

APR 29 1981

Docket Nos. 50-338  
and 50-339

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Mr. J. H. Ferguson  
Executive Vice President - Power  
Virginia Electric and Power Company  
Post Office Box 26666  
Richmond, Virginia 23261

Dear Mr. Ferguson:

The Commission has issued the enclosed Amendment Nos. 27 and 8 to Facility Operating License Nos. NPF-4 and NPF-7 for North Anna Power Station, Unit Nos. 1 and 2 (NA-1&2). The amendments consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 6, 1981 (Serial No. 109) and as supplemented by letter dated March 26, 1981 (Serial No. 195).

The amendments revise the Technical Specifications to allow an increase in enrichment for new and spent fuel from 3.5 weight percent of U-235 to 3.7 weight percent of U-235.

Copies of the related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by  
Robert A. Clark

Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing

Enclosures:

1. Amendment No. 27 to NPF-4
2. Amentment No. 8 to NPF-7
3. Safety Evaluation
4. Notice of Issuance



8105070427

P

*no legal objection to amend + F.R. notice*

OFFICE	ORB#30 DL	<i>[Signature]</i>	ORB#3 DL	AD:OR:DL	OELD		
SURNAME	<i>[Signature]</i>	L Engle	RAC Clark	T Novak	D SWANSON		
DATE	4/28/81	4/28/81	4/28/81	4/28/81	4/28/81		



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

DISTRIBUTION:  
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PMKreutzer

Docket No. 50-338 and 50-339

Docketing and Service Section  
Office of the Secretary of the Commission

SUBJECT: VIRGINIA ELECTRIC AND POWER COMPANY, North Anna Power Station,  
Units No. 1 and 2

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies ( 12 ) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Other: Amendment Nos. 27 and 8.  
Referenced documents have been provided PDR.

Division of Licensing, ORB#3  
Office of Nuclear Reactor Regulation

Enclosure:  
As Stated

OFFICE →	ORB#3:DL <i>PMKreutzer/ph</i>					
SURNAME →						
DATE →	4/29/81					



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

April 29, 1981

Docket Nos. 50-338  
and 50-339

Mr. J. H. Ferguson  
Executive Vice President - Power  
Virginia Electric and Power Company  
Post Office Box 26666  
Richmond, Virginia 23261

Dear Mr. Ferguson:

The Commission has issued the enclosed Amendment Nos. 27 and 8 to Facility Operating License Nos. NPF-4 and NPF-7 for North Anna Power Station, Unit Nos. 1 and 2 (NA-1&2). The amendments consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 6, 1981 (Serial No. 109) and as supplemented by letter dated March 26, 1981 (Serial No. 195).

The amendments revise the Technical Specifications to allow an increase in enrichment for new and spent fuel from 3.5 weight percent of U-235 to 3.7 weight percent of U-235.

Copies of the related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in cursive script, appearing to read "Robert A. Clark".

Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing

Enclosures:

1. Amendment No. 27 to NPF-4
2. Amentment No. 8 to NPF-7
3. Safety Evaluation
4. Notice of Issuance

8105070431

P

Virginia Electric and Power Company

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U.S. Environmental Protection Agency  
Washington, D. C. 20460

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Region III Office  
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Atomic Safety and Licensing  
Appeal Board Panel  
U.S. Nuclear Regulatory Commission  
Washington, D. C. 20555

cc w/enclosure(s) and incoming  
dtd: 3/6/81, 3/26/81

Commonwealth of Virginia  
Council of the Environment  
903 Ninth Street Office Building  
Richmond, Virginia 23129



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 27  
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated March 6, 1981 as supplemented March 26, 1981 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

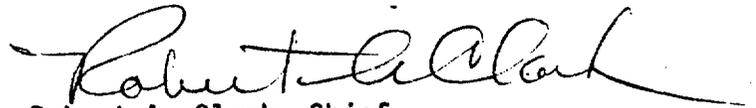
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 27, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 29, 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 27 TO FACILITY OPERATING LICENSE NO. NPF-4

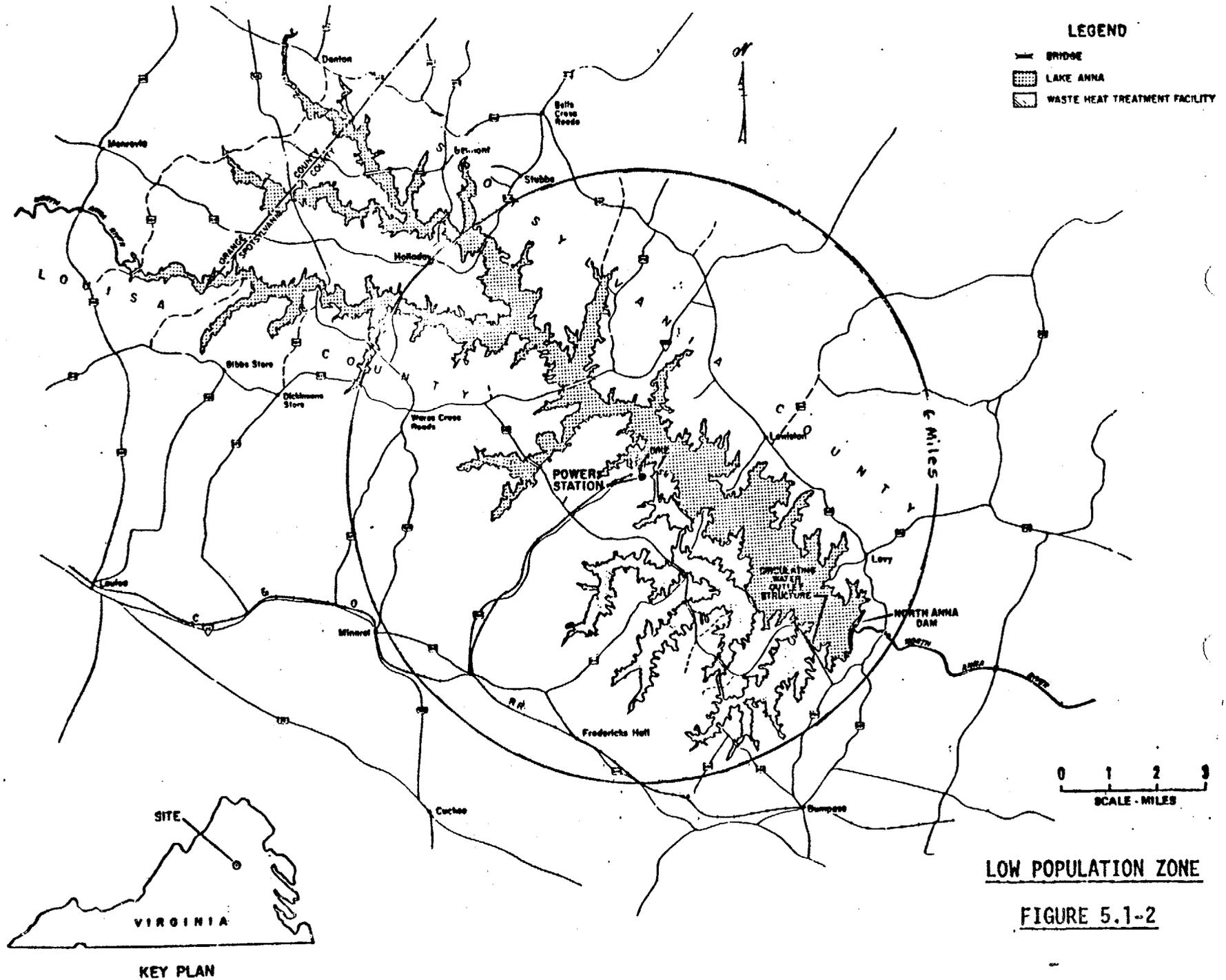
DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

5-4

5-5



## DESIGN FEATURES

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### DESIGN PRESSURE AND TEMPERATURE

5.2.2 The reactor containment building is designed and shall be maintained for a maximum internal pressure of 45 psig and a temperature of 280°F.

### 5.3 REACTOR CORE

#### FUEL ASSEMBLIES

5.3.1 The reactor core shall contain 157 fuel assemblies with each fuel assembly containing 264 fuel rods clad with Zircaloy -4. Each fuel rod shall have a nominal active fuel length of 144 inches and contain a maximum total weight of 1780 grams uranium. The initial core loading shall have a maximum enrichment of 3.2 weight percent U-235. Reload fuel shall be similar in physical design to the initial core loading and shall have a maximum enrichment of 3.7 weight percent U-235.

#### CONTROL ROD ASSEMBLIES

5.3.2 The reactor core shall contain 48 full length control rod assemblies. The full length control rod assemblies shall contain a nominal 142 inches of absorber material. The nominal values of absorber material shall be 80 percent silver, 15 percent indium and 5 percent cadmium. All control rods shall be clad with stainless steel tubing.

### 5.4 REACTOR COOLANT SYSTEM

#### DESIGN PRESSURE AND TEMPERATURE

5.4.1 The reactor coolant system is designed and shall be maintained:

## DESIGN FEATURES

- a. In accordance with the code requirements specified in Section 5.2 of the FSAR, with allowance for normal degradation pursuant to the applicable Surveillance Requirements,
- b. For a pressure of 2485 psig, and
- c. For a temperature of 650°F, except for the pressurizer which is 680°F.

### VOLUME

5.4.2 The total water and steam volume of the reactor coolant system is  $9957 \pm 10$  cubic feet at a nominal  $T_{avg}$  of 525°F.

### 5.5 METEOROLOGICAL TOWER LOCATION

5.5.1 The meteorological tower shall be located as shown on Figure 5.1-1.

### 5.6 FUEL STORAGE

#### CRITICALITY

5.6.1 The spent fuel storage racks containing new and/or spent fuel are designed and shall be maintained with a nominal 14 inch center-to-center distance between fuel assemblies placed in the spent fuel storage racks to ensure a  $k_{eff}$  equivalent of  $< 0.95$  with the storage pool filled with unborated water. The  $k_{eff}$  of  $\leq 0.95$  includes a conservative allowance of 3.4%  $\Delta k/k$  for uncertainties.

The new fuel pit storage racks are designed and shall be maintained with a nominal 21 inch center-to-center distance between new fuel assemblies such that, on a best estimate basis,  $k_{eff}$  will not exceed .98 with fuel of the highest anticipated enrichment in place assuming optimum moderation.\*

If fresh fuel is stored dry for a core loading in the spent fuel racks, a center-to-center distance between new fuel assemblies will be administratively limited to 28 inches. On a best estimate basis,  $k_{eff}$  will not exceed .98 with fuel of the highest anticipated enrichment in place assuming optimum moderation.\*

\*E.G., an aqueous foam envelopment as the result of fire fighting.

## DESIGN FEATURES

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### DRAINAGE

5.6.2 The spent fuel pit is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 288.83 feet. Mean Sea Level, USGS datum.

### CAPACITY

5.6.3 The fuel storage pool is designed and shall be maintained with a storage capacity limited to no more than 966 fuel assemblies.

### 5.7 COMPONENT CYCLIC or TRANSIENT LIMIT

5.7.1 The components identified in Table 5.7-1 are designed and shall be maintained within the cyclic or transient limits of Table 5.7-1.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 8  
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated March 6, 1981 and as supplemented March 26, 1981 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications in paragraph 2.C.(2) of Facility Operating License No. NPF-7 and paragraph 2.C.(2) is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 8, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

- (a) The written procedures required by Technical Specification 6.8.1 for the Technical Specifications listed below shall be implemented within 30 days after the date of this license:

4.3.2.1.3        Items 10 and 11 of Table 3.3-5  
4.3.3.6  
4.4.3.2.1  
4.4.3.2.2  
4.4.6.2.2

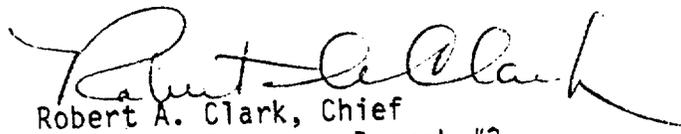
The required surveillance shall be completed before the end of the first surveillance interval.

- (b) The end of the current surveillance period for the Surveillance Requirements listed below may be extended beyond the time limit specified by Technical Specification 4.0.2.a. In each case, the required surveillance shall be completed by the revised due date. After May 31, 1981 the plant shall not be operated in Modes 1, 2, 3, or 4 until the Surveillance Requirements listed below have been completed. Upon accomplishment of the surveillance, the provisions of Technical Specification 4.0.2.a shall apply.

Specification 4.8.2.3.2.d  
4.8.2.4.2

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 29, 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 8 TO FACILITY OPERATING LICENSE NO. NPF-7

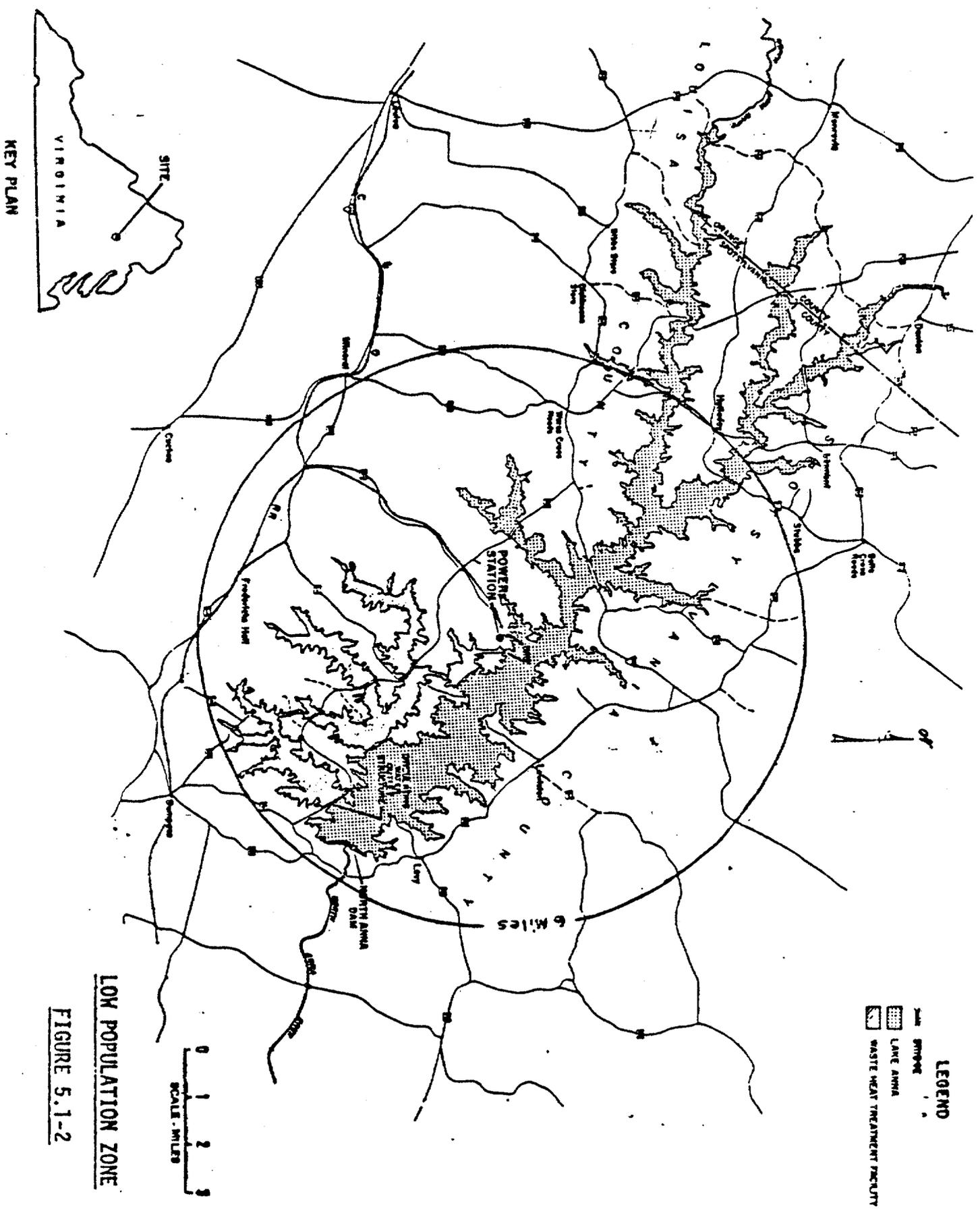
DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

5-4

5-5



LOW POPULATION ZONE  
FIGURE 5.1-2

## DESIGN FEATURES

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### 5.3 REACTOR CORE

#### FUEL ASSEMBLIES

5.3.1 The reactor core shall contain 157 fuel assemblies with each fuel assembly containing 264 fuel rods clad with Zircaloy -4. Each fuel rod shall have a nominal active fuel length of 144 inches and contain a maximum total weight of 1780 grams uranium. The initial core loading shall have a maximum enrichment of 3.2 weight percent U-235. Reload fuel shall be similar in physical design to the initial core loading and shall have a maximum enrichment of 3.7 weight percent U-235.

#### CONTROL ROD ASSEMBLIES

5.3.2 The reactor core shall contain 48 full length control rod assemblies. The full length control rod assemblies shall contain a nominal 142 inches of absorber material. The nominal values of absorber material shall be 80 percent silver, 15 percent indium and 5 percent cadmium. All control rods shall be clad with stainless steel tubing.

### 5.4 REACTOR COOLANT SYSTEM

#### DESIGN PRESSURE AND TEMPERATURE

5.4.1 The reactor coolant system is designed and shall be maintained:

- a. In accordance with the code requirements specified in Section 5.2 of the FSAR, with allowance for normal degradation pursuant to the applicable Surveillance Requirements,
- b. For a pressure of 2485 psig, and
- c. For a temperature of 650°F, except for the pressurizer which is 680°F.

#### VOLUME

5.4.2 The total water and steam volume of the reactor coolant system is  $.9957 \pm 10$  cubic feet at a nominal  $T_{avg}$  of 525°F.

## DESIGN FEATURES

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### 5.5 METEOROLOGICAL TOWER LOCATION

5.5.1 The meteorological tower shall be located as shown on Figure 5.1-1.

### 5.6 FUEL STORAGE

#### CRITICALITY

5.6.1.1 The spent fuel storage racks are designed and shall be maintained with:

- a. A  $K_{eff}$  equivalent to less than or equal to 0.95 when flooded with unborated water, which includes a conservative allowance of 3.4% delta k/k for uncertainties.
- b. A nominal 14 inch center-to-center distance between fuel assemblies placed in the storage racks.

5.6.1.2 The new fuel pit storage racks are designed and shall be maintained with a nominal 21 inch center-to-center distance between new fuel assemblies such that, on a best estimate basis,  $k_{eff}$  will not exceed .98, with fuel of the highest anticipated enrichment in place, when aqueous foam moderation is assumed.

5.6.1.3 If new fuel for the first core loading is stored dry in the spent fuel storage racks the center-to-center distance between the new fuel assemblies will be administratively limited to 28 inches and the  $k_{eff}$  shall not exceed 0.98 when aqueous foam moderation is assumed.

#### DRAINAGE

5.6.2 The spent fuel pit is designed and shall be maintained to prevent inadvertent draining of the pool below elevation 288.83 feet Mean Sea Level, USGS datum.

#### CAPACITY

5.6.3 The fuel storage pool is designed and shall be maintained with a storage capacity limited to no more than 966 fuel assemblies.

## DESIGN FEATURES

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### 5.7 COMPONENT CYCLIC or TRANSIENT LIMIT

5.7.1 The components identified in Table 5.7-1 are designed and shall be maintained within the cyclic or transient limits of Table 5.7-1.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 27 AND NO. 8 TO

FACILITY OPERATING LICENSE NOS. NPF-4 AND NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2

DOCKET NOS. 50-338 AND 50-339

Introduction:

By letter dated March 6, 1981 (Serial No. 109), as supplemented by letter dated March 26, 1981 (Serial No. 195), the Virginia Electric and Power Company (the licensee) requested amendments to Facility Operating Licenses, NPF-4 and NPF-7 for the North Anna Power Station, Units No. 1 and 2 (NA-1&2).

The licensee has proposed an enrichment of 4.1 weight percent U-235 in the new and spent fuel storage locations. This storage of increased enrichment fuel is necessary to permit the licensee's participation in a Department of Energy demonstration and evaluation program concerned with high burnup technology. This higher enrichment can permit a higher average discharge burnup of fuel from a reactor.

We have not completed our review of the safety aspects of operating a reactor similar to those at NA-1&2 at an enrichment of 4.1 weight percent U-235 and to an extended burnup of up to 45,000 Mega-watt-days per Metric Ton-Uranium (Mwd/MTU).

We have reviewed the safety aspects of storing 4.1 percent fuel in the NA-1&2 new and spent fuel storage racks. We have also reviewed the operation of the NA-1&2 reactors with fuel enriched to 3.7 percent which is an insignificant increase over the current Technical Specifications limit of 3.5 percent. Since we are limiting the operating value to 3.7 percent, the limit on storage will also be 3.7 percent even though the analysis was done for 4.1 weight percent U-235.

We are continuing our review of licensees' requests to operate reactors with an increase in enrichment to 4.1 percent and with burnups to 45,000 Mwd/MTU. Also, we note that we are currently engaged in discussions with the nuclear industry regarding investigations to address extended burnup in a generic manner.

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## Discussion

### 1. Increased Enrichment and Burnup:

We have reviewed the licensee's submittal in the context of our on-going review of the technology (analyses and data) for extended burnups. We find that there is sufficient information currently available from power reactor operations with similar fuel at other Westinghouse plants to justify operations at NA-1&2 with a fuel discharge batch average burnup up to 33,000 MWd/MTU.

The approval of 33,000 MWd/MTU burnup will allow the licensee to order fuel as requested in May 1981 for a planned discharge burnup of approximately 36,000 MWd/MTU in 1985. However, as stipulated in a similar case for the licensee's Surry Power Station, Units 1 & 2, we will require a safety analysis to explicitly address burnup as an operating parameter whenever planned operations would exceed the maximum batch average discharge value of 33,000 MWd/MTU. Therefore, the licensee should submit a safety analysis prior to the cycle of operation that would exceed the currently approved 33,000 MWd/MTU. If, as presently anticipated, the fuel vendors (including Westinghouse) submit generic safety analyses for our review, and subsequent approval is granted, then the licensee need only cite the approved analysis prior to the next projected burnup extension, and no further analyses will then be required by the licensee.

The licensee has requested that the NA-1&2 Technical Specifications be amended to allow reactor operations at an enrichment of 3.7 percent U-235. This is approximately a 6 percent increase over the present Technical Specification value of 3.5 percent. The reason for granting an increase to 3.7 percent from 3.5 percent is that the licensee will be procuring fuel at a 3.6 percent enrichment and fuel procured has a tolerance on enrichment which could slightly exceed 3.6 percent.

### 2. Effective Multiplication Factor (Storage Racks)

Changes in the effective multiplication factor for storage racks having an enrichment up to 4.1 weight percent U-235 have been calculated by the same methods previously used to obtain the former enrichment limit of 3.5 weight percent U-235. Scoping calculations and sensitivity studies were performed with the PDQ-7 code with neutron cross-sections prepared by the NUMICE code (a version of the LEOPARD code). The case for 4.1 weight percent U-235 and for nominal dimensions of the storage racks is verified by calculations with the KENO-IV Monte-Carlo code with cross-sections prepared by the AMPX codes. Both of these code combinations are widely used and we find them acceptable. The AMPX-KENO set has been verified by the licensee's consultant (NUS Corporation), and has been shown to be conservative for designs similar to NA.

For the base case, the calculated value of the effective multiplication factor is 0.916. To this number is added a value of 0.004 to account for the slightly higher reactivity of the Westinghouse 15X15 and the Babcock and Wilcox 15X15 and 17X17 assemblies which might be stored in the pool. Sensitivity studies were performed to account for the uncertainties in fuel enrichment, center to center spacing, can wall thickness, can distortion, stainless steel composition and eccentric fuel loading. Computational uncertainties for the 95 percent probability level and the effect of a pool water temperature increase to 212 degrees Fahrenheit were then obtained.

When the mechanical calculational uncertainties are combined statistically and added to the allowances for more reactive fuel and high pool water temperature, a total uncertainty of 0.027 is obtained. The final value for the effective multiplication factor with all uncertainties added is 0.943. This value meets our acceptance criteria of less than or equal to 0.95 for the effective multiplication factor and is acceptable.

### 3. Radiological Consequences of Accidents:

We have evaluated the radiological consequences of accidents at NA-1&2 based on raising the allowable enrichment of the fuel to 4.1 weight percent U-235. In the absence of information justifying higher burnups at this time, the safety of the proposed change has been evaluated for the new fuel up to 33,000 MWd/MTU.

Certain accidents, such as the steam line break, waste gas tank failure, and volume control tank failure would be unchanged from present evaluations since no changes are being made in the allowable activity of the primary coolant or in the content of the waste gas tank.

The loss-of-coolant accident (LOCA) presumes failure of all fuel rods in the core with the release of all the noble gases and 25 percent of the radioiodines. A change in enrichment should have, at worst, a minor effect on the core content of these nuclides and result in no worse radiological consequences than already evaluated for NA-1&2.

The steam generator tube rupture and fuel handling accidents are assumed to cause some failure of fuel. Again, the increase in enrichment will not cause a major change in the calculated core content of important radionuclides. If more rods were predicted to fail than previously assumed, the cycle-by-cycle reload analyses would include revised radiological consequences.

We conclude therefore, that a change in the fuel enrichment values, as reflected in the Technical Specifications, from the present value of 3.5 percent to 4.1 percent would not substantially change the consequences of accidents, provided the fuel burnup does not exceed the 33,000 MWD/MTU value.

#### Evaluation:

We have reviewed the safety aspects including accident analysis for operating the NA-1&2 reactors at an enrichment of 3.7 weight percent U-235. This is an increase from the present Technical Specification value of 3.5 percent. This change is less than 6 percent and any consequences of this change on accidents is insignificant. Also, the enrichment change is of small magnitude, and we have not approved operation of the reactors to burnups above those presently approved. Therefore, we find this change to be acceptable.

#### Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 29, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-338 AND 50-339VIRGINIA ELECTRIC AND POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments No. 27 and No. 8 to Facility Operating License Nos. NPF-4 and NPF-7 issued to the Virginia Electric and Power Company (the licensee) for operation of the North Anna Power Station, Units No. 1 and No. 2 (the facility) located in Louisa County, Virginia. The amendments are effective as of the date of issuance.

The amendments revise the Technical Specifications to allow an increase in enrichment for new and spent fuel from 3.5 weight percent of U-235 to 3.7 weight percent of U-235.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since these amendments do not involve a significant hazards consideration.

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The Commission has determined that the issuance of the amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated March 6, 1981 as supplemented March 26, 1981; (2) Amendment No. 27 and No. 8 to Facility Operating Licenses No. NPF-4 and NPF-7 and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. 20555 and at the Board of Supervisor's Office, Louisa County Courthouse, Louisa, Virginia 23093 and at the Alderman Library, Manuscripts Department, University of Virginia, Charlottesville, Virginia 22901. A copy of items (2) and (3) may be obtained upon request to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 29th day of April, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing