1x10 <sup>-6</sup>

TABLE 4.11-2 (Continued)

RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) <sup>a</sup> (uCi/ml)
E. Condenser Air Ejector Vent	W Grab Sample	W	Principle Gamma Emitters <sup>g</sup>	$1 \times 10^{-4}$
Steam Generator Blowdown Vent			H-3	$1 \times 10^{-6}$
F. Containment Vacuum Steam Ejector (Hogger)	P Grab Sample	P	Principle Gamma Emitters <sup>b</sup>	$1 \times 10^{-4}$
			H-3	$1 \times 10^{-6}$

## RADIOACTIVE EFFLUENTS

### GASEOUS RADWASTE TREATMENT

#### LIMITING CONDITION FOR OPERATION

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3.11.2.4 The GASEOUS RADWASTE TREATMENT SYSTEM and the VENTILATION EXHAUST TREATMENT SYSTEM shall be used to reduce radioactive materials in gaseous waste prior to their discharge when the projected gaseous effluent air doses due to gaseous effluent releases, from each reactor unit, from the site to areas at and beyond the SITE BOUNDARY (see Figure 5.1-1) would exceed 0.2 mrad for gamma radiation and 0.4 mrad for beta radiation over 31 days. The VENTILATION EXHAUST TREATMENT SYSTEM shall be used to reduce radioactive materials in gaseous waste prior to their discharge when the projected doses due to gaseous effluent releases, from each reactor unit, from the site to areas at and beyond the SITE BOUNDARY (see Figure 5.1-1) would exceed 0.3 mrem to the critical organ\* over 31 days.

APPLICABILITY: At all times.

ACTION:

- a. With gaseous waste being discharged without treatment and in excess of the above limits, in lieu of a Licensee Event Report, prepare and submit to the Commission within 30 days, pursuant to Specification 6.9.2, a Special Report that includes the following information:
  1. Explanation of why gaseous radwaste was being discharged without treatment, identification of any inoperable equipment or sub-systems, and the reason for the inoperability,
  2. ACTION(s) taken to restore the inoperable equipment to OPERABLE status, and
  3. Summary description of ACTION(s) taken to prevent a recurrence.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.11.2.4.1 Doses due to gaseous releases from the site shall be projected at least once per 31 days in accordance with the ODCM.

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\*The critical organ is addressed in the ODCM.



RADIOACTIVE EFFLUENTS

EXPLOSIVE GAS MIXTURE

LIMITING CONDITION FOR OPERATION

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3.11.2.5 The concentration of oxygen in the waste gas decay tanks shall be limited to less than or equal to 2% by volume whenever the hydrogen concentration exceeds 4% by volume and is less than 96% by volume.

APPLICABILITY: At all times.

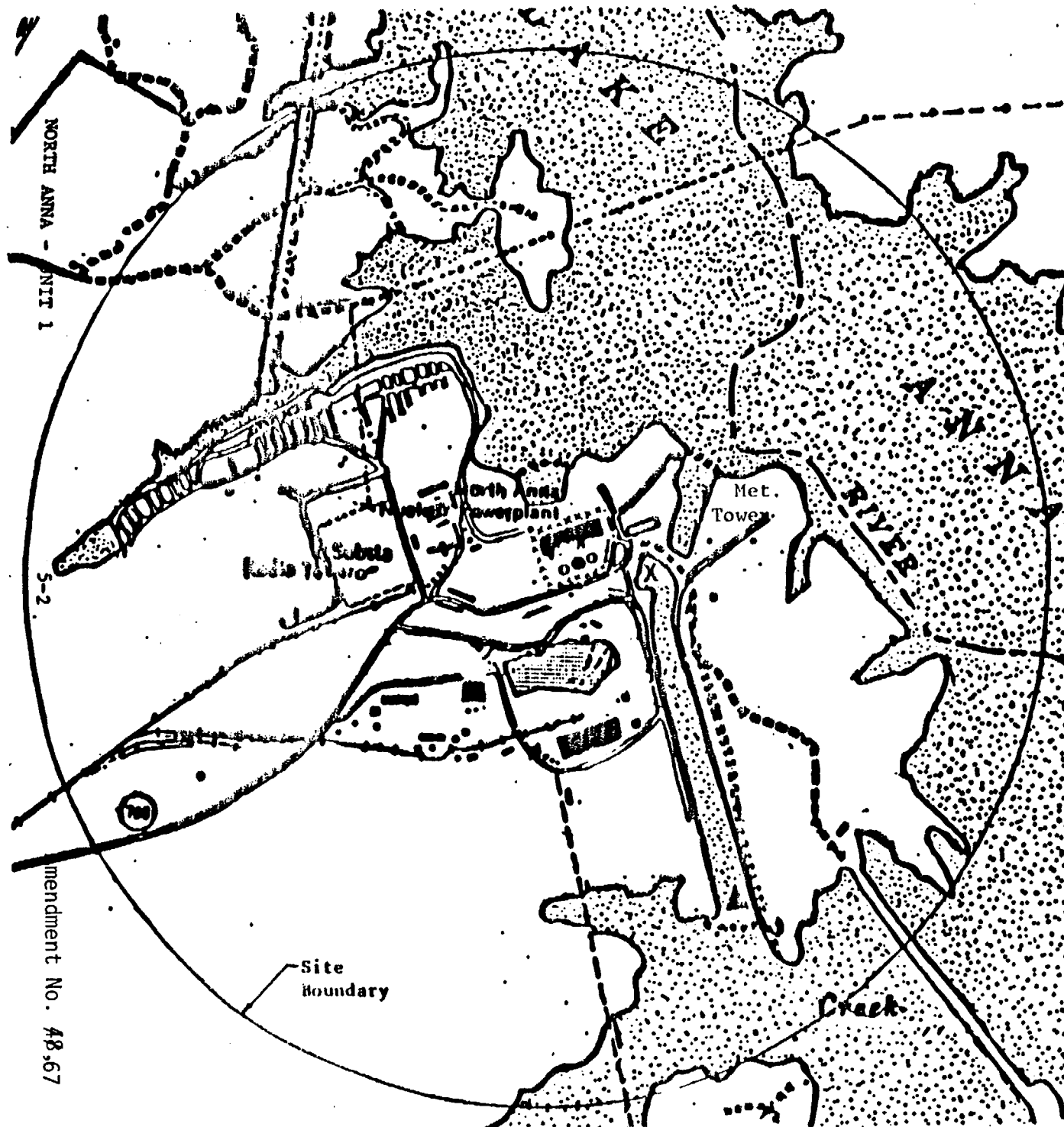
ACTION:

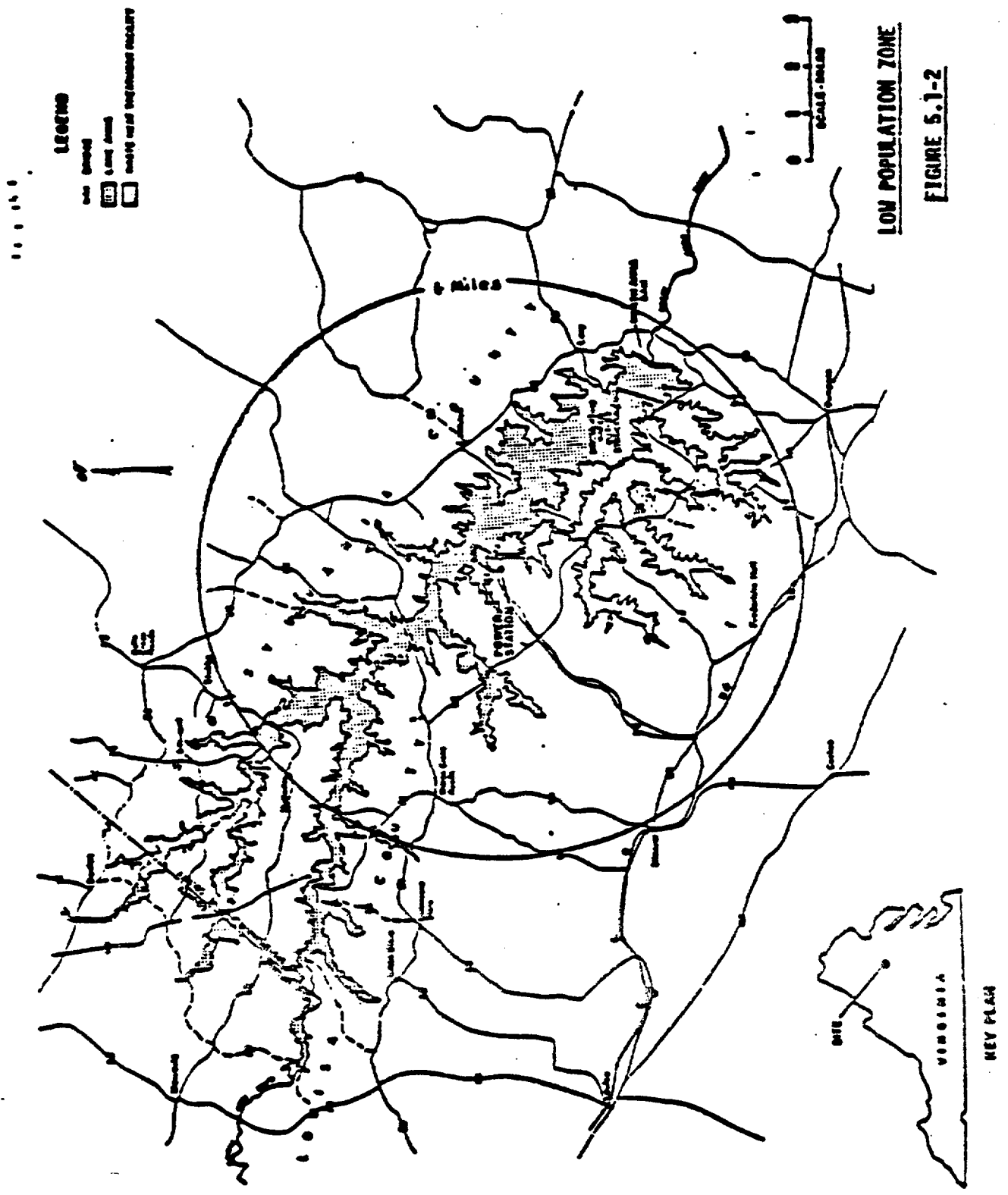
- a. With the concentration of oxygen in the waste gas decay tanks greater than 2% by volume but less than or equal to 4% by volume, reduce the oxygen concentration to the above limits within 48 hours.
- b. With the concentration of oxygen in the waste gas decay tanks greater than 4% volume and the hydrogen concentration greater than 2% by volume, immediately suspend all additions of waste gases to the system and reduce the concentration of oxygen to less than or equal to 2% by volume without delay.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

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4.11.2.5 The concentrations of hydrogen and oxygen in the waste gas decay tanks shall be determined to be within the above limits by continuously monitoring the waste gases in the waste gas decay tanks with the hydrogen and oxygen monitors required OPERABLE by Table 3.3-14 of Specification 3.3.3.11.







UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 53  
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company, et al., (the licensee) dated September 28, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 53, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Edward J. Butcher, Acting Chief  
Operating Reactors Branch #3  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 29, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 53

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Page

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5-1

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TABLE 4.11-2

## RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) <sup>a</sup> (uCi/ml)
A. Waste Gas Storage Tank	<sup>P</sup> Each Tank Grab Sample	<sup>P</sup> Each Tank	Principal Gamma Emitters <sup>b</sup>	$1 \times 10^{-4}$
B. Containment PURGE	<sup>P</sup> Each PURGE Grab Sample	<sup>P</sup> Each PURGE	Principal Gamma Emitters <sup>b</sup>	$1 \times 10^{-4}$
			H-3	$1 \times 10^{-6}$
C. Process Vent Vent. Vent A Vent. Vent B	<sup>M</sup> Grab Sample	<sup>M</sup>	Principal Gamma Emitters <sup>b</sup>	$1 \times 10^{-4}$
			H-3	$1 \times 10^{-6}$
D. All Release Types as listed in A, B, C above.	Continuous <sup>d</sup>	<sup>W</sup> <sup>e</sup> Charcoal Sample	I-131	$1 \times 10^{-12}$
	Continuous <sup>d</sup>	<sup>W</sup> <sup>e</sup> Particulate Sample	Principal Gamma Emitters <sup>b</sup>	$1 \times 10^{-11}$
	Continuous <sup>d</sup>	<sup>M</sup> Composite Particulate Sample	Gross Alpha	$1 \times 10^{-11}$
	Continuous <sup>d</sup>	<sup>Q</sup> Composite Particulate Sample	Sr-89, Sr-90	$1 \times 10^{-11}$
	Continuous <sup>d</sup>	Noble Gas Monitor	Noble Gases Gross Beta or Gamma	$1 \times 10^{-6}$

TABLE 4.11-2 (Continued)

RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

Gaseous Release Type	Sampling Frequency	Minimum Analysis Frequency	Type of Activity Analysis	Lower Limit of Detection (LLD) <sup>a</sup> (uCi/ml)
E. Condenser Air Ejector Vent	W Grab Sample	W	Principle Gamma Emitters <sup>g</sup>	$1 \times 10^{-4}$
Steam Generator Blowdown Vent			H-3	$1 \times 10^{-6}$
F. Containment Vacuum Steam Ejector (Hogger)	P Grab Sample	P	Principle Gamma Emitters <sup>b</sup>	$1 \times 10^{-4}$
			H-3	$1 \times 10^{-6}$



## 5.0 DESIGN FEATURES

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### 5.1 SITE

#### EXCLUSION AREA

5.1.1 The exclusion area shall be as shown in Figure 5.1-1.

#### LOW POPULATION ZONE

5.1.2 The low population zone shall be as shown in Figure 5.1-2.

#### MAP DEFINING UNRESTRICTED AREAS FOR RADIOACTIVE GASEOUS AND LIQUID EFFLUENTS

5.1.3 Information regarding radioactive gaseous and liquid effluents, which allows identification of structures and release points as well as definition of UNRESTRICTED AREAS within the SITE BOUNDARY that are accessible to MEMBERS OF THE PUBLIC, shall be as shown in Figure 5.1-1.

### 5.2 CONTAINMENT

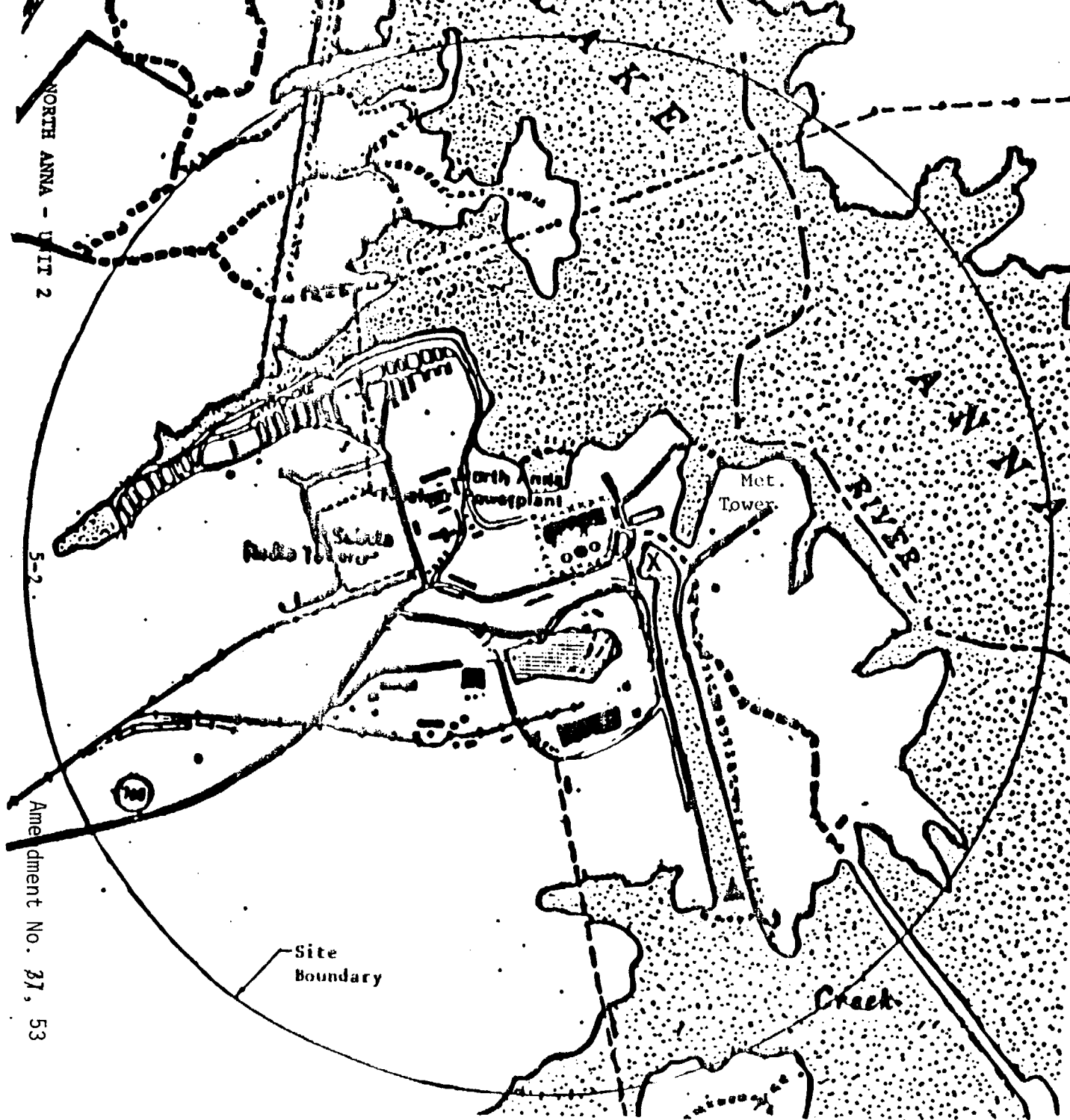
#### CONFIGURATION

5.2.1 The reactor containment building is a steel lined, reinforced concrete building of cylindrical shape with a dome roof and having the following design features:

- a. Nominal inside diameter = 126 feet.
- b. Nominal inside height = 190 feet, 7 inches.
- c. Minimum thickness of concrete walls = 4.5 feet.
- d. Minimum thickness of concrete roof = 2.5 feet.
- e. Minimum thickness of concrete floor pad = 10 feet.
- f. Nominal thickness of the cylindrical portion of the steel liner = 3/8 inches.
- g. Net free volume =  $1.825 \times 10^6$  cubic feet.
- h. Nominal thickness of hemispherical dome portion of the steel liner = 1/2 inch.

#### DESIGN PRESSURE AND TEMPERATURE

5.2.2. The reactor containment building is designed and shall be maintained for a maximum internal pressure of 45 psig and a temperature of 280°F.



NORTH ANNA - UNIT 2

5-2.

Amendment No. 37, 53

Site Boundary

Met. Tower

Crack



## DESIGN FEATURES

### 5.3 REACTOR CORE

#### FUEL ASSEMBLIES

5.3.1 The reactor core shall contain 157 fuel assemblies with each fuel assembly containing 264 fuel rods clad with Zircaloy-4. Each fuel rod shall have a nominal active fuel length of 144 inches and contain a maximum total weight of 1780 grams uranium. The initial core loading shall have a maximum enrichment of 3.2 weight percent U-235. Reload fuel shall be similar in physical design to the initial core loading and shall have a maximum enrichment of 4.1 weight percent U-235.

#### CONTROL ROD ASSEMBLIES

5.3.2 The reactor core shall contain 48 full length control rod assemblies. The full length control rod assemblies shall contain a nominal 142 inches of absorber material. The nominal values of absorber material shall be 80 percent silver, 15 percent indium and 5 percent cadmium. All control rods shall be clad with stainless steel tubing.

### 5.4 REACTOR COOLANT SYSTEM

#### DESIGN PRESSURE AND TEMPERATURE

5.4.1 The reactor coolant system is designed and shall be maintained:

- a. In accordance with the code requirements specified in Section 5.2 of the FSAR, with allowance for normal degradation pursuant to the applicable Surveillance Requirements,
- b. For a pressure of 2485 psig, and
- c. For a temperature of 650°F, except for the pressurizer which is 680°F.

#### VOLUME

5.4.2 The total water and steam volume of the reactor coolant system is  $9957 \pm 10$  cubic feet at a nominal  $T_{avg}$  of 525°F.

## ADMINISTRATIVE CONTROLS (Continued)

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- g. Records of reactor tests and experiments.
- h. Records of training and qualification for current members of the plant staff.
- i. Records of in-service inspections performed pursuant to these Technical Specifications.
- j. Records of Quality Assurance activities required by the QA Manual.
- k. Records of the service life of all hydraulic and mechanical snubbers listed in Tables 3.7-4a and 3.7-4b including the date at which the service life commences and associated installation and maintenance records.
- l. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- m. Records of meetings of the SNSOC.
- n. Records of meetings of the System Nuclear Safety and Operating Committee to issuance of Amendment No. 11.
- o. Records of secondary water sampling and water quality.
- p. Records for Environmental Qualification which are covered under the provisions of Paragraph 2.C(4)(e) of License No. NPF-7.
- q. Records of analyses required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analysis at a later date. This would include procedures effective at specified times and QA records showing that these procedures were followed.

### 6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

### 6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance

#### ADMINISTRATIVE CONTROLS (Continued)

thereto shall be controlled by requiring issuance of a Radiation Work Permit\*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
- c. An individual qualified in the protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the facility Health Physicist in the Radiation Work Permit.

6.12.2 The requirements of 6.12.1, above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Supervisor on duty and/or the Plant Health Physicist.

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\*Health Physics personnel or personnel escorted by Health Physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry in high radiation areas.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NOS. 67 AND 53 TO  
FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7  
VIRGINIA ELECTRIC AND POWER COMPANY  
OLD DOMINION ELECTRIC COOPERATIVE  
NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2  
DOCKET NOS. 50-338 AND 50-339

Introduction:

By letter dated September 28, 1984, the Virginia Electric and Power Company (the licensee) requested changes to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The proposed changes are administrative in nature and correct errors in the Radiological Effluent Technical Specifications (RETS) provided in the licensee's previous submittals and which were approved by the NRC on May 5, 1983.

Discussion:

A proposed change to the NA-1&2 TS 4.11-2 would include the Containment Vacuum Steam Ejector (Hogger) as a gaseous release path that should be monitored. A containment air sample would be grab-sampled and have a minimum analysis frequency of prior to each release for each ejector. The above release path was inadvertently left out of the licensee's RETS submittals of December 16, 1982 and February 25, 1983 which were subsequently approved by the NRC on May 5, 1983 as Amendments No. 48 and No. 31 for NA-1&2, respectively.

A proposed change to the NA-1&2 TS 5.1.2 would indicate that the low population zone is shown in Figure 5.1-2 instead of Figure 5.1-1. Figure 5.1-2 was inadvertently deleted in the licensee's previous submittals.

The NA-1&2 TS Figure 5.1-1 would be revised to include the location of the Meteorology (MET) tower. The location of the MET tower was inadvertently left out of the licensee's previous submittals.

A proposed change to the NA-1 TS 4.11.2.5 would revise the table number that is referenced in the surveillance requirement. The table number should be revised to indicate Table 3.3-14 instead of Table 3.3-13.

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Finally, Amendment No. 11 was inadvertently left off the NA-2 TS 6.10.2n. The proposed change to 6.10.2n would make the requirement read "Records of meetings of the System Nuclear Safety and Operating Committee to issuance of Amendment No. 11."

#### Evaluation:

The proposed changes are needed to correct the above stated administrative discrepancies inadvertently submitted by the licensee in the NA-1&2 RETS submittals and the subsequent issuance of Amendments No. 48 and No. 31 for NA-1&2, respectively. The proposed changes correct errors and provide consistency to the NA-1&2 TS, and on this basis, are therefore acceptable.

#### ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 29, 1985

Principal Contributor:  
Leon Engle