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MAR 6 1984

Docket No. 50-338

Mr. W. L. Stewart Vice President - Nuclear Operations Virginia Electric and Power Company Post Office Box 26666 Richmond, Virginia 23261

Dear Mr. Stewart:

The Commission has issued the enclosed Amendment No. 53 to Facility Operating License No. NPF-4 for the North Anna Power Station, Unit No. 1 (NA-1). The amendment revises the NA-1 Technical Specifications in response to your letter dated November 15, 1983 (Serial No. 646), and is effective as of the date of issuance.

The amendment is administrative in nature and revises the NA-1 TS Table 4.3-2 by removing the requirement for surveillance in Mode 4 (Hot Shutdown) for the auxiliary feedwater pump actuation signals from steam generator water level low-low and station blackout.

A copy of the Safety Evaluation is enclosed. The notice of issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

Original signed by:

Leon B. Engle, Project Manager Operating Reactors Branch #3 Division of Licensing

Enclosure:

1. Amendment No. 53 to NPF-4

2. Safety Evaluation

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

## VIRGINIA ELECTRIC AND POWER COMPANY

## OLD DOMINION ELECTRIC COOPERATIVE

### DOCKET NO. 50-338

#### NORTH ANNA POWER STATION, UNIT NO. 1

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 53 License No. NPF-4

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated November 15, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:
  - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 53, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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James R. Miller, Chief Operating Reactors Branch #3 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: March 6, 1984

### ATTACHMENT TO LICENSE AMENDMENT

## AMENDMENT NO. 53 TO FACILITY OPERATING LICENSE NO. NPF-4

### DOCKET NO. 50-338

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is provided to maintain document completness.

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## TABLE 4.3-2 (Continued)

# ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

| FUNCTIONAL UNIT |                                         |                                         | CHANNEL<br>CHECK                                 | CHANNEL<br>CALIBRATION | CHANNEL<br>FUNCTIONAL<br>TEST | MODES IN WHICH<br>SURVEILLANCE<br>REQUIRED |
|-----------------|-----------------------------------------|-----------------------------------------|--------------------------------------------------|------------------------|-------------------------------|--------------------------------------------|
| 5.              | TURBINE TRIP AND FEEDWATER<br>ISOLATION |                                         |                                                  |                        |                               |                                            |
|                 | a.                                      | Steam Generator Water<br>LevelHigh-High | S                                                | R                      | М                             | 1, 2, 3                                    |
| 6.              | AUXILIARY FEEDWATER PUMPS               |                                         |                                                  |                        |                               |                                            |
|                 | a.                                      | Manual                                  | N. A.                                            | N. A.                  | M(1)                          | 1, 2, 3                                    |
| • .             | b.                                      | Automatic Actuation<br>Logic            | N. A.                                            | N. A.                  | M(2)                          | 1, 2, 3                                    |
|                 | с.                                      | Steam Generator Water<br>LevelLow-Low   | S                                                | R                      | М                             | 1, 2, 3,                                   |
|                 | d.                                      | S.I.                                    | See 1 above (all S.I. Surveillance Requirements) |                        |                               |                                            |
|                 | e.                                      | Station Blackout                        | N. A.                                            | R                      | N. A.                         | 1, 2, 3,                                   |
|                 | f.                                      | Main Feedwater Pump<br>Trip             | N. A.                                            | N. A.                  | R                             | 1, 2                                       |
| 7.              |                                         | 5 OF POWER<br>5 KV Emergency Bus        |                                                  |                        |                               |                                            |
|                 | a.                                      | Loss of Voltage                         | N. A.                                            | R                      | M(2)                          | 1, 2, 3                                    |
|                 | b.                                      | Degraded Voltage                        | N. A.                                            | R                      | M(2)                          | 1, 2, 3                                    |

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### TABLE 4.3-2 (Continued)

### TABLE NOTATION

- (1) Manual actuation switches shall be tested at least once per 18 months during shutdown. All other circuitry associated with manual safeguards actuation shall receive a CHANNEL FUNCTIONAL TEST at least once every other 31 days.
- (2) Each train or logic channel shall be tested at least every other 31 days.
- (3) The CHANNEL FUNCTIONAL TEST shall include exercising the transmitter by applying either a vacuum or pressure to the appropriate side of the transmitter.

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

### SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 53 TO FACILITY OPERATING LICENSE NO. NPF-4

### VIRGINIA ELECTRIC AND POWER COMPANY

AND OLD DOMINION ELECTRIC COOPERATIVE

NORTH ANNA POWER STATION, UNIT NO. 1

DOCKET NO. 50-338

### Introduction

By letter dated November 15, 1983, the Virginia Electric and Power Company (the licensee) requested an amendment, in the form of changes to the Technical Specifications (TS), to Operating License No. NPF-4 for the North Anna Power Station Unit No. 1 (NA-1). Specifically, the proposed change would delete the requirement to perform surveillance in Mode 4 (Hot Shutdown) for the Auxiliary Feedwater Pump Actuation Signals (AFPAS) from Steam Generator Water Level (SGWL)-Low Low and Station Blackout (SB). The proposed change would provide consistency within the body of the NA-1 TS and also the NA-2 TS. Our discussion and evaluation of the proposed change is provided below.

#### Discussion:

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Presently, the NA-1 TS Table 3.3-3 requires that operability be verified by surveillance in Mode 1 (Power Operation), Mode 2 (Startup), and Mode 3 (Hot standby) for AFPAS from SGWL Low-Low and SB. However, Table 4.3.2 of the NA-1 TS requires that surveillance testing be performed for the AFPAS from SGWL Low-Low and SB not only in Modes 1, 2, and 3 but also Mode 4. Therefore, an inconsistency presently exists between the NA-1&2 TS Tables 3.3-3 and 4.3-2. In addition, the NA-2 TS require that surveillance of these actuation signals be performed in Modes 1, 2 and 3 only. Finally, as stated in NUREG-0452, Revision 4, Standard Technical Specifications for Westinghouse Pressurized Water Reactors, surveillance is required in Modes 1, 2 and 3 for these actuation signals.

Therefore, the proposed change would delete the requirement to perform surveillance of the above mentioned actuation signals from the NA-1 TS Table 4.3.2 in Mode 4 only.

#### Evaluation

Eliminating the required surveillance of the AFPAS from SGWL Low-Low and SB in Mode 4 from the NA TS Table 4.3.2 not only provides consistency with NA-1 Table 3.3-3, but also provides consistency with the approved NA-2 TS. In addition, the proposed change is in conformance with the NRC approved Westinghouse Standard Technical Specifications which are appropriately applied to NA-1. Therefore, based on the above, we find the proposed change to be acceptable.

#### Environmental Consideration

We have determined that the amendment is administrative in nature and does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR g51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### Conclusions

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 6, 1984

Principal Contributors:

Leon B. Engle