MAY 22 1981

Docket Nos. ~~50-338~~ and 50-339

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Mr. J. H. Ferguson
Executive Vice President - Power
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Dear Mr. Ferguson:

The Commission has issued the enclosed Amendment No. 30 to Facility Operating License No. NPF-4 and Amendment No. 11 to Facility Operating License No. NPF-7 for North Anna Power Station, Units No. 1 and 2 (NA-1&2) respectively. The amendments are to become effective on MAY 29 1981

The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letters dated July 30, 1980 (Serial No.487), August 28, 1980 (Serial No. 731), and October 15, 1980 (Serial No. 845) and as supplemented by letters dated August 1, 1980 (Serial No. 678) and February 23, 1981 (Serial No. 078).

These amendments consist of changes to the Technical Specifications which revise the Administrative Controls, Section 6. The changes reflect corporate and plant reorganizations including the addition of an Assistant Station Manager. A Safety Evaluation and Control group has been established which replaces the System Nuclear Safety and Operating Committee.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by
Robert A. Clark

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Enclosures:
1. Amendment No. 30 to NPF-4
2. Amendment No. 11 to NPF-7
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

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CP
1

OFFICE	ORB#3:DL	ORB#3:DL	LQB	LQB	ORB#3:DL	AD:ORB:DL	OELD
SURNAME	PKreutzer	LEngle/pn	FAllenspach	DVassallo	RAClark	TMNovak	DSwanson
DATE	5/6/81	5/6/81	5/12/81	5/12/81	5/12/81	5/12/81	5/14/81

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

DISTRIBUTION:
Docket File
ORB#3 Rdg
PMKreutzer

Docket No. 50-338 and 50-339

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: VIRGINIA ELECTRIC AND POWER COMPANY, North Anna Power Station, Unit Nos. 1 and 2

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

☐ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).

☐ Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.

☐ Notice of Availability of Applicant's Environmental Report.

☐ Notice of Proposed Issuance of Amendment to Facility Operating License.

☐ Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.

☐ Notice of Availability of NRC Draft/Final Environmental Statement.

☐ Notice of Limited Work Authorization.

☐ Notice of Availability of Safety Evaluation Report.

☐ Notice of Issuance of Construction Permit(s).

☐ Notice of Issuance of Facility Operating License(s) or Amendment(s).

X☐ Other: Amendment Nos. 30 and 11
Referenced documents have been provided PDR.

Division of Licensing, ORB#3
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE→	ORB#3:DL					
SURNAME→	PMKreutzer/pn					
DATE→	5/22/81					

NRC FORM 102 7—79

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 22, 1981

Docket Nos. 50-338
and 50-339

Mr. J. H. Ferguson
Executive Vice President - Power
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Dear Mr. Ferguson:

The Commission has issued the enclosed Amendment No. 30to Facility Operating License No. NPF-4 and Amendment No. 11 to Facility Operating License No. NPF-7 for North Anna Power Station, Units No. 1 and 2 (NA-1&2) respectively. The amendments are to become effective on May 29, 1981.

The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letters dated July 30, 1980 (Serial No.487), August 28, 1980 (Serial No. 731), and October 15, 1980 (Serial No. 845) and as supplemented by letters dated August 1, 1980 (Serial No. 678) and February 23, 1981 (Serial No. 078).

These amendments consist of changes to the Technical Specifications which revise the Administrative Controls, Section 6. The changes reflect corporate and plant reorganizations including the addition of an Assistant Station Manager. A Safety Evaluation and Control group has been established which replaces the System Nuclear Safety and Operating Committee.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Enclosures:
1. Amendment No.30 to NPF-4
2. Amendment No.11 to NPF-7
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

Virginia Electric and Power Company

cc:
Richard M. Foster, Esquire
Reick, Williamson, Schwartz,
Leavenworth & Cope, P.C.
P. O. Box 4579
Boulder, Colorado 80306

Michael W. Maupin, Esquire
Hunton, Williams, Gay and Gibson
P. O. Box 1535
Richmond, Virginia 23212

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Mr. Edward Kube
Board of Supervisors
Louisa County Courthouse
P. O. Box 27
Louisa, Virginia 23093

Ellyn R. Weiss, Esquire
Sheldon, Harman, Roisman and Weiss
1725 I Street, N.W. Suite 506
Washington, D. C. 20006

Mr. W. R. Cartwright, Station Manager
P. O. Box 402
Mineral, Virginia 23117

Mr. Anthony Gambardella
Office of the Attorney General
11 South 12th Street - Room 308
Richmond, Virginia 23219

Mr. Edward Webster
Resident Inspector/North Anna
c/o U.S.N.R.C.
Route 2, Box 78A
Mineral, Virginia 23117

Mrs. June Allen
North Anna Environmental Coalition
2118-C Olive Street
Greensboro, North Carolina 27401

Mr. James Torson
501 Leroy
Socorro, New Mexico 87891

Mrs. Margaret Dietrich
Route 2, Box 568
Gordonsville, Virginia 22042

Mr. James C. Dunstance
State Corporation Commission
Commonwealth of Virginia
Blandon Building
Richmond, Virginia 23209

Director, Criteria and Standards Division
Office of Radiation Programs (ANR-460)
U.S. Environmental Protection Agency
Washington, D. C. 20460

U.S. Environmental Protection Agency
Region III Office
ATTN: EIS COORDINATOR
Curtis Building
6th and Walnut Streets
Philadelphia, Pennsylvania 19106

Mr. Paul W. Purdom
Environmental Studies Institute
Drexel University
32nd and Chestnut Streets
Philadelphia, Pennsylvania 19104

Atomic Safety and Licensing
Appeal Board Panel
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

cc w/enclosure(s) and incoming
dtd: 7/30/80, 10/15/80, 8/1/80, 2/23/81

Commonwealth of Virginia
Council of the Environment
903 Ninth Street Office Building
Richmond, Virginia 23129

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 30
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendments by Virginia Electric and Power Company (the licensee) dated July 30, 1980, August 28, 1980 and October 15, 1980 as supplemented by letters dated August 1, 1980 and February 23, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 30 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective May 29, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 22, 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 30 TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages
XVIII
XIX
6-1
6-1a (added)
6-2
6-3
6-4a
6-5
6-6
6-7
6-8
6-9
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ADMINISTRATIVE CONTROLS

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ADMINISTRATIVE CONTROLS

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Station Manager shall be responsible for overall facility operation. In his absence, the Assistant Station Manager shall be responsible for overall facility operation. During the absence of both, the Station Manager shall delegate in writing the succession to this responsibility.

6.1.2 The Shift Supervisor (or during his absence from the Control Room, a designated individual) shall be responsible for the Control Room command function and shall be the only individual that may direct the licensed activites of licensed operators. A management directive to this effect, signed by the Executive Vice President-Power, shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.

b. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODES 1, 2, 3 or 4, at least one licensed Senior Reactor Operator shall be in the Control Room.

c. A health physics technician# shall be on site when fuel is in the reactor.

d. ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

e. A Fire Brigade of at least 5 members shall be maintained onsite at all times#. The Fire Brigade shall not include the minimum shift crew shown in Table 6.2-1 or any personnel required for other essential functions during a fire emergency.

#The health physics technician and Fire Brigage composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

6.2.3 SAFETY ENGINEERING STAFF (SES)

FUNCTION

6.2.3.1 The SES shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources which may indicate areas for improving plant safety.

COMPOSITION

6.2.3.2 The SES shall be composed of at least five dedicated, full-time engineers located onsite.

RESPONSIBILITIES

6.2.3.3 The SES shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

AUTHORITY

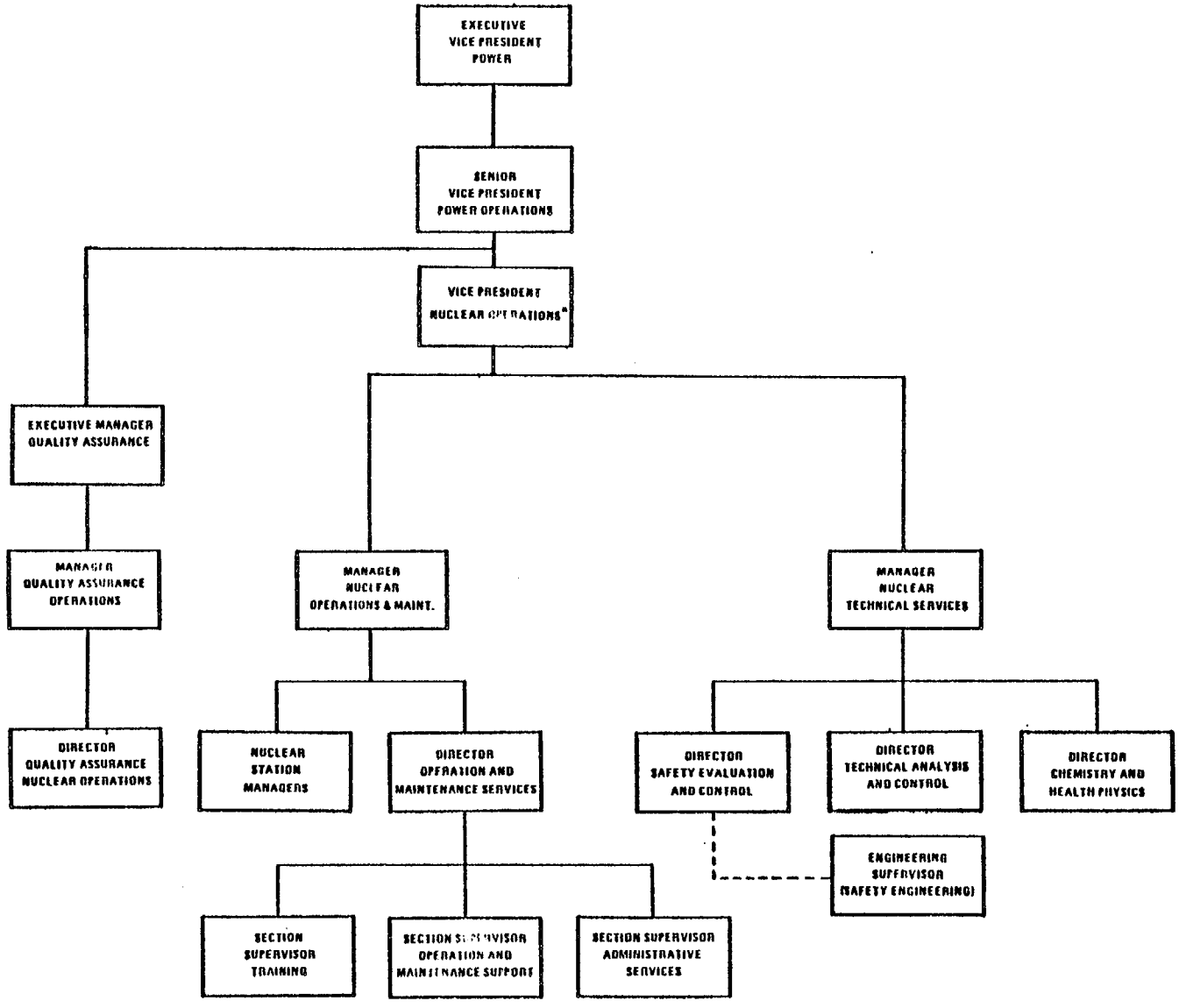
6.2.3.4 The SES shall make detailed recommendations for revised procedures, equipment modifications, or other means of improving plant safety to the Station Manager and the Director-Safety Evaluation and Control.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall serve in an advisory capacity to Shift Supervisor on matters pertaining to the engineering aspects of assuring safe operation of the unit.

6.2.4.2 The Shift Technical Advisor shall disseminate relevant operational experience identified by the SES.

*Not responsible for sign-off function.



*RESPONSIBLE FOR CORPORATE FIRE PROTECTION PROGRAM

Figure 6.2-1 Offsite Organization for Facility Management and Technical Support

Figure 6.2-2 Facility Organization - North Anna - Units 1 and 2

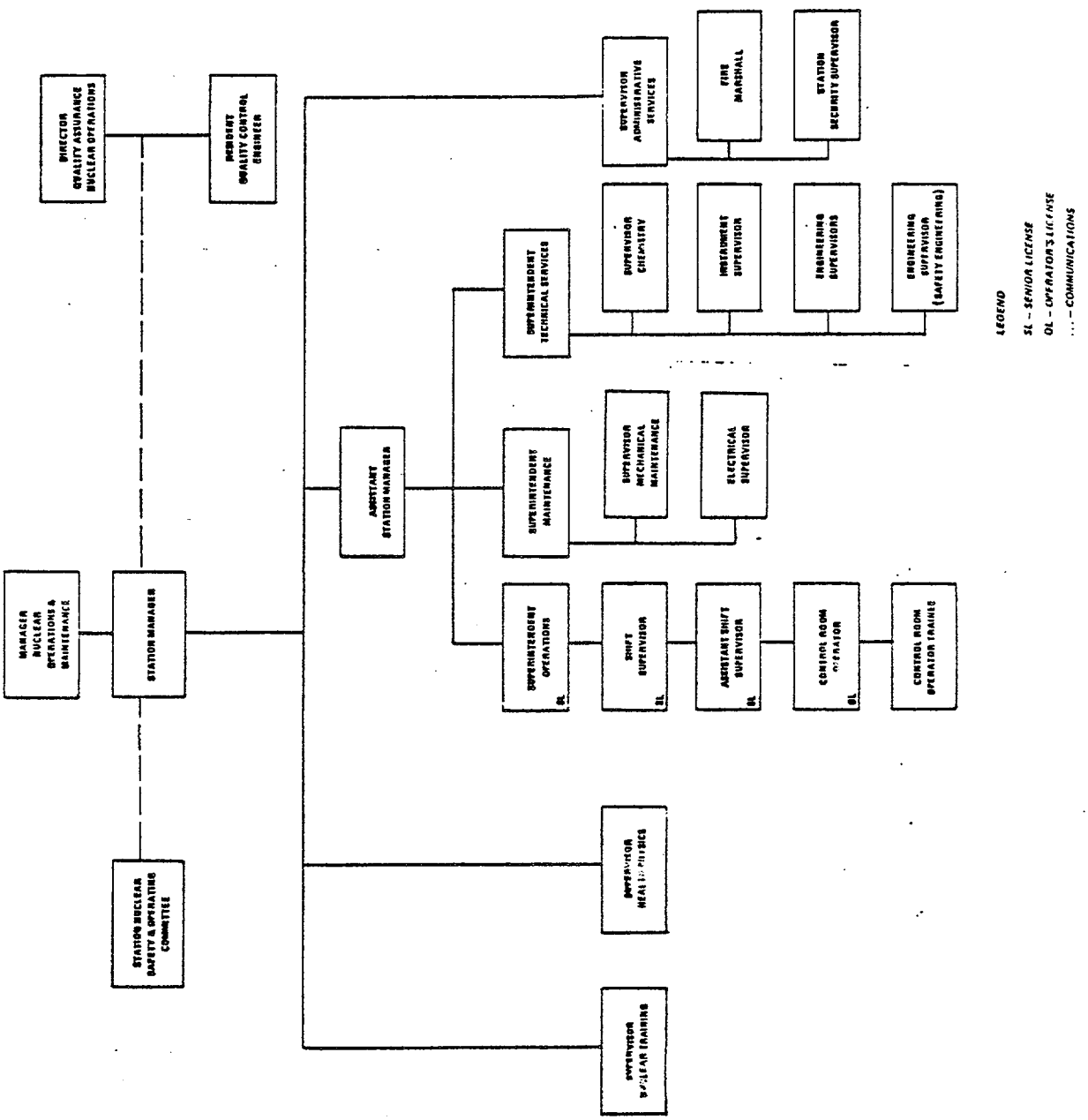


TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION

WITH UNIT 2 IN MODE 5 OR 6 OR DE-FUELED

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3, & 4	MODES 5 & 6
SS	1[a]	1[a]
SRO	1	none
RO	2	1
AO	2	2[b]
STA	1	none

WITH UNIT 2 IN MODES 1, 2, 3, OR 4

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3, & 4	MODES 5 & 6
SS	1[a]	1[a]
SRO	1[a]	none
RO	2[b]	1
AO	2[b]	1
STA	1[a]	none

a/ Individual may fill the same position of Unit 2.

b/ One of the two required individuals may fill the same position on Unit 2.

TABLE 6.2-1 (Continued)

SS - Shift Supervisor with a Senior Reactor Operators License on Unit 1.
SRO - Individual with a Senior Reactor Operators License on Unit 1.
RO - Individual with a Reactor Operators License on Unit 1.
AO - Auxiliary Operator
STA - Shift Technical Advisor

Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 1, 2, 3 or 4, an individual (other than the Shift Technical Advisor) with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 5 or 6, an individual with a valid RO license (other than the Shift Technical Advisor) shall be designated to assume the Control Room command function.

Licensed operators shall:*

1. Not work more than 12 hours straight,
2. Not work more than 24 hours in any 48-hour period,
3. Not work more than 72 hours in any 7-day period,
4. Not work more than 14 consecutive days without having 2 consecutive days off.

*Deviation from these requirements may be authorized by the Station Manager in accordance with established procedures and with documentation of the cause. Overtime limits do not include the shift turnover time.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1 - 1971 for comparable positions and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, except for (1) the Supervisor - Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 The Station Manager is responsible for ensuring that retraining and replacement training programs for the facility staff are maintained and that such programs meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1 - 1971 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the SES.

6.5 REVIEW AND AUDIT

6.5.1 STATION NUCLEAR SAFETY AND OPERATING COMMITTEE (SNSOC)

FUNCTION

6.5.1.1 The SNSOC shall function to advise the Station Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The SNSOC shall be composed of the :

Chairman:	Station Manager
Vice Chairman:	Assistant Station Manager
Member:	Superintendent-Operations
Member:	Superintendent-Maintenance
Member:	Superintendent-Technical Services
Member:	Supervisor-Health Physics

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SNSOC Chairman to serve on a temporary basis; however, no more than one alternate shall participate as a voting member in SNSOC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

a. Review of 1) all procedures required by Specification 6.8.1 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.

b. Review of all proposed tests and experiments that affect nuclear safety.

c. Review of all proposed changes to Appendix "A" Technical Specifications.

d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Manager-Nuclear Operations and Maintenance and the Director-Safety Evaluation and Control.

f. Review of events requiring 24 hour written notification to the Commission.

g. Review of facility operations to detect potential nuclear safety hazards.

h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee.

i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Station Nuclear Safety and Operating Committee.

j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Station Nuclear Safety and Operating Committee.

AUTHORITY

6.5.1.7 The SNSOC shall:

a. Recommend to the Station Manager written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.

b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.

c. Provide written notification within 24 hours to Manager-Nuclear Operations and Maintenance and the Director-Safety Evaluation and Control of disagreement between the SNSOC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Manager-Nuclear Operations and Maintenance and the Director-Safety Evaluation and Control.

6.5.2 SAFETY EVALUATION AND CONTROL (SEC)

FUNCTION

6.5.2.1 SEC shall function to provide independent review of designated activities in the areas of:

a. Nuclear power plant operations

b. Nuclear engineering

c. Chemistry and radiochemistry

d. Metallurgy

e. Instrumentation and control

f. Radiological safety

g. Mechanical and electrical engineering

h. Administrative controls and quality assurance practices

i. Other appropriate fields associated with the unique characteristics of the nuclear power plant

COMPOSITION

6.5.2.2 The SEC staff shall be composed of the Director-Safety Evaluation and Control and a minimum of three individuals who are qualified as staff specialists. Each SEC staff specialist shall have an academic degree in an engineering or physical science field and, in addition, shall have a minimum of five years technical experience in one or more areas given in Specification 6.5.2.1. These staff specialists shall not be directly involved in the licensing function.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the Director-Safety Evaluation and Control to provide expert advice to the SEC.

MEETING FREQUENCY

6.5.2.5 The SEC staff shall meet at least once per calendar month for the purpose of fostering interaction of reviews regarding safety-related operational activities.

REVIEW

6.5.2.7 The following subjects shall be reviewed by SEC:

a. Written safety evaluations of changes in the stations as described in the Safety Analysis Report, changes in procedures as described in the Safety Analysis Report and tests or experiments not described in the Safety Analysis Report which are completed without prior NRC approval under the provisions of 10 CFR 50.59(a)(1). This review is to verify that such changes, tests or experiments did not involve a change in the technical specifications or an unreviewed safety question as defined in 10 CFR 50.59(a)(2) and is accomplished by review of minutes of the Station Nuclear Safety and Operating Committee and the design change program.

b. Proposed changes in procedures, proposed changes in the station, or proposed tests or experiments, any of which may involve a change in the technical specifications or an unreviewed safety question as defined in 10 CFR 50.59(a)(2). Matters of this kind shall be referred to the Director-Safety Evaluation and Control by the Station Nuclear Safety and Operating Committee following its review prior to implementation.

c. Changes in the technical specifications or license amendments relating to nuclear safety prior to implementation except in those cases where the change is identical to a previously reviewed proposed change.

d. Violations and reportable occurrences such as:

1. Violations of applicable codes, regulations, orders, Technical Specifications, license requirements or internal procedures or instructions having safety significance;

2. Significant operating abnormalities or deviations from normal or expected performance of station safety-related structures, systems, or components; and

3. Reportable occurrences as defined in the station Technical Specification 6.9.1.8.

Review of events covered under this paragraph shall include the results of any investigations made and recommendations resulting from such investigations to prevent or reduce the probability of recurrence of the event.

e. The Quality Assurance Department audit program at least once per 12 months and audit reports.

f. Any other matter involving safe operation of the nuclear power stations which is referred to the Director-Safety Evaluation and Control by the Station Nuclear Safety and Operating Committee.

g. Reports and meeting minutes of the Station Nuclear Safety and Operating Committee.

AUTHORITY

6.5.2.9 The Director-Safety Evaluation and Control shall report to and advise the Manager-Nuclear Technical Services, who shall advise the Vice President-Nuclear Operations on those areas of responsibility specified in Section 6.5.2.7.

RECORDS

6.5.2.10 Records of SEC activities required by Section 6.5.2.7 shall be prepared and maintained in the SEC files and a summary shall be disseminated as indicated below each calendar month.

1. Vice President-Nuclear Operations

2. Nuclear Power Station Managers

3. Manager-Nuclear Operations and Maintenance

4. Manager-Nuclear Technical Services

5. Manager-Quality Assurance, Operations

6. Others that the Director-Safety Evaluation and Control may designate.

6.5.3 QUALITY ASSURANCE DEPARTMENT

FUNCTION

6.5.3.1 The Quality Assurance Department shall function to audit station activities. These audits shall encompass:

a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.

b. The performance, training and qualifications of the entire facility staff at least once per 12 months.

c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.

d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.

e. The Station Emergency Plan and implementing procedures at least once per 24 months.

f. The Station Security Plan and implementing procedures at least once per 24 months.

g. Any other area of facility operation considered appropriate by the Executive Manager-Quality Assurance or the Senior Vice President-Power Operations.

h. The Station Fire Protection Program and implementing procedures at least once per 24 months.

i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.

j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

AUTHORITY

6.5.3.2 The Quality Assurance Department shall report to and advise the Executive Manager-Quality Assurance, who shall advise the Senior Vice President-Power Operations on those areas of responsibility specified in Section 6.5.3.1.

RECORDS

6.5.3.3 Records of the Quality Assurance Department audits shall be prepared and maintained in the department files. Audit reports shall be disseminated as indicated below:

1. Nuclear Power Station Manager
2. Manager-Nuclear Operations and Maintenance
3. Manager-Nuclear Technical Services
4. Manager-Quality Assurance, Operations
5. Director-Quality Assurance, Nuclear Operations
6. Director-Safety Evaluation and Control
7. Supervisor of area audited
8. Nuclear Power Station Resident Quality Control Engineer

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the SNSOC and submitted to the Director-Safety Evaluation and Control and the Manager-Nuclear Operations and Maintenance.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

a. The facility shall be placed in at least HOT STANDBY within one hour.

b. The Safety Limit violation shall be reported to the Commission, the Manager-Nuclear Operations and Maintenance and to the Director-Safety Evaluation and Control within 24 hours.

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSOC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission, the Director-Safety Evaluation and Control and the Manager-Nuclear Operations and Maintenance within 14 days of the violation.

a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.

b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.

c. Records of facility radiation and contamination surveys.

d. Records of radiation exposure for all individuals entering radiation control areas.

e. Records of gaseous and liquid radioactive material released to the environs.

f. Records of transient operational cycles for those facility components identified in Table 5.9-1.

g. Records of reactor tests and experiments.

h. Records of training and qualification for current members of the plant staff.

i. Records of in-service inspections performed pursuant to these Technical Specifications.

j. Records of Quality Assurance activities required by the QA Manual.

k. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

l. Records of meetings of the SNSOC.

m. Records of meetings of the System Nuclear Safety and Operating Committee to issuance of Amendment No. _____.

n. Records of secondary water sampling and water quality.

o. Records of Environmental Qualification which are covered under the provisions of paragraph 6.13.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit.* Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.

c. An individual qualified in radiation protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the facility Health Physicist in the Radiation Work Permit.

6.12.2 The requirements of 6.12.1, above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Supervisor on duty and/or the Plant Health Physicist.

*Health Physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 30 TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "B" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

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I-5-4
I-5-5

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TABLE OF CONTENTS (Cont'd)

5.0 ADMINISTRATIVE CONTROLS

Administrative controls below apply to Appendix B Part I only.

5.1 Responsibility

The responsibility for implementing the Environmental Technical Specifications is assigned to the Manager-Nuclear Operations and Maintenance at the corporate level and to the Station Manager at the station level. The Superintendent - Operations shall be responsible for ensuring that the station is operated in accordance with the Limiting Conditions of Operation. The Station Supervisor - Health Physics shall be responsible for the radiological environmental surveillance requirements. The Executive Manager of Environmental Services shall be responsible for providing services which will fulfill the non-radiological environmental surveillance requirements.

5.2 Organization

The relationship between the Nuclear Operations Department and the Environmental Services Department is shown in Figure 5.2-1.

5.3 Review and Audit

5.3.1 Station Nuclear Safety and Operating Committee (SNSOC)

5.3.1.1 Function

The SNSOC, as described in Section 6.5.1 of Appendix A of this license, shall function to advise the Station Manager on matters related to the environmental impact of the station. With the SNSOC is exercising its responsibility for non-radiological aspects of the ETS, the Station Biologist or his alternate shall be consulted.

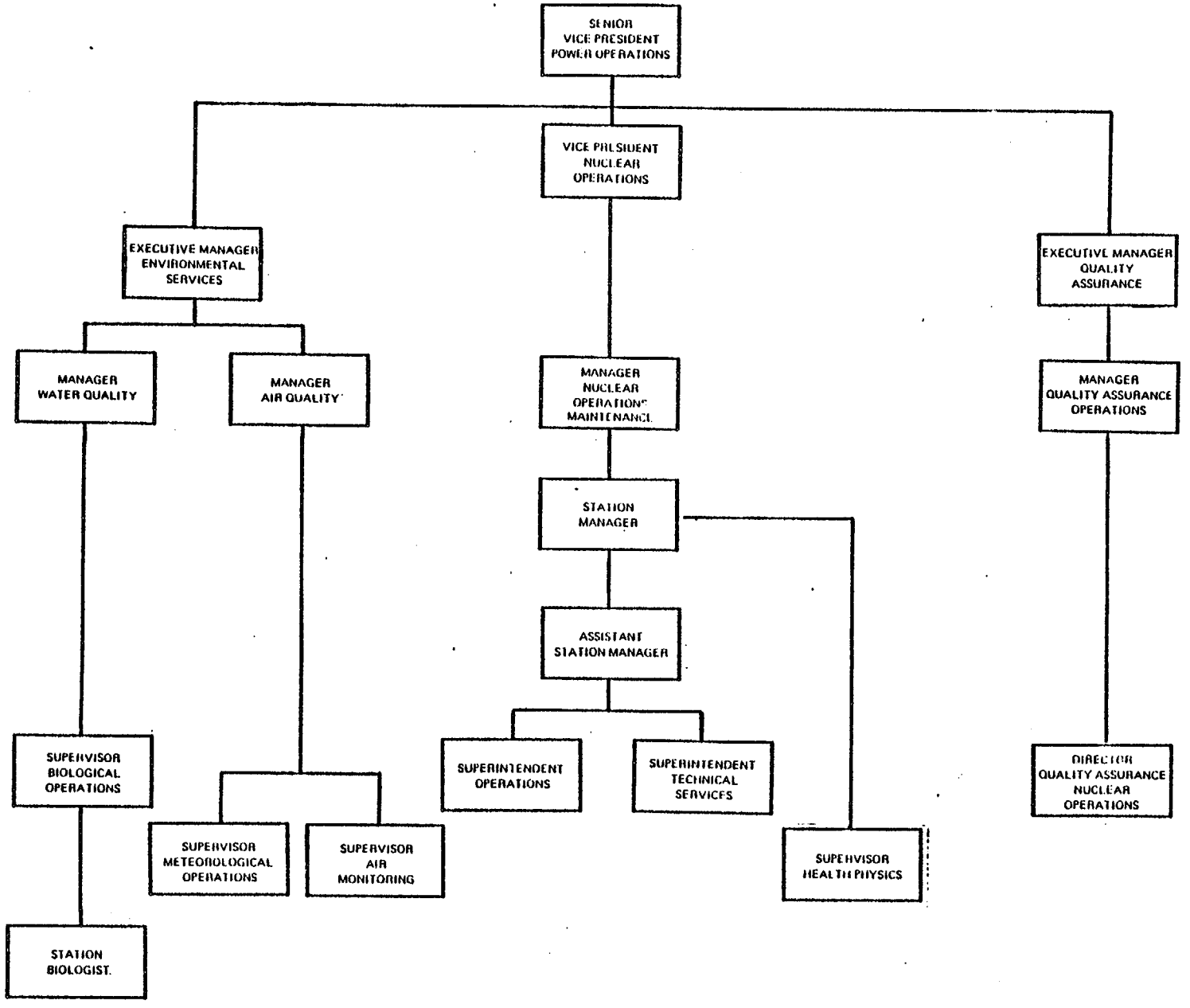
5.3.1.2 Responsibility

The SNSOC shall be responsible for:

a. Coordination of the Environmental Technical Specifications with the Safety Technical Specifications (Appendix A) to avoid conflicts and maintain consistency.

b. Review of changes to the Environmental Technical Specifications and the evaluation of the environmental impact of the change.

FIGURE 6.2-1
ORGANIZATION CHART
ENVIRONMENTAL TECH SPEC IMPLEMENTATION



c. Review of proposed written procedures required by Section 5.5.2 and 5.5.3 below and changes thereto, which affect the environmental impact of the station.

d. Review of proposed changes to station systems to determine the environmental impact of the changes.

e. Investigation of all reported instances of violation of the Environmental Technical Specifications; and where the investigation indicates, evaluation and formulation of recommendations to prevent recurrence.

f. Review of environmental monitoring programs to detect potential or existing significant adverse environmental impacts that have not been evaluated, or that are significantly greater than that evaluated by the Commission.

5.3.1.3 Authority

The SNSOC shall:

a. Review the environmental evaluation of all changes described in Section 5.3.1.2 a, b and c, above. When the evaluation indicates that such activitiy may result in a significant adverse environmental impact that was not evaluated, or that is significantly greater than that evaluated by the Commission, the SNSOC shall ensure that a written evaluation of such activities is provided to and prior approval is obtained from the Director of Nuclear Reactor Regulation for the activities.

b. If the SNSOC determines that unexpected harmful effects or evidence of irreversible damage are occurring as a result of operation of the station, the SNSOC shall ensure that an acceptable analysis of the problem and a plan of action to eliminate or significantly reduce the harmful effects or damage is submitted to the Commission for review and approval.

c. Review written reports prepared as a result of investigations and reviews conducted under 5.3.1.2 a, e and f.

5.3.1.4 Records

The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Director-Safety Evaluation and Control.

5.3.2 Quality Assurance Department

5.3.2.1 Function

The Quality Assurance Department shall perform independent audits of the implementation of the Environmental Technical Specifications.

5.3.2.2 Audits

The following audits shall be completed:

a. The conformance of facility operation to provisions contained within these Environmental Technical Specifications and applicable license conditions at least once per 12 months.

b. The performance, training and qualifications of the facility staff involved in ensuring and monitoring compliance with these Environmental Technical Specifications at least once per 12 months.

c. The results of action taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect the environmental impact of the station at least once per 12 months.

5.3.2.3 Records

Provide a written report of the results of the audits required by 5.3.2.2 above to the SNSOC, the Director-Safety Evaluation and Control, and the Station Manager.

5.3.3 Safety Evaluation and Control (SEC)

5.3.3.1 Function

SEC, as described in Section 6.5.2 of Appendix A of this license, shall function to provide independent review of designated activities related to the environmental impact of the station.

5.3.3.2 Review

SEC shall review and, where necessary, comment on the results of the reviews conducted by the SNSOC and the independent audits conducted by the Quality Assurance Department.

5.3.3.3 Responsibility

SEC has the responsibility for ensuring, through appropriate reviews, that the station is operated in accordance with the requirements of this license and applicable NRC regulations.

5.3.3.4 Authority

The Director-Safety Evaluation and Control shall report to and advise the Manager-Nuclear Technical Services, who shall advise the Vice President-Nuclear Operations on those areas relating to the environmental impact of the station.

5.3.3.5 Records

The records of SEC activities relating to the environmental impact of the station shall be prepared and maintained in the SEC files and a summary shall be disseminated as indicated below each calendar month.

1. Vice President - Nuclear Operations
2. Nuclear Power Station Managers
3. Manager-Nuclear Operations and Maintenance
4. Manager-Nuclear Technical Services
5. Manager-Quality Assurance, Operations
6. Others that the Director-Safety Evaluation and Control may designate.

5.4 State and Federal Permits and Certificates

None

5.5 Procedures

5.5.1 Written Procedures

Detailed written procedures, including applicable checklists and instructions, shall be prepared and followed for all activities involved in carrying out the Environmental Technical Specifications as defined in Section 5.5.2, 5.5.3, and 5.5.4, below. Procedures shall include sampling, data recording and storage, instrument calibration, measurements and analyses, and actions to be taken when limits are approached or exceeded. Testing frequency of any alarm shall be included. These frequencies shall be determined from experience with similar instruments in similar environments and from manufacturer's technical manuals.

5.5.2 Operating Procedures

Plant standard operating procedures shall include provisions, in addition to the procedures specified in Section 5.5.1, to ensure that all plant systems and components are operated in compliance with the Limiting Conditions of Operations established as part of the Environmental Technical Specifications.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 11
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendments by Virginia Electric and Power Company (the licensee) dated July 30, 1980, August 28, 1980 and October 15, 1980 as supplemented by letters dated August 1, 1980 and February 23, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 11 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective May 29, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 22, 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

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UNITED STATES
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   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 11 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective May 29, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 22, 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages
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ADMINISTRATIVE CONTROLS

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Station Manager shall be responsible for overall facility operation. In his absence, the Assistant Station Manager shall be responsible for overall facility operation. During the absence of both, the Station Manager shall delegate in writing the succession to this responsibility.

6.1.2 The Shift Supervisor (or during his absence from the Control Room, a designated individual) shall be responsible for the Control Room command function and shall be the only individual that may direct the licensed activites of licensed operators. A management directive to this effect, signed by the Executive Vice President-Power, shall be reissued to all station personnel on an annual basis.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.

b. At least one licensed Reactor Operator shall be in the control room when fuel is in the reactor. In addition, while the unit is in MODES 1, 2, 3 or 4, at least one licensed Senior Reactor Operator shall be in the Control Room.

c. A health physics technician# shall be on site when fuel is in the reactor.

d. ALL CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

e. A site Fire Brigade of at least 5 members shall be maintained onsite at all times#. The Fire Brigade shall not include the minimum shift crew shown in Table 6.2-1 or any personnel required for other essential functions during a fire emergency.

#The health physics technician and Fire Brigade composition may be less than the minimum requirement for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided immediate action is taken to fill the required positions.

6.2.3 SAFETY ENGINEERING STAFF (SES)

FUNCTION

6.2.3.1 The SES shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports, and other sources which may indicate areas for improving plant safety.

COMPOSITION

6.2.3.2 The SES shall be composed of at least five dedicated, full-time engineers located onsite.

RESPONSIBILITIES

6.2.3.3 The SES shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

AUTHORITY

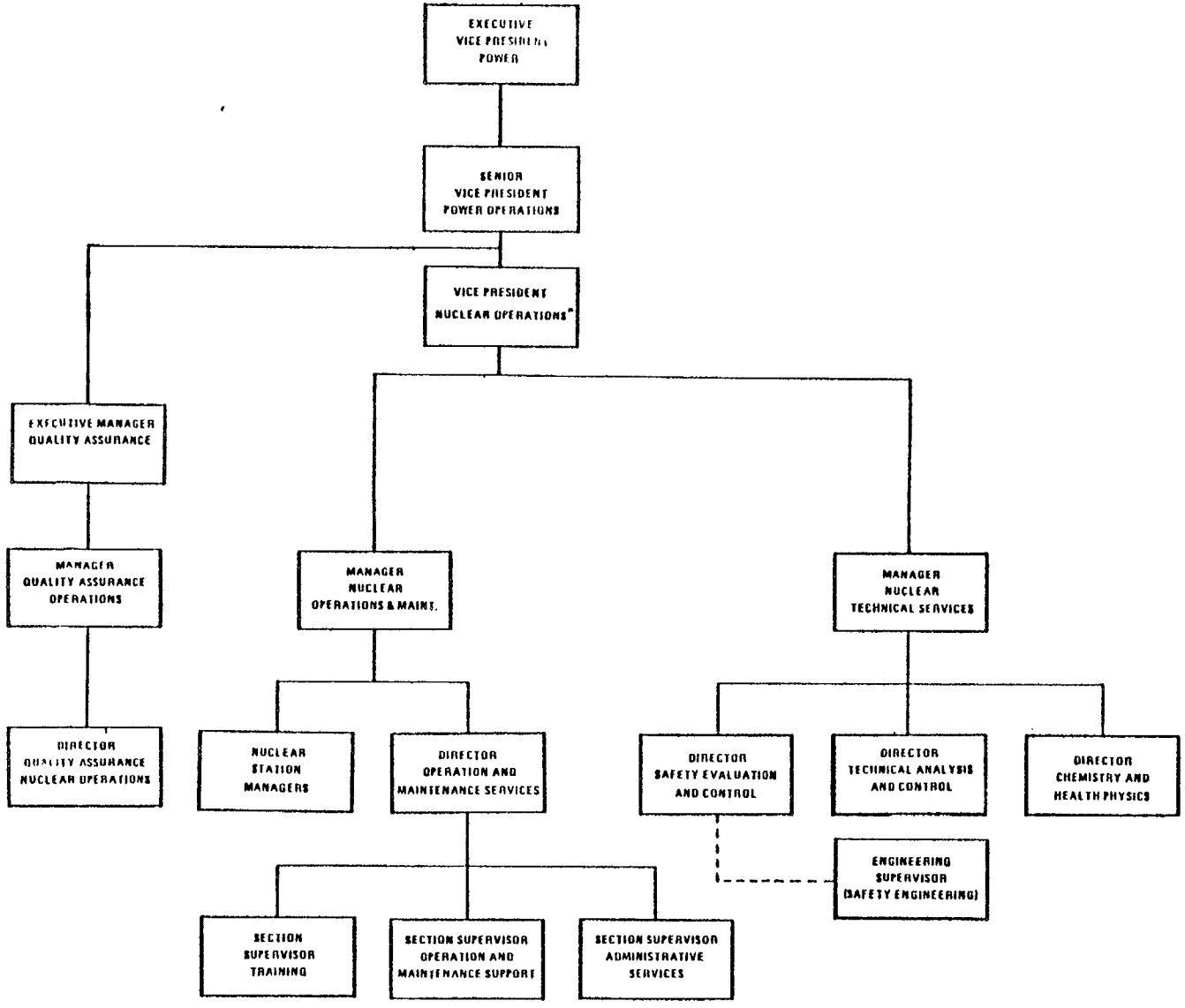
6.2.3.4 The SES shall make detailed recommendations for revised procedures, equipment modifications, or other means of improving plant safety to the Station Manager and the Director-Safety Evaluation and Control.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall serve in an advisory capacity to Shift Supervisor on matters pertaining to the engineering aspects of assuring safe operation of the unit.

6.2.4.2 The Shift Technical Advisor shall disseminate relevant operational experience identified by the SES.

*Not responsible for sign-off function.



*RESPONSIBLE FOR CORPORATE FIRE PROTECTION PROGRAM

Figure 6.2-1 Offsite Organization for Facility Management and Technical Support

Figure 6.2-2 Facility Organization - North Anna - Units 1 and 2

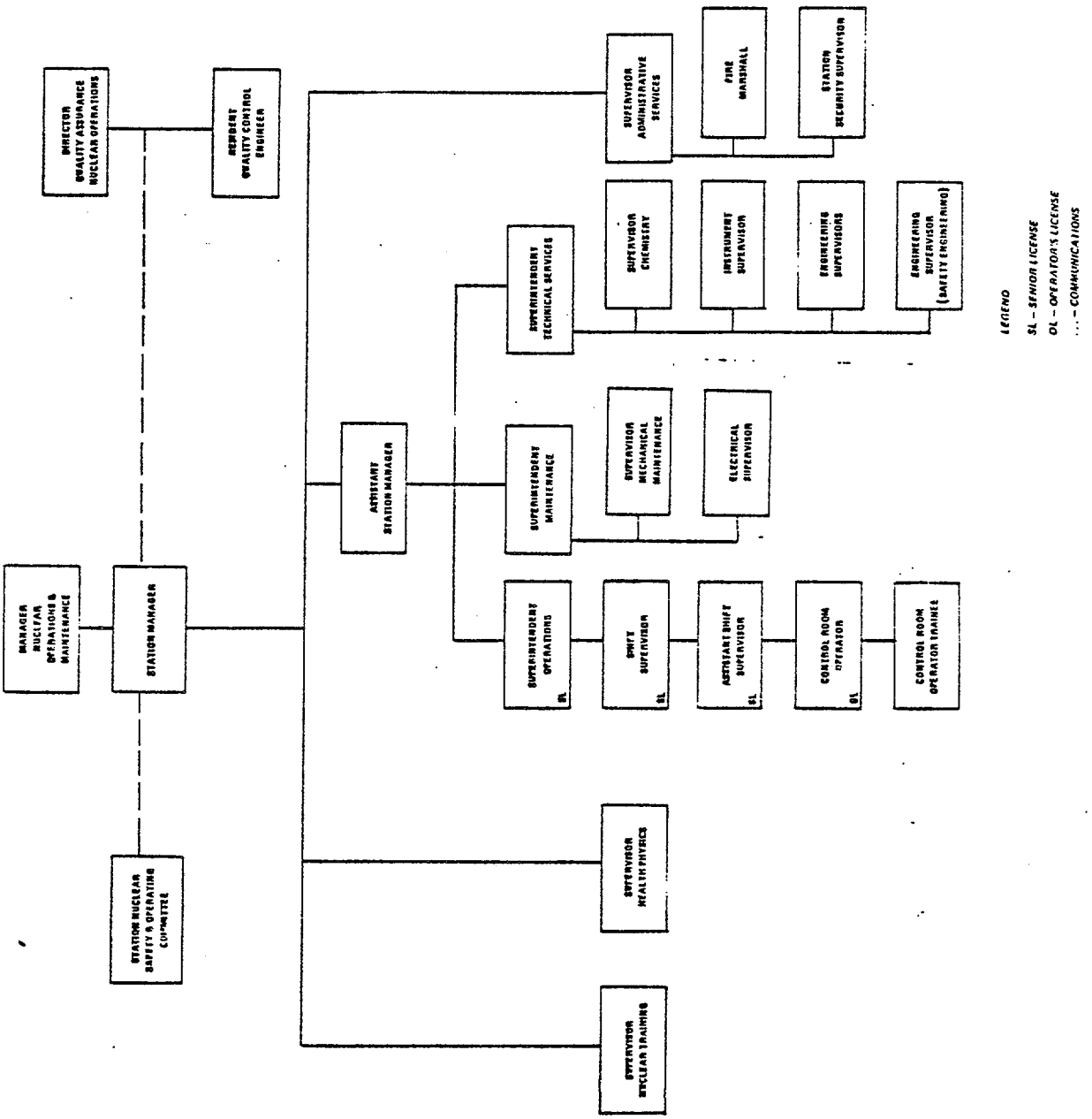


TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION

WITH UNIT 1 IN MODE 5 OR 6 OR DE-FUELED

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3, & 4	MODES 5 & 6
SS	1[a]	1[a]
SRO	1	none
RO	2	1
AO	2	2[b]
STA	1	none

WITH UNIT 1 IN MODES 1, 2, 3, OR 4

POSITION	NUMBER OF INDIVIDUALS REQUIRED TO FILL POSITION	
	MODES 1, 2, 3, & 4	MODES 5 & 6
SS	1[a]	1[a]
SRO	1[a]	none
RO	2[b]	1
AO	2[b]	1
STA	1[a]	none

a/ Individual may fill the same position of Unit 1.

b/ One of the two required individuals may fill the same position on Unit 1.

TABLE 6.2-1 (Continued)

SS - Shift Supervisor with a Senior Reactor Operators License on Unit 2.
SRO - Individual with a Senior Reactor Operators License on Unit 2.
RO - Individual with a Reactor Operators License on Unit 2.
AO - Auxiliary Operator
STA - Shift Technical Advisor

Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 6.2-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 6.2-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent.

During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 1, 2, 3 or 4, an individual (other than the Shift Technical Advisor) with a valid SRO license shall be designated to assume the Control Room command function. During any absence of the Shift Supervisor from the Control Room while the unit is in MODE 5 or 6, an individual with a valid RO license (other than the Shift Technical Advisor) shall be designated to assume the Control Room command function.

Licensed operators shall:*

1. Not work more than 12 hours straight,
2. Not work more than 24 hours in any 48-hour period,
3. Not work more than 72 hours in any 7-day period,
4. Not work more than 14 consecutive days without having 2 consecutive days off.

*Deviation from these requirements may be authorized by the Station Manager in accordance with established procedures and with documentation of the cause. Overtime limits do not include the shift turnover time.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1 - 1971 for comparable positions and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, except for (1) the Supervisor - Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 The Station Manager is responsible for ensuring that retraining and replacement training programs for the facility staff are maintained and that such programs meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1 - 1971 and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the SES.

6.5 REVIEW AND AUDIT

6.5.1 STATION NUCLEAR SAFETY AND OPERATING COMMITTEE (SNSOC)

FUNCTION

6.5.1.1 The SNSOC shall function to advise the Station Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The SNSOC shall be composed of the :

Chairman:	Station Manager
Vice Chairman:	Assistant Station Manager
Member:	Superintendent-Operations
Member:	Superintendent-Maintenance
Member:	Superintendent-Technical Services
Member:	Supervisor-Health Physics

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SNSOC Chairman to serve on a temporary basis; however, no more than one alternate shall participate as a voting member in SNSOC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the SNSOC shall consist of the Chairman or Vice-Chairman and two members including alternates.

RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

a. Review of 1) all procedures required by Specification 6.8.1 and changes thereto, 2) all programs required by Specification 6.8.4 and changes thereto, 3) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.

b. Review of all proposed tests and experiments that affect nuclear safety.

c. Review of all proposed changes to Appendix "A" Technical Specifications.

d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Manager-Nuclear Operations and Maintenance and the Director-Safety Evaluation and Control.

f. Review of events requiring 24 hour written notification to the Commission.

g. Review of facility operations to detect potential nuclear safety hazards.

h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee.

i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Station Nuclear Safety and Operating Committee.

j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Station Nuclear Safety and Operating Committee.

AUTHORITY

6.5.1.7 The SNSOC shall:

a. Recommend to the Station Manager written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.

b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.

c. Provide written notification within 24 hours to Manager-Nuclear Operations and Maintenance and the Director-Safety Evaluation and Control of disagreement between the SNSOC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Manager-Nuclear Operations and Maintenance and the Director-Safety Evaluation and Control.

6.5.2 SAFETY EVALUATION AND CONTROL (SEC)

FUNCTION

6.5.2.1 SEC shall function to provide independent review of designated activities in the areas of:

a. Nuclear power plant operations

b. Nuclear engineering

c. Chemistry and radiochemistry

d. Metallurgy

e. Instrumentation and control

f. Radiological safety

g. Mechanical and electrical engineering

h. Administrative controls and quality assurance practices

i. Other appropriate fields associated with the unique characteristics of the nuclear power plant

COMPOSITION

6.5.2.2 The SEC staff shall be composed of the Director-Safety Evaluation and Control and a minimum of three individuals who are qualified as staff specialists. Each SEC staff specialist shall have an academic degree in an engineering or physical science field and, in addition, shall have a minimum of five years technical experience in one or more areas given in Specification 6.5.2.1. These staff specialists shall not be directly involved in the licensing function.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the Director-Safety Evaluation and Control to provide expert advice to the SEC.

MEETING FREQUENCY

6.5.2.5 The SEC staff shall meet at least once per calendar month for the purpose of fostering interaction of reviews regarding safety-related operational activities.

REVIEW

6.5.2.7 The following subjects shall be reviewed by SEC:

a. Written safety evaluations of changes in the stations as described in the Safety Analysis Report, changes in procedures as described in the Safety Analysis Report and tests or experiments not described in the Safety Analysis Report which are completed without prior NRC approval under the provisions of 10 CFR 50.59(a)(1). This review is to verify that such changes, tests or experiments did not involve a change in the technical specifications or an unreviewed safety question as defined in 10 CFR 50.59(a)(2) and is accomplished by review of minutes of the Station Nuclear Safety and Operating Committee and the design change program.

b. Proposed changes in procedures, proposed changes in the station, or proposed tests or experiments, any of which may involve a change in the technical specifications or an unreviewed safety question as defined in 10 CFR 50.59(a)(2). Matters of this kind shall be referred to the Director-Safety Evaluation and Control by the Station Nuclear Safety and Operating Committee following its review prior to implementation.

c. Changes in the technical specifications or license amendments relating to nuclear safety prior to implementation except in those cases where the change is identical to a previously reviewed proposed change.

REVIEW (Cont'd)

d. Violations and reportable occurrences such as:

1. Violations of applicable codes, regulations, orders, Technical Specifications, license requirements or internal procedures or instructions having safety significance;

2. Significant operating abnormalities or deviations from normal or expected performance of station safety-related structures, systems, or components; and

3. Reportable occurrences as defined in the station Technical Specification 6.9.1.8.

Review of events covered under this paragraph shall include the results of any investigations made and recommendations resulting from such investigations to prevent or reduce the probability of recurrence of the event.

e. The Quality Assurance Department audit program at least once per 12 months and audit reports.

f. Any other matter involving safe operation of the nuclear power stations which a duly appointed subcommittee or committee member deems appropriate for consideration, or which is referred to the Director-Safety Evaluation and Control by the Station Nuclear Safety and Operating Committee.

g. Reports and meeting minutes of the Station Nuclear Safety and Operating Committee.

AUTHORITY

6.5.2.9 The Director-Safety Evaluation and Control shall report to and advise the Manager-Nuclear Technical Services, who shall advise the Vice President-Nuclear Operations on those areas of responsibility specified in Section 6.5.2.7.

RECORDS

6.5.2.10 Records of SEC activities required by Section 6.5.2.7 shall be prepared and maintained in the SEC files and a summary shall be disseminated as indicated below each calendar month.

1. Vice President-Nuclear Operations

2. Nuclear Power Station Managers

3. Manager-Nuclear Operations and Maintenance

RECORDS (Cont'd)

4. Manager-Nuclear Technical Services

5. Manager-Quality Assurance, Operations

6. Others that the Director-Safety Evaluation and Control may designate.

6.5.3 QUALITY ASSURANCE DEPARTMENT

FUNCTION

6.5.3.1 The Quality Assurance Department shall function to audit station activities. These audits shall encompass:

a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.

b. The performance, training and qualifications of the entire facility staff at least once per 12 months.

c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.

d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.

e. The Station Emergency Plan and implementing procedures at least once per 24 months.

f. The Station Security Plan and implementing procedures at least once per 24 months.

g. Any other area of facility operation considered appropriate by the Executive Manager-Quality Assurance or the Senior Vice President-Power Operations.

h. The Station Fire Protection Program and implementing procedures at least once per 24 months.

i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.

j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

AUTHORITY

6.5.3.2 The Quality Assurance Department shall report to and advise the Executive Manager-Quality Assurance, who shall advise the Senior Vice President-Power Operations on those areas of responsibility specified in Section 6.5.3.1.

RECORDS

6.5.3.3 Records of the Quality Assurance Department audits shall be prepared and maintained in the department files. Audit reports shall be disseminated as indicated below:

1. Nuclear Power Station Manager
2. Manager-Nuclear Operations and Maintenance
3. Manager-Nuclear Technical Services
4. Manager-Quality Assurance, Operations
5. Director-Quality Assurance, Nuclear Operations
6. Director-Safety Evaluation and Control
7. Supervisor of area audited
8. Nuclear Power Station Resident Quality Control Engineer

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the SNSOC and submitted to the Director-Safety Evaluation and Control and the Manager-Nuclear Operations and Maintenance

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

a. The facility shall be placed in at least HOT STANDBY within one hour.

b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Manager-Nuclear Operations and Maintenance, and the Director-Safety Evaluation and Control shall be notified within 24 hours.

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSOC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission, the Director-Safety Evaluation and Control and the Manager-Nuclear Operations and Maintenance within 14 days of the violation.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.

b. Refueling operations.

c. Surveillance and test activities of safety related equipment.

d. Security Plan implementation.

e. Emergency Plan implementation.

f. Fire Protection Program Implementation.

6.8.2 Each procedure of 6.8.1 above, and changes thereto, shall be reviewed by the SNSOC and approved by the Station Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

a. The intent of the original procedure is not altered.

b. The change is approved by two members of the plant supervisory staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.

c. The change is documented, reviewed by the SNSOC and approved by the Station Manager within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the recirculation spray, safety injection, chemical and volume control, gas stripper, and hydrogen recombiners. The program shall include the following:

(i) Preventive maintenance and periodic visual inspection requirements and

(ii) Integrated leak test requirements for each system at refueling cycle intervals or less.

c. Each REPORTABLE OCCURRENCE submitted to the Commission.

d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.

e. Records of changes made to Operating Procedures.

f. Records of radioactive shipments.

g. Records of sealed source leak tests and results.

h. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.

b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.

c. Records of facility radiation and contamination surveys.

d. Records of radiation exposure for all individuals entering radiation control areas.

e. Records of gaseous and liquid radioactive material release to the environs.

f. Records of transient or operational cycles for those facility components identified in Table 5.7-1.

g. Records of reactor tests and experiments

h. Records of training and qualification for current members of the plant staff.

i. Records of in-service inspections performed pursuant to these Technical Specifications.

j. Records of Quality Assurance activities required by the QA Manual.

k. Records of the service lives of all hydraulic snubbers listed on Table 3.7-4a including the date at which the service life commences. (This requirement commences with the Full Power License.)

l. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

m. Records of meetings of the SNSOC.

n. Records of meetings of the System Nuclear Safety and Operating Committee to issuance of Amendment No. _____.

o. Records of secondary water sampling and water quality.

p. Records for Environmental Qualification which are covered under the provisions of Paragraph 2.C(4)(e) of License No. NPF-7.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit.* Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.

c. An individual qualified in the protection procedures who is equipped with a radiation dose rate monitoring device. This individual shall be responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the facility Health Physicist in the Radiation Work Permit.

*Health Physics personnel or personnel escorted by Health Physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry in high radiation areas.

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "B" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Pages

I-i
I-ii
I-5-1
I-5-2
I-5-3
I-5-4
I-5-5

TABLE OF CONTENTS

TABLE OF CONTENTS (Cont'd)

5.0 ADMINISTRATIVE CONTROLS

Administrative controls below apply to Appendix B Part I only.

5.1 Responsibility

The responsibility for implementing the Environmental Technical Specifications is assigned to the Manager-Nuclear Operations and Maintenance at the corporate level and to the Station Manager at the station level. The Superintendent - Operations shall be responsible for ensuring that the station is operated in accordance with the Limiting Conditions of Operation. The Station Supervisor - Health Physics shall be responsible for the radiological environmental surveillance requirements. The Executive Manager of Environmental Services shall be responsible for providing services which will fulfill the non-radiological environmental surveillance requirements.

5.2 Organization

The relationship between the Nuclear Operations Department and the Environmental Services Department is shown in Figure 5.2-1.

5.3 Review and Audit

5.3.1 Station Nuclear Safety and Operating Committee (SNSOC)

5.3.1.1 Function

The SNSOC, as described in Section 6.5.1 of Appendix A of this license, shall function to advise the Station Manager on matters related to the environmental impact of the station. With the SNSOC is exercising its responsibility for non-radiological aspects of the ETS, the Station Biologist or his alternate shall be consulted.

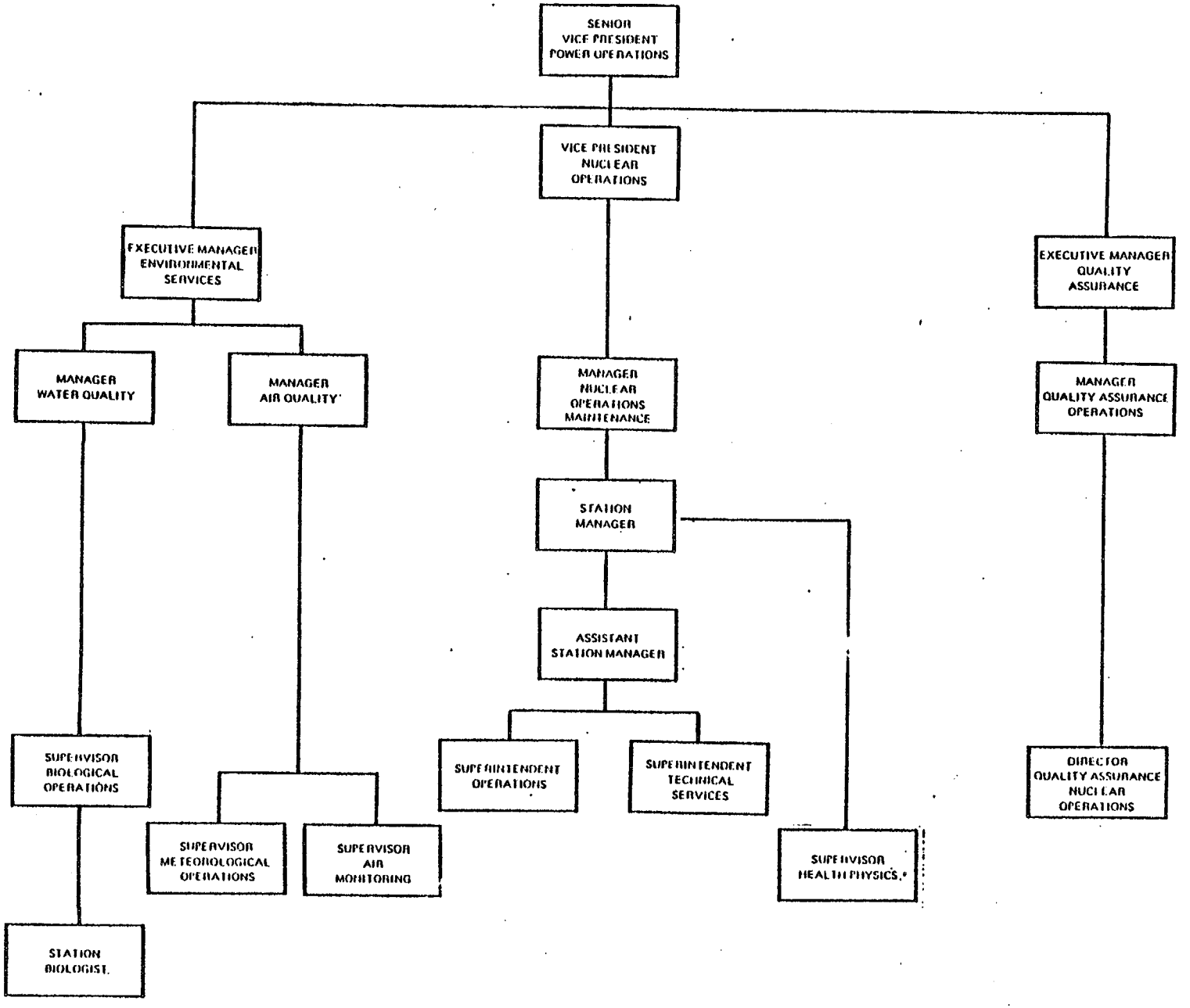
5.3.1.2 Responsibility

The SNSOC shall be responsible for:

a. Coordination of the Environmental Technical Specifications with the Safety Technical Specifications (Appendix A) to avoid conflicts and maintain consistency.

b. Review of changes to the Environmental Technical Specifications and the evaluation of the environmental impact of the change.

FIGURE 6.2-1
ORGANIZATION CHART
ENVIRONMENTAL TECH SPEC IMPLEMENTATION



c. Review of proposed written procedures required by Section 5.5.2 and 5.5.3 below and changes thereto, which affect the environmental impact of the station.

d. Review of proposed changes to station systems to determine the environmental impact of the changes.

e. Investigation of all reported instances of violation of the Environmental Technical Specifications; and where the investigation indicates, evaluation and formulation of recommendations to prevent recurrence.

f. Review of environmental monitoring programs to detect potential or existing significant adverse environmental impacts that have not been evaluated, or that are significantly greater than that evaluated by the Commission.

5.3.1.3 Authority

The SNSOC shall:

a. Review the environmental evaluation of all changes described in Section 5.3.1.2 a, b and c, above. When the evaluation indicates that such activitiy may result in a significant adverse environmental impact that was not evaluated, or that is significantly greater than that evaluated by the Commission, the SNSOC shall ensure that a written evaluation of such activities is provided to and prior approval is obtained from the Director of Nuclear Reactor Regulation for the activities.

b. If the SNSOC determines that unexpected harmful effects or evidence of irreversible damage are occurring as a result of operation of the station, the SNSOC shall ensure that an acceptable analysis of the problem and a plan of action to eliminate or significantly reduce the harmful effects or damage is submitted to the Commission for review and approval.

c. Review written reports prepared as a result of investigations and reviews conducted under 5.3.1.2 a, e and f.

5.3.1.4 Records

The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Director-Safety Evaluation and Control.

5.3.2 Quality Assurance Department

5.3.2.1 Function

The Quality Assurance Department shall perform independent audits of the implementation of the Environmental Technical Specifications.

5.3.2.2 Audits

The following audits shall be completed:

a. The conformance of facility operation to provisions contained within these Environmental Technical Specifications and applicable license conditions at least once per 12 months.

b. The performance, training and qualifications of the facility staff involved in ensuring and monitoring compliance with these Environmental Technical Specifications at least once per 12 months.

c. The results of action taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect the environmental impact of the station at least once per 12 months.

5.3.2.3 Records

Provide a written report of the results of the audits required by 5.3.2.2 above to the SNSOC, the Director-Safety Evaluation and Control, and the Station Manager.

5.3.3 Safety Evaluation and Control (SEC)

5.3.3.1 Function

SEC, as described in Section 6.5.2 of Appendix A of this license, shall function to provide independent review of designated activities related to the environmental impact of the station.

5.3.3.2 Review

SEC shall review and, where necessary, comment on the results of the reviews conducted by the SNSOC and the independent audits conducted by the Quality Assurance Department.

5.3.3.3 Responsibility

SEC has the responsibility for ensuring, through appropriate reviews, that the station is operated in accordance with the requirements of this license and applicable NRC regulations.

5.3.3.4 Authority

The Director-Safety Evaluation and Control shall report to and advise the Manager-Nuclear Technical Services, who shall advise the Vice President-Nuclear Operations on those areas relating to the environmental impact of the station.

5.3.3.5 Records

The records of SEC activities relating to the environmental impact of the station shall be prepared and maintained in the SEC files and a summary shall be disseminated as indicated below each calendar month.

1. Vice President - Nuclear Operations
2. Nuclear Power Station Managers
3. Manager-Nuclear Operations and Maintenance
4. Manager-Nuclear Technical Services
5. Manager-Quality Assurance, Operations
6. Others that the Director-Safety Evaluation and Control may designate.

5.4 State and Federal Permits and Certificates

None

5.5 Procedures

5.5.1 Written Procedures

Detailed written procedures, including applicable checklists and instructions, shall be prepared and followed for all activities involved in carrying out the Environmental Technical Specifications as defined in Sections 5.5.2, 5.5.3, and 5.5.4, below. Procedures shall include sampling, data recording and storage, instrument calibration, measurements and analyses, and actions to be taken when limits are approached or exceeded. Testing frequency of any alarm shall be included. These frequencies shall be determined from experience with similar instruments in similar environments and from manufacturer's technical manuals.

5.5.2 Operating Procedures

Plant standard operating procedures shall include provisions, in addition to the procedures specified in Section 5.5.1, to ensure that all plant systems and components are operated in compliance with the Limiting Conditions of Operations established as part of the Environmental Technical Specifications.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 30 TO FACILITY OPERATING LICENSE NO. NPF-4

AND AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. NPF-7

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION, UNITS NO. 1 AND NO. 2

DOCKET NOS. 50-338 AND 50-339

Introduction:

The Virginia Electric and Power Company (the licensee) has made application for amendments to Facility Operating License No. NPF-4 and No. NPF-7 for the North Anna Power Station, Units No. 1 and 2 (NA-1&2). The applications are dated July 30, August 28, and October 15, 1980 as supplelmented by letters dated August 1, 1980 and February 23, 198[illegible].

The proposed amendments consist of changes to the Technical Specifications which revise the Administrative Controls, Section 6. The changes reflect corporate and plant reorganizations including the addition of an Assistant Station Manager and establishment of a Safety Evaluation and Control group to replace the Systems Nuclear Safety and Operating Committee.

Discussion:

Technical Specifications Figure 6.2-1 has been revised to show the new structure of the offsite organization for facility management and technical support. The previous positions of Senior Vice President-Power and Vice President-Power Supply and Production Operations have been deleted from the Technical Specifications. The deleted positions have been replace by the Executive Vice President-Power, the Senior Vice President-Power Operations, and the Vice President-Nuclear Operations. All previous references to the Senior Vice President-Power and Vice President-Power Supply and Production Operations positions have been revised to reference either the Senior Vice President-Power Operations or the Vice President-Nuclear Operations positions.

As indicated by Figure 6.2-1, the Manager-Nuclear Operations and Maintenance and the Manager-Nuclear Technical Services replace the previous position of Director-Nuclear Operations. Both managers will report to the Vice President-Nuclear Operations. The Manager-Nuclear Operations and Maintenance has responsibility for the supervision of the Nuclear Station Managers. Both Nuclear Station Managers and the Director-Operation and Maintenance Services report to the Manager-Nuclear Operations and Maintenance. The Section Supervisor-Training, the Section Supervisor-Operation and Maintenance Support, and the Section Supervisor-Administrative Services will report to the Director-Operation and Maintenance Services. Reporting to the Manager-Nuclear Technical Services are the Director-Chemistry and Health Physics, the Director-Technical Analysis and Control, and the Director-Safety Evaluation and Control.

The August 28, 1980 submittal (Serial No. 731) proposed the transfer of the independent review responsibility from the existng System Nuclear Safety and Operating Committee (SyNSOC) to the Safety Evaluation and Control (SEC) staff and proposed the transfer of the System Nuclear Safety and Operating Committee audit responsibility to the Quality Assurance Department.

The Safety Evaluation and Control staff will be composed of a Director and a minimum of three staff members qualified to perform independent reviews. The Director-Safety Evaluation and Control will report to the Manager-Nuclear Technical Services, who will advise the Vice President-Nuclear Operations on the activities of the Safety Evaluation and Control Staff. This organizational structure will assure that sufficient attention is directed towards examination and evaluation of safety concerns and that management is cognizant of the results of these Safety Evaluation and Control staff reviews.

The Safety Evaluation and Control staff will provide the independent reviews of station operational activities required by ANSI N18.7-1976/ANS 3.2 (Administrative Controls and Quality Assurance Program for Operational Nuclear Power Plants), which are presently provided by the System Nuclear Safety and Operating Committee. The advantages of this proposed change are threefold.

1. The Safety Evaluation and Control staff is composed of individuals with direct nuclear experience in their respective technical disciplines. Presently, Safety Evaluation and Control staff specialists meet or exceed the criteria proposed in Technical Specification 6.1.C.2.b. The proposed requirement in Specificaiton 6.1.C.2.b exceeds the criteria for review "staff specialists" established in ANSI N18.1-1971/ANS 3.1-1978 (Standard For Selection and Training of Personnel for Nuclear Power Plants) and meets the present qualification position of the Commission. Additionally, the Director-Safety Evaluation and Control exceeds the criteria established by ANSI N18.1-1971/ANS 3.1-1978 for the Supervisor of an independent review staff. Future Safety Evaluation and Control staff members may not meet the qualification criteria for "staff specialists" positions; however, such individuals will not be directly responsible for the independent review function of Safety Evaluation and Control.

2. The independent reviews of the station operational activities by the Safety Evaluation and Control staff will be performed on a continuous basis as part of their routine responsibilities. This is in contrast to an intermittent or scheduled review committee approach; therefore, the Safety Evaluation and Control reviews will be responsive to station time constraints while also providing a more comprehensive review of items presently processed by the System Nuclear Safety and Operating Committee.

3. Other Safety Evaluation and Control staff routine responsibilities require continuous review of industry-wide operational experience, technical information, and regulatory issues. This effort should maintain a high level of "state-of-the-art" expertise within Safety Evaluation and Control regarding operational activities and general industry concerns. It is anticipated that the Safety Evaluation and Control staff will provide the majority of the expertise necessary to perform independent review activities; however, technical consultants and in-house specialists will be utilized for special concerns when Safety Evaluation and Control review requires additional expertise.

As a result of this change, safety will be enhanced by the proposed transfer of independent review responsibilities from the System Nuclear Safety and Operating Committee to the Safety Evaluation and Control staff by upgrading the quality and timely processing of the independent reviews of station operational activities.

Specification 6.5.2.8 for North Anna Units No. 1 and No. 2 requires the performance of audits of station activities under the cognizance of the System Nuclear Safety and Operating Committee. As a result of this proposed change, the audit responsibilities outlined in Specification 6.5.3.1, which are presently being performed under the congnizance of the System Nuclear Safety and Operating Committee by the Quality Assurance Department, will be transferred to the Ouality Assurance Department. Such a transfer will provide for appropriate organizational channels for the reporting of audit results, ensure the availability of appropriate technical expertise for the performance of these audits, and ensure that these audits are performed in an effective and timely manner.

As a result of the reorganization of the Licensing and Quality Assurance Department, the Licensing function has been transferred to Safety Evaluation and Control and the Quality Assurance organization has been restructured. The Executive Manager-Quality Assurance will be responsible for the quality assurance effort encompassing the areas of engineering, construction, and operational activities of both the fossil and the nuclear stations. He will also be responsible for the area of corporate emergency response planning and implementation. The Executive Manager-Quality Assurance will report to the Senior Vice President-Power Operations. The Manager-Quality Assurance, Operations will be responsible for implementing quality assurance programs which are related to the operational activities of the fossil and nuclear power stations. He will report to the Executive Manager-Quality Assurance. The Director-Quality Assurance, Nuclear Operations will be responsible for the implementation of quality assurance programs which are related to the operational activities associated with the nuclear power stations. He will report to the Manager-Quality Assurance, Operations.

The organization charts for the North Anna Environmental Technical Specifications and the Appendix "A" Technical Specifications have been revised to reflect the restructuring of the Quality Assurance Department and changes to other organization titles. The previous title of Executive Manager-Licensing and Quality Assurance has been revised to Executive Manager-Quality Assurance. In addition, the positions of Manager-Quality Assurance, Operations and Director-Quality Assurance, Nuclear Operations are shown on the organization chart.

Technical Specification Figure 6.2-2 has been revised to show the new structure of the North Anna Power Station organization. As explained above, the Station Manager reports to the Manager-Nuclear Operations and Maintenance. The licensee's July 30, 1980 submittal (Serial No. 487) proposes revised Section 6.0 to add the new position of Assistant Station Manager to the North Anna Power Station organization. The Assistant Station Manager will be directly responsible for the safe operation and maintenance of the power station. He will serve in a coordinating capacity to the Station Manager, for the off-site activities of the Station. During the absence of the Station Manager the Assistant Manager will act as Station Manager. The Assistant Station Manager will also be the Vice Chairman of the Station Nuclear Safety and Operating Committee (SNSOC). The Superintendent-Operations, the Superintendent-Maintenance, and the Superintendent-Technical Services will report to the Assistant Station Manager. All three of these superintendents will remain members of the SNSOC.

The positions of Operating Supervisor and Auxiliary Operator have been eliminated from the organization chart. In addition, the position title of Assistant Control Room Operator has been revised to Control Room Operator Trainee. The positions of Maintenance Coordinator and Mechanical Supervisor have been replaced on the Station organization chart by the position of Supervisor-Mechanical Maintenance. Both the Maintenance Coordinator and the Mechanical Supervisor report to the Supervisor-Mechanical Maintenance. Other title changes made to the organization chart include the change of Supervisor-Safety Engineering to Engineering Supervisor (Safety Engineering), the change of Engineering Supervisor to Engineering Supervisors to reflect the two positions of Engineering Supervisor-Performance and Tests and Engineering Supervisor-Design Changes and Projects and, the change of Training Supervisor to Supervisor-Nuclear Training. The Supervisor-Health Physics position has been shifted such that he will now report directly to the Station Manager instead of reporting to the Superintendent-Technical Services. The positions of Supervisor-Nuclear Training and Supervisor-Administrative Services will also report to the Station Manager.

Evaluation:

The proposed changes to the Administrative Controls, Section 6 of the NA-1&2 Appendix A Technical Specifications formalize the licensee's new corporate and plant organization. Based on our review, we conclude that the licensee's new organization is acceptable.

Also, we conclude that the Safety Evaluation and Control group will provide timely processing of the independent reviews of station operational activities.

Environmental Consideration:

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion:

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 22, 1981

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-338 AND 50-339

VIRGINIA ELECTRIC AND POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments No. 30 and No. 11 to Facility Operating License Nos. NPF-4 and NPF-7 issued to the Virginia Electric and Power Company (the licensee) for operation of the North Anna Power Station, Units No. 1 and No. 2 (the facility) located in Louisa County, Virginia. The amendments are effective on May 29, 1981.

The amendments consist of changes to the Technical Specifications which revise the Administrative Controls, Section 6. The changes reflect corporate and plant reorganizations including the addition of an Assistant Station Manager. A Safety Evaluation and Control group has been established which replaces the System Nuclear Safety and Operating Committee.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since these amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated July 30, August 28, October 15, 1980 as supplemented August 1, 1980 and February 23, 1981; (2) Amendment No. 30 and No. 11 to Facility Operating Licenses No. NPF-4 and NPF-7, respectively, and (3) the Commission's related Safety Evaluation. These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. 20555 and at the Board of Supervisor's Office, Louisa County Courthouse, Louisa, Virginia 23093 and at the Alderman Library, Manuscripts Department, University of Virginia, Charlottesville, Virginia 22901. A copy of items (2) and (3) may be obtained upon request to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 22nd day of May, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing