

April 25, 2002

Mr. Ross T. Ridenoure
Division Manager - Nuclear Operations
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 550
Hwy. 75 - North of Fort Calhoun
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 - ISSUANCE OF AMENDMENT
(TAC NO. MB3448)

Dear Mr. Ridenoure:

The Commission has issued the enclosed Amendment No. 208 to Facility Operating License No. DPR-40 for the Fort Calhoun Station, Unit No. 1. The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated November 21, 2001, as supplemented on February 13, 2002.

The amendment reformats and revises TSs 2.15(5) and (6), "Instrumentation and Control Systems." The new TSs clarify the scope of the alternate shutdown panels (ASPs). The change resulted from a corrective action needed to address the regulatory requirements for the ASPs and the associated auxiliary feedwater panel as documented in Licensee Event Report 97-002, Revision 0, dated May 14, 1997.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Alan B. Wang, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosures: 1. Amendment No. 208 to DPR-40
2. Safety Evaluation

cc w/encls: See next page

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Ft. Calhoun Station, Unit 1

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OMAHA PUBLIC POWER DISTRICT

DOCKET NO. 50-285

FORT CALHOUN STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.208
License No. DPR-40

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Omaha Public Power District (the licensee) dated November 21, 2001, as supplemented by letter dated February 13, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-40 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-40 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 208, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Stephen Dembek, Chief, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: April 25, 2002

ATTACHMENT TO LICENSE AMENDMENT NO. 208

FACILITY OPERATING LICENSE NO. DPR-40

DOCKET NO. 50-285

Replace the following pages of Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

REMOVE

2-65
2-66
2-66a
2-66b
2-66c

INSERT

2-65
2-66
2-66a
2-66b
2-66c
2-66d

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 208 TO FACILITY OPERATING LICENSE NO. DPR-40
OMAHA PUBLIC POWER DISTRICT
FORT CALHOUN STATION, UNIT NO. 1
DOCKET NO. 50-285

1.0 INTRODUCTION

By application dated November 21, 2001, as supplemented by letter dated February 13, 2002, Omaha Public Power District (OPPD) requested changes to the Technical Specifications (Appendix A to Facility Operating License No. DPR-40) for the Fort Calhoun Station, Unit No. 1. The requested changes would reformat and revise Technical Specifications (TSs) 2.15(5) and (6), "Instrumentation and Control Systems." The new TSs will clarify the scope of the alternate shutdown panels (ASPs). The change resulted from a corrective action needed to address the regulatory requirements for the ASPs and the associated auxiliary feedwater (AFW) panel, as documented in License Event Report (LER) 97-002, Revision 0, dated May 14, 1997.

The supplemental letter dated February 13, 2002, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on December 26, 2001 (66 FR 66470).

2.0 BACKGROUND

Due to the questions raised by the OPPD Operations Staff in December 1996, the Operations Department requested the OPPD Licensing Department to determine if the seven day limiting conditions for operation (LCO) of TSs 2.15(5) or (6) should be entered if charging pump CH-1B is inoperable, since its control circuit is on the ASP. After researching the issue, a Condition Report (CR) was originated. This CR documented that TSs 2.15(5) and (6) were inadequate. The CR concluded that while the TSs list the requirements for the instrumentation on the ASP, they do not adequately address the controls for the equipment on the ASP. Specifically, CH-1B control function is not listed, nor is AFW control function listed. The CR also noted that in response to concerns expressed by the NRC in the introduction to Amendment 125, dated March 19, 1990, the words "Control Circuits," were added to TSs 2.15(5) and (6), but the applicable control circuits were not specifically listed in the TSs. This event was submitted to the NRC on May 14, 1997, as LER 97-002, Revision 0.

In LER 97-002, Revision 0, OPPD identified a corrective action needed to address regulatory requirements for the ASPs and the associated AFW panel. In particular, the current TSs

2.15(5) and (6) needed to address the requirements for the control functions for charging pump flow or AFW flow. Currently, only the indications for wide range logarithmic power, source range power, reactor coolant cold leg temperature, reactor coolant hot leg temperature, pressurizer level, volume control tank level, steam generator narrow range level, steam generator pressure, and pressurizer pressure are addressed.

3.0 EVALUATION

The proposed revision to TSs 2.15(5) and (6) will clarify the scope of the ASPs. The changes will identify: (1) all indication and control functions required for the alternate (remote) shutdown panels; (2) panel locations of the functions; and (3) the number of channels required. The LCO (operability and shutdown requirements) for the ASPs and the AFW panel are not being changed and the applicable modes are not being changed. However, TSs 2.15(5) and (6) are being combined.

The original design of ASP AI-185 contained one reactor coolant system (RCS) hot leg temperature indication (TI-121H) and one RCS cold leg temperature indication (TI-121C-1), both associated with loop 1. Both of these were narrow indications with a temperature range from 515 - 615°F. During the 1998 refueling outage, modifications were made to replace the narrow range with wide range instrumentation on loop 1 and to add wide range instrumentation to loop 2. Both steam generator (SG) loops now have wide range (50 - 700°F) hot and cold leg RCS temperature indication. However, by letter dated February 13, 2002, OPPD modified the TS change to state that the TS will require that the hot and cold leg instruments must be operable in one loop. This is different from the November 21, 2001, request which required a hot and cold leg indication from either loop. The requirement for only one loop of instrumentation was previously reviewed and approved by the staff and is consistent with the current TS.

TSs 2.15(5) and (6) are being combined into proposed TS Section 2.15(5), which is patterned after the Improved Technical Specifications, NUREG-1432, "Standard Technical Specifications, Combustion Engineering Plants." This provides a tabular format of function/instrumentation or control parameters, location, and the required number of operable channels. The LCO (operability and shutdown requirements) for the ASPs and AFW panel have not been changed and the applicable modes have not been changed. The change is more restrictive as it provides additional requirements to the TS relating to the control functions for the charging pump flow and AFW flow. Table 3-3A addresses surveillance for all the equipment located on the panels including the surveillances for the ASP and the AFW control circuits. The surveillances are accomplished by procedure OP-ST-ASP-2000, "Alternate Shutdown Capability Control Circuitry Verification." The proposed changes do not affect Table 3-3A in Section 3.0 of the TSs pertaining to the surveillance requirements for the APSs (AI-185 and AI-212) and the AFW panel (AI-179). Therefore, the format change does not delete any requirements or functions from the current TS. The change is more restrictive than the current TS and is consistent with the regulatory requirements for the ASPs and the associated AFW panel and therefore, acceptable. In addition, several TS pages were reformatted due to the addition of elements to TS section 2.15(5). No changes were made to the text itself.

4.0 STATE CONSULTATION

In accordance with the Commission regulations, the Nebraska state official was notified of the proposed issuance of the amendment. The state official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (66 FR 66470). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A Wang

Date: April 25, 2002