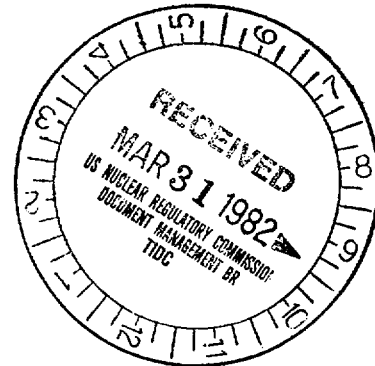


Docket File
DCS MS-016

MAR 25 1982

Docket Nos. 50-338
and 50-339

Mr. R. H. Leasburg
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261



Dear Mr. Leasburg:

The Commission has issued the enclosed Amendment Nos. 3 and 19 to Facility Operating License Nos. NPF-4 and NPF-7 for North Anna Power Station, Unit Nos. 1 and 2 (NA-1&2). The amendments consist of changes to the Technical Specifications (TS) as requested in your applications transmitted by letters dated March 1, 1982 (Serial No. 095), March 2, 1982 (Serial No. 132) and in our discussions with you regarding these matters.

The changes as requested in your March 1, 1982 submittal would add two containment isolation valves to Table 3.6.1 for both NA-1&2 in order to meet the requirements of NUREG-0737, Action Item II.B.3, which requires a dedicated containment penetration for Post Accident Sampling of the containment atmosphere. The installation of these valves will be implemented during the present refueling outage for NA-2 (March 5, 1982 to May 14, 1982) and the forthcoming refueling outage for NA-1 presently scheduled from May 21, 1982 to July 2, 1982.

Also, as part of the NA-2 amendment, you will find our followup actions which support our letter to you dated March 3, 1982, wherein we confirmed our telephone authorization to you on March 3, 1982 regarding your March 2, request for a one-time relief from the requirements of the NA-2 TS. The one-time relief from the requirements of the NA-2 TS allowed you to perform the required tests for boron mixing and cooldown under natural circulation conditions specified in License Conditions 2.C(15)(f) and 2.C(20)(b) for NA-2.

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| OFFICE | | | | | | | |
| SURNAME | | | | | | | |
| DATE | | | | | | | |

Copies of the related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:

Leon B. Engle, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:

1. Amendment No. 38 to NPF-4
2. Amendment No. 19 to NPF-7
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

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| DATE | 3/19/82 | 3/19/82 | 3/19/82 | 3/21/82 | 3/22/82 | |

*no legal objection to notice or
order, subject to changes or
agreed to by L.E. & marked
on cover letter & notice.*



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

DISTRIBUTION:
Docket File
ORB#3 Rdg
PMKreutzer

Docket No. 50-338/339

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: VIRGINIA ELECTRIC AND POWER COMPANY, North Anna Power Station,
Unit Nos. 1 and 2

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

- ☐ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- ☐ Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- ☐ Notice of Availability of Applicant's Environmental Report.
- ☐ Notice of Proposed Issuance of Amendment to Facility Operating License.
- ☐ Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- ☐ Notice of Availability of NRC Draft/Final Environmental Statement.
- ☐ Notice of Limited Work Authorization.
- ☐ Notice of Availability of Safety Evaluation Report.
- ☐ Notice of Issuance of Construction Permit(s).
- ☐ Notice of Issuance of Facility Operating License(s) or Amendment(s).
- ☒ Other: Amendment Nos. 38 and 19.
Referenced documents have been provided PDR.

Division of Licensing
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

| | | | | | | |
|---------|---------------|--|--|--|--|--|
| OFFICE | ORB#3:DL | | | | | |
| SURNAME | PMKreutzer/pr | | | | | |
| DATE | 3/29/82 | | | | | |

Virginia Electric and Power Company

cc:

Richard M. Foster, Esquire
Musick, Williamson, Schwartz,
Leavenworth & Cope, P.C.
P. O. Box 4579
Boulder, Colorado 80306

Michael W. Maupin, Esquire
Hunton, Williams, Gay and Gibson
P. O. Box 1535
Richmond, Virginia 23212

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Louisa, Virginia 23093

Ellyn R. Weiss, Esquire
Sheldon, Harman, Roisman and Weiss
1725 I Street, N.W. Suite 506
Washington, D. C. 20006

Mr. W. R. Cartwright, Station Manager
P. O. Box 402
Mineral, Virginia 23117

Mr. Anthony Gambardella
Office of the Attorney General
11 South 12th Street - Room 308
Richmond, Virginia 23219

Resident Inspector/North Anna
c/o U.S.N.R.C.
Route 2, Box 78A
Mineral, Virginia 23117

Mr. J. H. Ferguson
Executive Vice President - Power
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Mr. James Torson
501 Leroy
Socorro, New Mexico 87891

Mrs. Margaret Dietrich
Route 2, Box 568
Gordonsville, Virginia 22042

Mr. James C. Dunstance
State Corporation Commission
Commonwealth of Virginia
Blandon Building
Richmond, Virginia 23209

Mrs. June Allen
North Anna Environmental Coalition
8720 Lockmoor Circle
Wichita, Kansas 67207

U.S. Environmental Protection Agency
Region III Office
ATTN: Regional Radiation Representative
Curtis Building
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

Mr. Paul W. Purdom
Environmental Studies Institute
Drexel University
32nd and Chestnut Streets
Philadelphia, Pennsylvania 19104

Atomic Safety and Licensing
Appeal Board Panel
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Regional Administrator
Nuclear Regulatory Commission, Region II
Office of Executive Director for Operations
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 38
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Virginia Electric and Power Company (the licensee) dated March 1 and March 2, 1982, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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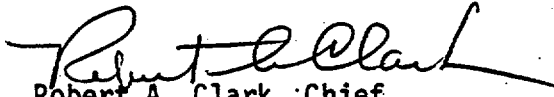
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 38, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 25, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Pages

3/4 6-27a

TABLE 3.6-1 (Cont.)

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>ISOLATION TIME (SEC.)</u> |
|-------------------------|--|----------------------------------|
| 17. TV-CV100* | Containment Air Ejector Suction | NA |
| 18. MOV-1869A* | High Head Safety Injection to RCS Except Boron Injection Line | NA |
| 19. MOV-1836* | High Head Safety Injection to RCS Except Boron Injection Line | NA |
| 20. MOV-1869B* | High Head Safety Injection to RCS Except Boron Injection Line | NA |
| 21. HCV-1142* | Reactor Coolant Letdown Line From RHR System | NA |
| 22. TV-SS107A* | Residual Heat Removal System Sample Lines | NA |
| 23. TV-SS107B* | Residual Heat Removal System Sample Lines | NA |
| 24. MOV-1890A* | LHSI Pump Discharge to Reactor Coolant System Hot Legs | NA |
| 25. MOV-1890B* | LHSI Pump Discharge to Reactor Coolant System Hot Legs | NA |
| 26. MOV-1890C* | LHSI Pump Discharge to Reactor Coolant System Cold Legs | NA |
| 27. MOV-1890D* | LHSI Pump Discharge to Reactor Coolant System Cold Legs | NA |
| 28. FCV-1160* | Loop Fill Header | NA |
| 29. MOV-1289A* | Charging Line | NA |
| 30. MOV-1867C* | High Head Safety Injection, Boron Injection Tank | NA |
| 31. MOV-1867D* | High Head Safety Injection, Boron Injection Tank | NA |

TABLE 3.6-1 (Cont.)

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>ISOLATION TIME (SEC.)</u> |
|-------------------------|--|----------------------------------|
| 32. MOV-RS-100A* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 33. MOV-RS-100B* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 34. MOV-RS-101A* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 35. MOV-RS-101B* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 36. TV-HC-108A* | Containment Atmosphere Sample Line | NA |
| 37. TV-HC-108B* | Containment Atmosphere Sample Line | NA |

NORTH ANNA - UNIT 1

3/4 6-27a

Amendment No. 5, 38



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY
DOCKET NO. 50-339
NORTH ANNA POWER STATION, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 19
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Virginia Electric and Power Company (the licensee) dated March 1 and March 2, 1982, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

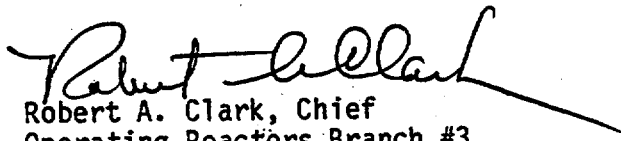
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 19, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 25, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 19 TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 4-2

3/4 6-27

3/4.4 REACTOR COOLANT SYSTEM

3/4.4.1 REACTOR COOLANT LOOPS AND COOLANT CIRCULATION

STARTUP AND POWER OPERATION

LIMITING CONDITION FOR OPERATION

3.4.1.1 All reactor coolant loops shall be in operation with power removed from the loop stop valve operators.

APPLICABILITY: MODES 1 and 2.*

ACTION:

With less than the above required reactor coolant loops in operation, be in at least HOT STANDBY within 1 hour.

SURVEILLANCE REQUIREMENTS

4.4.1.1 The above required reactor coolant loops shall be verified to be in operation and circulating reactor coolant at least once per 12 hours.

4.4.1.2 At least once per 31 days, with the reactor coolant loops in operation by verifying that the power is removed from the loop stop valve operators.

*See Special Test Exception 3.10.4.

REACTOR COOLANT SYSTEM

HOT STANDBY

LIMITING CONDITION FOR OPERATION

- 3.4.1.2 a. At least two of the reactor coolant loops listed below shall be OPERABLE:
1. Reactor Coolant Loop A and its associated steam generator and reactor coolant pump,
 2. Reactor Coolant Loop B and its associated steam generator and reactor coolant pump,
 3. Reactor Coolant Loop C and its associated steam generator and reactor coolant pump,
- b. At least one of the above coolant loops shall be in operation.* **

APPLICABILITY: MODE 3

ACTION:

- a. With less than the above required reactor coolant loops OPERABLE, restore the required loops to OPERABLE status within 72 hours or be in HOT SHUTDOWN within the next 12 hours.
- b. With no reactor coolant loop in operation, suspend all operations involving a reduction in boron concentration of the Reactor Coolant System and immediately initiate corrective action to return the required coolant loop to operation.

SURVEILLANCE REQUIREMENTS

4.4.1.2.1 At least the above required reactor coolant pumps, if not in operation, shall be determined to be OPERABLE once per 7 days by verifying correct breaker alignments and indicated power availability.

4.4.1.2.2 At least one cooling loop shall be verified to be in operation and circulating reactor coolant at least once per 12 hours.

* All reactor coolant pumps may be de-energized for up to 1 hour provided (1) no operations are permitted that would cause dilution of the reactor coolant system boron concentration, and (2) core outlet temperature is maintained at least 10°F below saturation temperature.

**The requirement to have one coolant loop in operation is exempted during the performance of the boron mixing tests as stipulated in License Conditions 2.C(15)(f) and 1.C(20)(b).

TABLE 3.6-1 (Cont.)

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-------------------------|--|--|
| 27. MOV-2890D* | LHSI Pump Discharge to Reactor Coolant System Cold Legs | NA |
| 28. FCV-2160* | Loop Fill Header | NA |
| 29. MOV-2289A* | Charging Line | NA |
| 30. MOV-2867C* | High Head Safety Injection, Boron Injection Tank | NA |
| 31. MOV-2867D* | High Head Safety Injection, Boron Injection Tank | NA |
| 32. MOV-RS-200A* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 33. MOV-RS-200B* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 34. MOV-RS-201A* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 35. MOV-RS-201B* | Casing Cooling to Outside Recirculation Spray Pump | NA |
| 36. TV-HC-208A* | Containment Atmosphere Sample Line | NA |
| 37. TV-HC-208B* | Containment Atmosphere Sample Line | NA |

F. CHECK

| | | |
|-------------|--|----|
| 1. 2-CC-194 | Component Cooling Water to RHR System and Excess Letdown Heat Exchanger | NA |
| 2. 2-CC-199 | Component Cooling Water to RHR System and Excess Letdown Heat Exchanger | NA |
| 3. 2-SI-93 | High Head Safety Injection, Boron Injection to RCS | NA |
| 4. 2-CC-302 | Component Cooling Water to Containment Air Recircu- lation Coils | NA |
| 5. 2-CC-289 | Component Cooling Water to Containment Air Recircu- lation Coils | NA |

TABLE 3.6-1 (Cont.)

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC.)</u> |
|-------------------------|--|--|
| 6. 2-CC-276 | Component Cooling Water to Containment Air Recirculation Coils | NA |
| 7. 2-CH-335 | Charging Line | NA |
| 8. 2-CC-152 | Component Cooling Water to Reactor Coolant Pumps | NA |
| 9. 2-CC-115 | Component Cooling Water to Reactor Coolant Pumps | NA |
| 10. 2-CC-78 | Component Cooling Water to Reactor Coolant Pumps | NA |
| 11. 2-CH-331 | Reactor Coolant Pumps, Seal Water Return | NA |
| 12. 2-SI-136 | Safety Injection Accumulator Make Up | NA |
| 13. 2-SI-85 | High Head Safety Injection to RCS except Boron Injection Line | NA |
| 14. 2-HC-20 | Discharge From Containment Atmosphere Clean-up System | NA |
| 15. 2-HC-15 | Discharge From Containment Atmosphere Clean-up System | NA |
| 16. 2-CH-308# | Reactor Coolant Pump Seal Water Supply | NA |
| 17. 2-CH-260# | Reactor Coolant Pump Seal Water Supply | NA |
| 18. 2-CH-284# | Reactor Coolant Pump Seal Water Supply | NA |
| 19. 2-IA-428 | Air Radiation Monitor Return | NA |
| 20. 2-RC-162 | Primary Grade Water | NA |
| 21. 2-CH-332 | Loop Fill Header | NA |
| 22. 2-IA-250 | Containment Instrument Air Return | NA |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. NPF-4
AND AMENDMENT NO. 19 TO FACILITY OPERATING LICENSE NO. NPF-7
VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-338 AND 50-339

Introduction:

By letter dated March 1, 1982 (Serial No. 095), the Virginia Electric and Power Company (the licensee) requested a change to the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). The change would revise the NA-1&2 TS Table 3.6.1, "Containment Isolation Valves," by adding containment isolation valves numbered 36 and 37 to meet the requirements of NUREG-0737, Item II.B.3, for the Post Accident Sampling of the containment atmosphere.

By letter dated March 2, 1982 (Serial No. 132), the licensee requested that the NA-2 TS be revised to allow all Reactor Coolant Pumps (RCP) to be de-energized in Mode 3 (Hot Standby) while conducting the boron mixing and natural circulation condition tests stipulated in License Condition 2.C(15)(f) and 2.C(20)(b). Our letter dated March 3, 1982 provided confirmation of our telephone authorization provided on March 3, 1982 to the licensee for the required NA-2 TS change in order to perform the boron mixing tests.

Our discussion and evaluation regarding the licensee's requests of March 1 and March 2, 1982 is provided below.

Discussion and Evaluation:

Containment Isolation Valves for Post Accident Sampling

In order to meet the requirements of NUREG-0737, Section II.B.3, Post Accident Sampling of the Containment Isolation, a dedicated penetration is required which necessitates the addition of two containment isolation valves for both NA-1&2.

The isolation valves (to be numbered 36 and 37) for both NA-1&2 are Category I Containment Isolation Valves which meet the requirements for containment isolation integrity as specified in the NA-1&2 FSAR. Double barrier protection is provided by the two valves to assure that no single failure will result in the loss of containment integrity. Containment penetration piping including isolation valves No. 36 and No. 37 are designed to Seismic Category I requirements.

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Isolation valves Nos. 36 and No. 37 for NA-1&2 will be closed at all times and can only be opened upon remote-manual actuation from the control room. Opening of these valves will only take place under specific administrative control as specified in post-accident procedures.

We have already reviewed and approved the specifications for Category I Containment Isolation Valves as specified in NA-1&2 FSAR. Also, as stated above, these isolation valves are required to meet the provisions of NUREG-0737, Item II.B.3, Post Accident Sampling. Therefore, we find the inclusion of containment isolation valves No. 36 and No. 37 to Table 3.6.1 in the NA-1&2 TS to be acceptable.

Tests For Boron Mixing and Cooldown Under Natural Circulation Conditions

The NA-2 Facility Operating License NPF-7 License Conditions 2.C(15)(f) and 2.C(20)(b) require tests be completed to evaluate boron mixing and cooldown under natural circulation conditions. The tests are to be completed at the end of NA-2 Cycle 1 operations when sufficient decay heat will be available to produce natural circulation for the duration of the test. These tests, as previously conducted at the Salem Nuclear Generating Station, Unit No. 2, were successfully completed in about 9 hours. Since the NA-2 TS 3.4.1.2.b allows all Reactor Coolant Pumps (RCP) to be deenergized for not more than 1 hour while in Mode 3 (Hot Standby), the licensee requested relief from TS 3.4.1.2.b in order to conduct the required boron mixing tests.

Prior to initiating these tests, NA-2 will be in Mode 3 with at least a 1.77% shutdown margin. Boron concentration stability will be established with the RCP then deenergized and stable natural circulation confirmed. Heavily borated water will then be added to the Reactor Coolant System (RCS) to borate to Mode 5 (Cold Shutdown) concentration. The RCS will be sampled frequently to verify mixing, and when cold shutdown boron concentration is confirmed, a cooldown will be initiated.

The probability of occurrence or the consequences of a malfunction of equipment important to safety and previously evaluated in the NA-2 FSAR is not compromised by these tests because:

1. No safety related equipment will be required to operate outside of its normal operating parameters,
2. The RCPs will be available at all times to return the RCS to a forced circulation condition, and
3. The reactor will be shutdown in Mode 3.

When engaged in these tests all safety related equipment will be operable. Also, the margin of safety as described in the bases of the NA-2 TS is not reduced because the ability to remove heat under natural circulation conditions has already been successfully demonstrated under the provisions of a special low power test program approved by the NRC staff and conducted in July 1980.

Natural circulation tests conducted during this period included Special Tests 2-ST-6 (Cooldown Capability of the Chemical Volume and Control System) and 2-ST-11 (Effect of Steam Generator Secondary Side Isolation on Natural Circulation). Test 2-ST-8 (Natural Circulation Verification) was performed five times and 2-ST-9 (Natural Circulation with Loss of Offsite Power and Loss of Offsite and Onsite AC Power) was performed four times. For all of these tests the NRC operational safety criteria discussed in our Safety Evaluation Report attached to Amendment No. 1 to Facility License No. NPF-7 (Low Power Testing) dated July 3, 1980 were not exceeded in any of the tests and all test parameters remained well within safety margins. Finally, the licensee satisfied the requirement for operating training by having every licensed operator participate in at least one test and observe two more.

Therefore, based on the above, we find acceptable the one time relief from the requirement as specified in TS 3.4.1.2 for having one RC loop in operation in Mode 3 when conducting the boron mixing tests as specified in License Condition 2.C(15)(f) and 2.C(20)(b).

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 25, 1982

Principal Contributor:

L. B. Engle

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-338 AND 50-339VIRGINIA ELECTRIC AND POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments No. 38 and No. 19 to Facility Operating License Nos. NPF-4 and NPF-7 issued to the Virginia Electric and Power Company (the licensee) for operation of the North Anna Power Station, Units No. 1 and No. 2 (the facility) located in Louisa County, Virginia. The amendments are effective as of the date of issuance.

The amendments revise the NA-1&2 Technical Specifications by adding two containment isolation valves to Table 3.6.1 to meet the requirements of NUREG-0737, Item II.B.3, which requires a dedicated containment penetration for Post Accident Sampling of the containment atmosphere. The amendment for NA-2 also allows a one-time relief from Technical Specification 3.4.1.2 for requiring one reactor coolant loop to be in operation during Mode 3 (Hot Standby) when conducting the boron mixing and cooldown test under natural circulation conditions using decay heat.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since these amendments do not involve a significant hazards consideration.

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- 2 -

The Commission has determined that the issuance of the amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated March 1 and March 2, 1982, (2) Amendment No. 38 and No. 19 to Facility Operating Licenses No. NPF-4 and NPF-7, respectively, and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. 20555 and at the Board of Supervisor's Office, Louisa County Courthouse, Louisa, Virginia 23093 and at the Alderman Library, Manuscripts Department, University of Virginia, Charlottesville, Virginia 22901. A copy of items (2) and (3) may be obtained upon request to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 25th day of March, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing