

DCS MS-014

OCT 20 1982

Docket No. 50-338

Mr. R. H. Leasburg
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

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Dear Mr. Leasburg:

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. NPF-4 for the North Anna Power Station, Unit No. 1. The amendment revises the NA-1 Technical Specifications in response to your letter dated June 25, 1982 (Serial No. 373) and in our discussions with you regarding this matter. The amendment is effective as of its date of issuance.

The changes as requested in your June 25, 1982 request would upgrade and add twenty (20) containment isolation valves to Table 3.6-1 for NA-1 in order to meet the requirements of NUREG-0737, Action Items II.B.2 and II.B.3 for Post-Accident Shielding and Post Accident Sampling, respectively. In addition, eight (8) isolation valves would be added to Table 3.6-1 for modifications completed to the Interior Fire Protection System and the Steam Generator Wet Layup Circulation System.

The additions to Table 3.6-1 include two (2) Phase A direct-acting solenoid valves, two (2) Phase A air-operated trip valves, sixteen (16) remote-manual operated valves, seven (7) manual isolation valves, and one (1) check valve.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Leon B. Engle, Project Manager
Operating Reactors Branch #3
Division of Licensing

8211110415 821020
PDR ADDCK 0500033B
P PDR

Enclosures:

1. Amendment No. 43 to NPF-4
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

DL - no legal objection to and on notice

OFFICE	ORB#3:DL	ORB#3:DL	ORB#3:DL	AD:OP:DL	OELD		
SURNAME	PMKreutzer	LBEngle/pr	RAClark	GCLamas	D SWANSON		
DATE	10/7/82	10/7/82	10/7/82	10/8/82	10/18/82		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

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PMKreutzer

Docket No. 50-338

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: VIRGINIA ELECTRIC AND POWER COMPANY, North Anna Power
Station, Unit No. 1

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Other: Amendment No. 43.
Referenced documents have been provided PDR.

Division of Licensing
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE →	ORB#3:DL					
SURNAME →	PMKreutzer/pr					
DATE →	10/20/82					

Virginia Electric and Power Company

cc:

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Mrs. June Allen
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U.S. Environmental Protection Agency
Region III Office
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6th and Walnut Streets
Philadelphia, Pennsylvania 19106

Mr. Paul W. Purdom
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Atomic Safety and Licensing
Appeal Board Panel
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Regional Administrator
Nuclear Regulatory Commission, Region II
Office of Executive Director for Operations
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 43
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated June 25, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 43, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 20, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

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TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
35. TV-LM100A	Reactor Containment Leakage Monitoring Lines to Open Pressure Taps	60
36. TV-LM100B	Reactor Containment Leakage Monitoring Lines to Open Pressure Taps	60
37. TV-LM100C	Reactor Containment Leakage Monitoring Lines to Open Pressure Taps	60
38. TV-LM100D	Reactor Containment Leakage Monitoring Lines to Open Pressure Taps	60
39. TV-LM100E	Reactor Containment Leakage Monitoring Lines to Open Pressure Taps	60
40. TV-LM100F	Reactor Containment Leakage Monitoring Lines to Open Pressure Taps	60
41. TV-LM100G	Reactor Containment Leakage Monitoring Lines to Open Pressure Taps	60
42. TV-LM100H	Reactor Containment Leakage Monitoring Lines to Open Pressure Taps	60
43. TV-SS101A	Pressurizer Vapor Space Sample	60
44. TV-SS101B	Pressurizer Vapor Space Sample	60
45. TV-SV102-1	Condenser Air Ejector Vent	60
46. TV-SV103	Condenser Air Ejector Vent	60
47. TV-CV150A	Containment Vacuum Pump Suction	60
48. TV-CV150B	Containment Vacuum Pump Suction	60
49. TV-CV150C	Containment Vacuum Pump Suction	60

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
50.	TV-CV150D Containment Vacuum Pump Suction	60
51.	TV-SS103A Residual Heat Removal System Sample Lines	60
52.	TV-SS103B Residual Heat Removal System Sample Lines	60
53.	TV-LM101A Reactor Containment Leakage Monitoring Lines to Reference System	60
54.	TV-LM101B Reactor Containment Leakage Monitoring Lines to Reference System	60
55.	TV-LM101C Reactor Containment Leakage Monitoring Lines to Reference System	60
56.	TV-LM101D Reactor Containment Leakage Monitoring Lines to Reference System	60
57.	TV-1859 Safety Injection Test Line	60
58.	TV-1842 Safety Injection Test Line	60
59.	TV-SS112A Steam Generator Surface Sample	60
60.	TV-SS112B Steam Generator Surface Sample	60
61.	TV-MS109# Main Steam Drains to Condenser	60
62.	TV-MS110# Main Steam to Blowdown	60
63.	TV-SV102-2 Condenser Air Ejector Vent	60
64.	FCV-AS100A# Condenser Air Ejector Steam Supply	60
65.	FCV-AS100B# Condenser Air Ejector Steam Supply	60
66.	TV-DA103A Post Accident Sample System Containment Return Line	60
67.	TV-DA103B Post Accident Sample System Containment Return Line	60

NORTH ANNA - UNIT 1

3/4 6-20

Amendment No. 43

TABLE 3.6-1 (Cont.)

VALVE NUMBER	FUNCTION	ISOLATION TIME (SEC.)
B. PHASE "B" ISOLATION		
1. TV-CC103A	Component Cooling Water From RHR System and Excess Letdown Heat Exchanger	60
2. TV-CC103B	Component Cooling Water From RHR System and Excess Letdown Heat Exchanger	60
3. TV-CC101A	Reactor Coolant Pump Thermal Barrier Cooling Water Return	60
4. TV-CC101B	Reactor Coolant Pump Thermal Barrier Cooling Water Return	60
5. TV-CC100A	Component Cooling Water From Containment Air Recirculation Coils	60
6. TV-CC100B	Component Cooling Water From Containment Air Recirculation Coils	60
7. TV-CC100C	Component Cooling Water From Containment Air Recirculation Coils	60
8. TV-CC105A	Component Cooling Water From Containment Air Recirculation Coils	60
9. TV-CC105B	Component Cooling Water From Containment Air Recirculation Coils	60
10. TV-CC105C	Component Cooling Water From Containment Air Recirculation Coils	60
11. TV-CC104A	Reactor Coolant Pumps, Cooling Water In	60
12. TV-CC104B	Reactor Coolant Pumps, Cooling Water In	60
13. TV-CC104C	Reactor Soolant Pumps, Cooling Water In	60

NORTH ANNA - UNIT 1

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Amendment No. 43

NORTH ANNA - UNIT 1

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Amendment No. 76,

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
14. TV-CC102A	Reactor Coolant Pumps, Bearing and Shroud, Cooling Water Out	60
15. TV-CC102B	Reactor Coolant Pumps, Bearing and Shroud, Cooling Water Out	60
16. TV-CC102C	Reactor Coolant Pumps, Bearing and Shroud, Cooling Water Out	60
17. TV-CC102D	Reactor Coolant Pumps, Bearing and Shroud, Cooling Water Out	60
18. TV-CC102E	Reactor Coolant Pumps, Cooling Water Out	60
19. TV-CC102F	Reactor Coolant Pumps, Cooling Water Out	60
20. TV-BD100A	Steam Generator Blowdown	60
21. TV-BD100B	Steam Generator Blowdown	60
22. TV-BD100C	Steam Generator Blowdown	60
23. TV-BD100D	Steam Generator Blowdown	60
24. TV-BD100E	Steam Generator Blowdown	60
25. TV-BD100F	Steam Generator Blowdown	60
26. TV-IA102B	Instrument Air to Reactor Containment	60

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
C. CONTAINMENT PURGE AND EXHAUST (VENTILATION DUCTS)		
1. MOV-HV100A*	Purge Supply	NA
2. MOV-HV100B*	Purge Supply	NA
3. MOV-HV102*	Alternate Supply	NA
4. MOV-HV100C*	Purge Exhaust	NA
5. MOV-HV100D*	Purge Exhaust	NA
6. MOV-HV101*	Bypass	NA

NORTH ANNA - UNIT 1

3/4 6-23

Amendment No. 43

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
D. MANUAL		
1. 1-SI-58*	Safety Injection Accumulator Make Up	NA
2. 1-RH-36*	Residual Heat Removal System to Refueling Water Storage Tank	NA
3. 1-RH-37*	Residual Heat Removal System to Refueling Water Storage Tank	NA
4. Deleted		
5. Deleted		
6. Deleted		
7. Deleted		
8. 1-DA-39*	Primary Vent Pot Vent	NA
9. 1-DA-41*	Primary Vent Pot Vent	NA
10. 1-CH-310#*	Reactor Coolant Pump Seal Water Supply	NA
11. 1-CH-318#*	Reactor Coolant Pump Seal Water Supply	NA
12. 1-CH-314#*	Reactor Coolant Pump Seal Water Supply	NA
13. 1-SA-29*	Service Air	NA
14. 1-SA-2*	Service Air	NA
15. Deleted		
16. NA*	Fuel Transfer (Tube Penetration #65)	NA

NORTH ANNA - UNIT 1

3/4 6-24

Amendment No. 43

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
17. 1-CV-4*	Air Ejector Suction	NA
18. 1-RC-176*	Dead Weight Pressure Calibrator	NA
19. 1-RC-178*	Dead Weight Pressure Calibrator	NA
20. 1-RP-26*	Refueling Purification Inlet	NA
21. 1-RP-28*	Refueling Purification Inlet	NA
22. 1-RP-6*	Refueling Purification Inlet	NA
23. 1-RP-8*	Refueling Purification Inlet	NA
24. 1-WT-354#*	Chemical Feed Lines	NA
25. 1-WT-357#*	Chemical Feed Lines	NA
26. 1-WT-351#*	Chemical Feed Lines	NA
27. 1-FP-274*	Fire Protection Hose Standpipe	NA
28. 1-WT-465*	Steam Generator Wet Layup	NA
29. 1-WT-468*	Steam Generator Wet Layup	NA
30. 1-WT-488*	Steam Generator Wet Layup	NA
31. 1-WT-491*	Steam Generator Wet Layup	NA
32. 1-WT-511*	Steam Generator Wet Layup	NA
33. 1-WT-514*	Steam Generator Wet Layup	NA

NORTH ANNA - UNIT 1

3/4 6-25

Amendment No. 43

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
E. REMOTE MANUAL		
1. MOV-QS101A*	Quench Spray Pump Discharge	NA
2. MOV-QS101B*	Quench Spray Pump Discharge	NA
3. MOV-RS155A#*	Recirculation Spray Pump Suction	NA
4. MOV-RS155B#*	Recirculation Spray Pump Suction	NA
5. MOV-1860A#*	LHSI Pump Suction From Containment Sump	NA
6. MOV-1860B#*	LHSI Pump Suction From Containment Sump	NA
7. MOV-RS156A*	Recirculation Spray Pump Discharge	NA
8. MOV-RS156B*	Recirculation Spray Pump Discharge	NA
9. MOV-SW103A*	Service Water to Recirculation Spray Coolers	NA
10. MOV-SW103B*	Service Water to Recirculation Spray Coolers	NA
11. MOV-SW103C*	Service Water to Recirculation Spray Coolers	NA
12. MOV-SW103D*	Service Water to Recirculation Spray Coolers	NA
13. MOV-SW104A*	Service Water to Recirculation Spray Coolers	NA
14. MOV-SW104B*	Service Water to Recirculation Spray Coolers	NA
15. MOV-SW104C*	Service Water to Recirculation Spray Coolers	NA
16. MOV-SW104D*	Service Water to Recirculation Spray Coolers	NA
17. TV-CV100*	Containment Air Ejector Suction	NA
18. MOV-1869A*	High Head Safety Injection to RCS Except Boron Injection Line	NA

NORTH ANNA - UNIT 1

3/4 6-25

Amendment No. 43

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
19. MOV-1836*	High Head Safety Injection to RCS Except Boron Injection Line	NA
20. MOV-1869B*	High Head Safety Injection to RCS Except Boron Injection Line	NA
21. HCV-1142*	Reactor Coolant Letdown Line From RHR System	NA
22. Deleted		
23. Deleted		
24. MOV-1890A*	LHSI Pump Discharge to Reactor Coolant System Hot Legs	NA
25. MOV-1890B*	LHSI Pump Discharge to Reactor Coolant System Hot Legs	NA
26. MOV-1890C*	LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA
27. MOV-1890D*	LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA
28. FCV-1160*	Loop Fill Header	NA
29. MOV-1289A*	Charging Line	NA
30. MOV-1867C*	High Head Safety Injection, Boron Injection Tank	NA
31. MOV-1867D*	High Head Safety Injection, Boron Injection Tank	NA
32. MOV-RS-100A*	Casing Cooling to Outside Recirculation Spray Pump	NA
33. MOV-RS-100B*	Casing Cooling to Outside Recirculation Spray Pump	NA
34. MOV-RS-101A*	Casing Cooling to Outside Recirculation Spray Pump	NA

NORTH ANNA - UNIT 1

3/4 6-27

Amendment No. 43

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
35. MOV-RS-101B*	Casing Cooling to Outside Recirculation Spray Pump	NA
36. TV-HC-108A*	Containment Atmosphere Sample Line	NA
37. TV-HC-108B*	Containment Atmosphere Sample Line	NA
38. TV-HC-100A	Suction Hydrogen Analyzer	NA
39. TV-HC-100B	Suction Hydrogen Analyzer	NA
40. TV-HC-101A	Discharge Hydrogen Analyzer	NA
41. TV-HC-101B	Discharge Hydrogen Analyzer	NA
42. TV-HC-102A	Suction Hydrogen Analyzer	NA
43. TV-HC-102B	Suction Hydrogen Analyzer	NA
44. TV-HC-103A	Discharge Hydrogen Analyzer	NA
45. TV-HC-103B	Discharge Hydrogen Analyzer	NA
46. TV-HC-104A	Suction Hydrogen Recombiner	NA
47. TV-HC-104B	Suction Hydrogen Recombiner	NA
48. TV-HC-105A	Discharge Hydrogen Recombiner	NA
49. TV-HC-105B	Discharge Hydrogen Recombiner	NA
50. TV-HC-106A	Suction Hydrogen Recombiner	NA
51. TV-HC-106B	Suction Hydrogen Recombiner	NA
52. TV-HC-107A	Discharge Hydrogen Recombiner	NA
53. TV-HC-107B	Discharge Hydrogen Recombiner	NA

NORTH ANNA - UNIT 1

3/4 6-28

Amendment No. 38, 43

TABLE 3.6-1 (Cont.)

VALVE NUMBER	FUNCTION	ISOLATION TIME (SEC.)
F. CHECK		
1. 1-CC-193	Component Cooling Water to RHR System and Excess Letdown Heat Exchanger	NA
2. 1-CC-198	Component Cooling Water to RHR System and Excess Letdown Heat Exchanger	NA
3. 1-SI-79	High Head Safety Injection, Boron Injection to RCS	NA
4. 1-CC-572	Component Cooling Water to Containment Air Recirculation Coils	NA
5. 1-CC-559	Component Cooling Water to Containment Air Recirculation Coils	NA
6. 1-CC-546	Component Cooling Water to Containment Air Recirculation Coils	NA
7. 1-CH-322	Charging Line	NA
8. 1-CC-154	Component Cooling Water to Reactor Coolant Pumps	NA
9. 1-CC-119	Component Cooling Water to Reactor Coolant Pumps	NA
10. 1-CC-84	Component Cooling Water to Reactor Coolant Pumps	NA
11. 1-CH-402	Reactor Coolant Pumps, Seal Water Return	NA
12. 1-SI-110	Safety Injection Accumulator Make Up	NA
13. 1-SI-185	High Head Safety Injection to RCS Except Boron Injection Line	NA
14. 1-HC-18	Discharge From Containment Atmosphere Cleanup System (Hydrogen Recombiner)	NA
15. 1-HC-14	Discharge From Containment Atmosphere Cleanup System (Hydrogen Recombiner)	NA
16. 1-CH-380#	Reactor Coolant Pump Seal Water Supply	NA
17. 1-CH-336#	Reactor Coolant Pump Seal Water Supply	NA
18. 1-CH-358#	Reactor Coolant Pump Seal Water Supply	NA
19. 1-IA-149	Air Radiation Monitor Return	NA
20. 1-RC-149	Primary Grade Water to Pressurizer Relief Tank	NA
21. 1-CH-330	Loop Fill Header	NA

NORTH ANNA - UNIT 1

3/4 6-29

Amendment No. 16, 43

TABLE 3.6-1 (Cont.)

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>ISOLATION TIME (SEC.)</u>
22.	1-IA-55 Instrument Air Line	NA
23.	1-SI-106 Nitrogen to Pressurizer Relief Tank and SI Accumulators	NA
24.	1-SI-206 LHSI Pump Discharge to Reactor Coolant System Hot Legs	NA
25.	1-SI-207 LHSI Pump Discharge to Reactor Coolant System Hot Legs	NA
26.	1-SI-195 LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA
27.	1-SI-197 LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA
28.	1-SI-199 LHSI Pump Discharge to Reactor Coolant System Cold Legs	NA
29.	1-QS-19** Quench Spray Pump Discharge	NA
30.	1-QS-11** Quench Spray Pump Discharge	NA
31.	1-RS-27** Recirculation Spray Pump Discharge	NA
32.	1-RS-18** Recirculation Spray Pump Discharge	NA
33.	1-VP-12 Air Ejector Vent	NA
34.	1-SI-90 High Head Safety Injection to RCS Except Boron Injection Line	NA
35.	1-SI-201 High Head Safety Injection to RCS Except Boron Injection Line	NA
36.	1-SI-85 High Head Safety Injection to RCS Except Boron Injection Line	NA
37.	1-FW-47# Feedwater to Steam Generators	NA
38.	1-FW-111# Feedwater to Steam Generators	NA
39.	1-FW-79# Feedwater to Steam Generators	NA
40.	1-WT-50# Chemical Feed Lines	NA
41.	1-WT-66# Chemical Feed Lines	NA
42.	1-WT-38# Chemical Feed Lines	NA
43.	1-FW-68# Auxiliary Feedwater to Steam Generator	NA
44.	1-FW-100# Auxiliary Feedwater to Steam Generator	NA
45.	1-FW-132# Auxiliary Feedwater to Steam Generator	NA
46.	1-FP-272# Fire Protection Hose Standpipe	NA

NORTH ANNA - UNIT 1

3/4 6-30

Amendment No. 76, 43



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 43 TO FACILITY OPERATING LICENSE NO. NPF-4

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION, UNIT NO. 1

DOCKET NO. 50-338

Introduction:

By letter dated June 25, 1982 (Serial No. 373), the Virginia Electric and Power Company (the licensee) requested a change to the Technical Specifications (TS) for the North Anna Power Station, Unit No. 1 (NA-1).

The requested change would add and delete certain containment isolation valves to improve post-accident sampling activities. Also, presently installed manual isolation valves would be replaced with remote operated isolation valves for post-accident containment hydrogen analysis and control equipment. In addition, one manual isolation valve and one check valve have been added to provide increased isolation for the Interior Fire Protection System. And finally, six manual isolation valves have been added to provide forced circulation capabilities for the Steam Generator Wet Layup Circulation System.

A discussion and our evaluation and conclusion regarding the licensee's requested change are provided below.

Discussion:

The proposed change would revise the NA-1 TS 3/4.6.3.1, Table 3.6-1 to reflect the addition and deletion of containment isolation valves.

The addition of two (2) direct-acting solenoid valves (TV-SS103A and TV-SS103B) will be used to replace two (2) air-operated trip valves (TV-SS107A and TV-SS107B) in the Residual Heat Removal System Sample Lines. The double isolation direct-acting solenoid valves are being installed to provide increased assurance of reliable operation during accident conditions. The valves will be normally closed and receive a Phase A signal to assure they are tripped closed on a safety injection signal. These modifications are required to meet the provisions of NUREG-0737, II.B.3, Post-Accident Sampling.

Two air-operated Phase A trip valves (TV-DA103A and TV-DA103B) are being installed on the Post-Accident Sampling System return lines. These valves will reduce radiation levels outside containment should post-accident samples be required to be withdrawn from the reactor coolant system and containment sump. These modifications are required to meet the provisions of NUREG-0737 Item II.B.3, Post-Accident Sampling.

Four (4) manual isolation valves (1-HC-12, 1-HC-31, 1-HC-16, and 1-HC-28) in the hydrogen-recombiner and analyzer system are being replaced with sixteen (16) remote-manual valves (HC-series valves) to upgrade the hydrogen re-combiner and analyzer system. The addition of these remote-manual isolation valves will reduce radiation levels outside the containment should the hydrogen analyzer and recombiner be required to be in service for post-accident conditions.

Eight of these valves are in the suction and return lines for the hydrogen analyzers. These remote-manual valves are TV-HC-100 A&B (in series), TV-HC-101 A&B (in series), TV-HC-102 A&B (in series) and TV-HC-103 A&B (in series).

Eight of these valves are in the suction and discharge lines for the hydrogen recombiners. These remote-manual valves are TV-HC-104 A&B (in series) TV-HC-105 A&B (in series), TV-HC-106 A&B (in series) and TV-HC-107 A&B (in series).

The above modifications are required to meet the provisions of NUREG-0737, Item II.B.2, Post-Accident Shielding.

The Fire Hose Rack Standpipe System is an extension of the Interior Fire Protection System. The fire hose rack standpipe is routed through the Auxiliary Building to penetration no. 34 of Unit 1 Containment. Containment isolation is provided by one manual isolation valve (1-FP-274) and one check valve (1-FP-272).

The Steam Generator Wet Layup Circulation (SGWLC) system is being implemented at NA-1 to augment secondary chemistry control by adding forced circulation capabilities to each of the three steam generators. Each steam generator will have a new two (2) inch penetration installed through the upper shell wall with piping running to existing spare containment penetrations. Return flow to each steam generator will be provided through existing steam generator blowdown lines. The installation of a new piping line from each of the three steam generator shells to a containment penetration requires the installation of six manual isolation valves (1-WT-465, 1-WT-468, 1-WT-488, 1-WT-491, 1-WT-511 and 1-WT-514).

The installation of the SGWLC system for NA-2 was completed prior to initial power operations and the listing and inclusion of the appropriate SGWLC system isolation valves was included in the NA-2 TS (Table 3.6-1) at the time NA-2 was issued a full power license on August 20, 1980.

The Fire Hose Rack Standpipe System is an extension of the Interior Fire Protection System. The fire hose rack standpipe piping is routed through the Auxiliary Building to penetration no. 34 of the NA-1 Containment. Containment isolation is provided by one manual isolation valve (1-FP-274) and one check valve (1-FP-272). These modifications and listing of the appropriate isolation valves in the NA-2 TS (Table 3.6-1) were completed prior to issuance of the full power license for NA-2 on August 20, 1980.

Evaluation:

The upgrading and installation of the above containment isolation valves meets the requirements for Category 1 Containment Isolation Valves specified in the NA-1&2 Final Safety Analysis Report (FSAR). Double barrier protection is provided by two (2) valves to assure that no single failure will result in the loss of containment integrity. Containment penetration piping including the isolation valves is designed to Seismic Category I Requirements.

As stated above, isolation valves TV-SS103A, TV-SS103B, TV-DA103A and TV-DA103B are normally closed and receive a Phase A signal to assure they are tripped closed on a safety injection signal. Maximum isolation time for these valves is specified to be 60 seconds. We have already reviewed Phase A isolation as specified in the NA-1&2 FSAR and found it to be acceptable as well as a maximum closure time of 60 seconds for containment isolation.

The remote-manual valves (HC series) being added for the post-accident containment hydrogen and control equipment and numbered 100 A&B through 107 A&B (sixteen valves in all) will be closed at all times and can be opened only upon remote-manual activation from the control room. Opening of these valves will take place only under specific administrative control as specified in post-accident procedures.

Based on the above, we find the licensee's request to add the above specified valves to the NA-1 TS 3/4.6.3.1, Table 3.6-1, to be acceptable. Also, these valves are necessary to meet the provisions of NUREG-0737, Item II.B.2, Post-Accident Shielding and Item II.B.3, Post-Accident Sampling.

The addition of seven manual isolation and one check valve required to complete the NA-1 modifications for the SGWLC system and the Interior Fire Protection system have already been reviewed and approved for NA-2. Therefore, we find the licensee's request to add the above specified valves to the NA-1 TS 3/4.6.3.1, Table 3.6-1, to be acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 20, 1982

Principal Contributor:

Leon B. Engle

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-338VIRGINIA ELECTRIC AND POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 43 to Facility Operating License No. NPF-4 issued to the Virginia Electric and Power Company (the licensee) for operation of the North Anna Power Station, Unit No. 1 (the facility) located in Louisa County, Virginia. The amendment is effective as of its date of issuance.

The amendment revises the NA-1 Technical Specifications by upgrading and adding twenty (20) containment isolation valves to Table 3.6-1 to meet the requirements of NUREG-0737, Action Item II.B.2, Post-Accident Shielding, and Action Item II.B.3, Post-Accident Sampling. In addition, eight (8) isolation valves have been added to Table 3.6-1 for upgrading modifications recently completed for the Interior Fire Protection System and the Steam Generator Wet Layup Circulation System.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

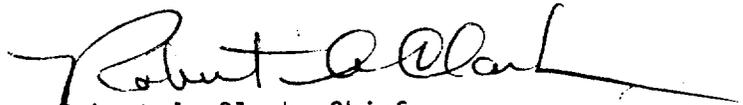
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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated June 25, 1982; (2) Amendment No. 43 to Facility Operating License No. NPF-4; and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. 20555 and at the Board of Supervisors Office, Louisa County Courthouse, Louisa, Virginia 23093 and at the Alderman Library, Manuscripts Department, University of Virginia, Charlottesville, Virginia 22901. A copy of items (2) and (3) may be obtained upon request to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 20th day of October, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing